



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION III
2443 WARRENVILLE RD. SUITE 210
LISLE, ILLINOIS 60532-4352

May 16, 2018

Mr. Bryan C. Hanson
Senior VP, Exelon Generation Co., LLC
President and CNO, Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3—NRC BIENNIAL
PROBLEM IDENTIFICATION AND RESOLUTION INSPECTION REPORT
05000237/2018010 AND 05000249/2018010

Dear Mr. Hanson:

On April 27, 2018, the U.S. Nuclear Regulatory Commission (NRC) completed the Biennial Problem Identification and Resolution inspection at your Dresden Nuclear Power Station, Units 2 and 3. On that date, the NRC inspectors discussed the results of this inspection with Mr. P. Karaba and other members of your staff. The results of this inspection are documented in the enclosed report.

Based on the results of this inspection, no violations or findings of significance were identified.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with 10 CFR 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Jamnes Cameron, Chief
Branch 4
Division of Reactor Projects

Docket Nos. 50-237; 50-249; 72-037
License Nos. DPR-19; DPR-25

Enclosure:
IR 05000237/2018010; 05000249/2018010

cc: Distribution via ListServ®

Letter to Bryan C. Hanson from Jamnes Cameron dated May 16, 2018

SUBJECT: DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3—NRC BIENNIAL
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05000237/2018010 AND 05000249/2018010

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket Numbers: 05000237; 05000249; 07200037

License Numbers: DPR-19; DPR-25

Report Number: 05000237/2018010; 05000249/2018010

Enterprise Identifier: I-2018-010-0014

Licensee: Exelon Generation Company, LLC

Facility: Dresden Nuclear Power Station, Units 2 and 3

Location: Morris, IL

Dates: April 9 through April 27, 2018

Inspectors: J. Gilliam, Reactor Engineer (Lead)
J. Bozga, Senior Reactor Engineer
J. Neurauter, Senior Reactor Engineer
J. Rutkowski, Senior Project Engineer
M. Porfirio, Resident Inspector (IEMA)

Approved by: J. Cameron, Chief
Branch 4
Division of Reactor Projects/Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring licensee's performance by conducting a Problem Identification and Resolution inspection at Dresden Nuclear Power Station, Units 2 and 3 in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations were identified.

Additional Tracking Items

None

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71152—Problem Identification and Resolution

The inspectors performed a biennial assessment of the licensee's corrective action program, use of operating experience, self-assessments and audits, and safety conscious work environment. The assessment is documented below.

- (1) Corrective Action Program (CAP) Effectiveness: Problem Identification, Problem Prioritization and Evaluation, and Corrective Actions (CA) – The inspection team reviewed the station's corrective action program and the station's implementation of the program to evaluate its effectiveness in identifying, prioritizing, evaluating, and correcting problems, and to confirm that the station was complying with NRC regulations and licensee standards for corrective action programs.
- (2) Operating Experience and Self-Assessments and Audits – The team evaluated the station's processes for use of industry and NRC operating experience information and the effectiveness of the station's audits and self-assessments.
- (3) Safety Conscious Work Environment – The team reviewed the station's programs to establish and maintain a safety-conscious work environment, and interviewed station personnel to evaluate the effectiveness of these programs.

INSPECTION RESULTS (findings/violation listed separately)

71152—Problem Identification and Resolution

Observation	71152
<p>Corrective Action Program: Based on the samples reviewed, the team determined that the licensee's performance in each of these areas adequately supported nuclear safety.</p> <p>Effectiveness of Problem Identification: Overall, the station was effective at identifying issues at a low threshold and was properly entering them into the CAP as required by station procedures. The team determined that the station was generally effective at identifying negative trends that could potentially impact nuclear safety. Action plans to address these items were reviewed by station leadership to ensure appropriate corrective actions were issued and the items were being resolved effectively to improve performance.</p>	

Effectiveness of Prioritization and Evaluation of Issues:

In-depth reviews of condition reports (CR), work orders, and cause evaluations were completed for a 5 year time period for the Nuclear Instrumentation (NI) system. This system includes Oscillation Power Range Monitors, Intermediate and Local Range Monitors, Emergency Diesel Generator (EDG) system and the Instrument Air (IA) system. The team determined that the licensee had established a low threshold for entering deficiencies into the CAP for these systems, that the items were generally being appropriately prioritized and evaluated for resolution, and that CAs were taken to mitigate the future risk of issues occurring that could affect overall system operation and/or reliability.

