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02	Pipe stress reanalysis on IE Bulletin 79-14 lines (reactor cavity purification and seal
03	return) inside containment has revealed an overstressed condition on the piping pene-
04	trations would exist in the event of a Design Basis Earthquake. This could lead to
05	a loss of containment integrity and is reportable per Tech. Spec. 6.6.2.a.9. Once
06	the problem was recognized the reactor was brought to a cold shutdown condition and the
07	health and safety of the public were not affected.
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10	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Stress calculations on the affected lines indicate that the supports attached would not
	prevent an overstressed condition on the containment penetrations should a DBE occur.
12	Additional constraints have been designed and will be installed to relieve the overstress
13	disclosed.
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1 5 7 8	FACILITY % POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCOVERY DISCOVERY DESCRIPTION 32   9 10 12 13 44 45 46 50
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	NAME OF PREPARER W. L. Stewart PHONE: (804) 375-3184

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Attachment, Page 1 of 1 Surry Power Station, Unit 1 Docket No. 50-280 Report No. 80-018/01T-0 Event Date: 02/19/80 Title of Report: Excessive Stress on I.E. Bulletin 79-14 Lines Inside Containment

# 1. Description of Event:

During the pipe stress reanalysis it was determined that an overstressed condition would be imposed upon the containment piping penetrations for lines 3"-RL-6-152, 3"-CH-99-152, should a Design Basis Earthquake be experienced. This event is reportable in accordance with T.S. 6.6.2.a.9.

### 2. Probable Consequences & Status of Redundant Systems:

Left uncorrected, the problem could have led to a loss of containment integrity.

3. Cause:

The overstress condition was identified by the architect engineer in his reanalysis effort required by I.E. Bulletin 79-14, as an initial design inadequacy in the provision of constraints on the affected lines.

# 4. Immediate Corrective Action:

The unit was brought to a cold shutdown condition prior to the occurrence of a DBE so the health and safety of the public were not affected.

#### 5. Subsequent Corrective Action:

The required constraints have been designed and will be installed prior to future reactor operation.

#### 6. Future Corrective Action:

Further inadequacies, if any, in design features will be addressed in a similar fashion.

### 7. Generic Implications:

The reanalysis continues on all lines covered by I.E. Bulletin 79-14 and appropriate corrective actions will be taken if other overstressed conditions are identified.