VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

March 19, 1980

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attn: Mr. A. Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Serial No. 203
PO/HSM:smv
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Dear Mr. Denton:

We have reviewed your letter of February 28, 1980 concerning the reactor coolant pump supports at Surry Power Station and your request that we inspect the supports during the Unit No. 1 steam generator repair outage, and have some concerns about the scope of the requested inspection program. To disassemble and inspect all of the maraging steel components as indicated in your letter, is a major undertaking which has the potential of adversely affecting the steam generator replacement schedule, accruing considerable occupational exposure, and increasing the cost of the project. We therefore propose to inspect one monoball assembly of the reactor coolant pump supports. If significant indications are discovered, the inspection program would be expanded to include additional assemblies.

In order to completely disassemble all of the reactor coolant pump supports and not impact the steam generator replacement schedule, the work must be done either right after removal of the reactor coolant pump motors and prior to the cutting of the reactor coolant piping or right after the completion of the reactor coolant pipe welding but prior to reinstallation of the reactor coolant pump motors. The inspection cannot take place during the time period that the reactor coolant pipe is partially disassembled due to the probability of disturbing the alignment of the reactor coolant pipe stub ends. It also should be pointed out that the reactor coolant supports are specialty items with very long lead times for some of their components (12 months). If any of the components were damaged by disassembly, it could adversely affect the steam generator repair program.

Occupational exposure should also be considered in a major undertaking such as this. During the Unit 2 steam generator replacement, Vepco, at your request, partially disassembled one of the steam generator supports with an expenditure of approximately 33 manrem. The disassembly of the reactor coolant pump supports would take place in the same area as the steam generator support disassembly took place. As the scope of the requested program is considerably more work than that performed in the steam generator support inspection, it can be expected that considerable occupational exposure will result.

In our letter of September 14, 1979, we supplied data showing that the operational stresses on the vascomax components in the reactor coolant pump supports are in all cases below the industry accepted threshold value (20 ksi) for the

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initiation of stress corrosion cracking. In addition, the inspection of the vascomax dowel pins in the steam generator supports showed no signs of unacceptable degradation.

We are presently preparing procedures to disassemble one leg assembly of one of the reactor coolant pump supports (see items 1-4 or 5-8, drawing 11448-FM-53A, provided in letter of September 14, 1979) and fully inspect one of the monoball assemblies. We will provide you with the details of this inspection as soon as the procedures are completed at least 45 days prior to conducting the inspection.

If you require any further information, we would be pleased to discuss this matter with your staff.

Very truly yours,

C. M. Stallings
Vice President - Power Supply
and Production Operations

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cc: Mr. James P. O'Reilly, Director
Office of Inspection and Enforcement