One observation the team provided the licensee was related to an action request (AR) screening to identify whether the concern was a potential aging issue. Specifically, AR 4108807 documented the identification of approximately 10 pin hole sized through wall leaks on radwaste service water piping, a non-safety-related system. Although the identified degradation may have been indicative of microbiologically induced corrosion (MIC), an aging issue, the team determined that the licensee had concluded the concern was not a potential aging issue, in part due to a misunderstanding that some non-safety-related systems may be within the scope of the licensee's Aging Management program. Upon subsequent review of the applicable license renewal boundary diagram of service water piping, the licensee determined that the concern affected a section of service water piping that was within the bounds of Aging Management scope and should have been screened as a potential aging issue.

As a result, the licensee initiated issue report (IR) 4131127. Immediate corrective action: requested AR 4108807 be rescreened. Recommended corrective actions included: share lesson learned at an engineering meeting that non-safety related systems can contain components that are within Aging Management scope, and to perform an Engineering evaluation that includes an extent of condition review.

Effectiveness of Corrective Actions:

The team concluded that the licensee was generally effective in identifying CAs that were appropriately focused to correct the problem and to address the root and contributing causes for significant conditions adverse to quality to preclude repetition. The licensee generally completed CAs in a timely manner and in accordance with procedural requirements commensurate with the safety significance of the issue. For NRC-identified issues, the team determined that the licensee generally assigned CAs that were effective and timely

Under this assessment area, the inspectors also completed a review of the remaining CAs and effectiveness reviews that were open at the time of the Supplemental Inspection related to the White finding and associated violation for the failure of the Unit 3 High Pressure Coolant Injection Auxiliary Oil Pump Motor (reference NRC Inspection Reports 05000249/2016010 and 05000249/2017010). The team did not identify any issues during this review.

Observation	71152
Operating Experience and Self-Assessments and Audits: Based on the samples reviewed, the team determined that the licensee's performance in each of these areas adequately supported nuclear safety.	

Observation	71152
Safety Conscious Work Environment: The team found no evidence of challenges to the licensee's safety-conscious work environment. Licensee employees appeared willing to raise nuclear safety concerns through at least one of the several means available.	

EXIT MEETINGS AND DEBRIEFS

The inspectors confirmed that proprietary information was controlled to protect from public disclosure. No proprietary information was documented in this report.

- On April 27, 2018, the inspectors presented the Biennial Problem Identification and Resolution inspection results to Mr. P. Karaba, and other members of the licensee staff.

DOCUMENTS REVIEWED

71152—Problem Identification and Resolution

Corrective Actions

- AR 2454690; Security – Management Model Alignment PIIM Review; 02/18/15
- AR 2647226; Security – Contractor Onsite Without Contract in Place; 03/29/16
- AR 3943151; Security – Explosive Detector #1 OOS – Attempting to Connect; 11/20/16
- AR 2741013; Security – Bravo Crew Discretionary Reset; 11/14/16
- AR 4018054; Security – Admin Baseboard Heater Emitted Flame; 06/03/17
- AR 3943627; Delivery Vehicle Contact Staged Pipe; 11/21/16
- AR 3948301; NOS ID: Quality Records were Lost and Another was Incomplete; 12/01/16
- AR 3964435; DOS 6600-12 not Revised per ACIT; 01/18/17
- AR 2738559; D3R24LL: TS 3.6.4.1 Entry / ENS Notification (RB D/P> -0,25'); 11/08/16
- AR 3944279; Security: CCTV Camera Repair Delay; 11/22/16
- AR 2656586; First Level Security CFAM Elevation Received; 04/13/16
- AR 2633927; Security – XF 0607 Supervisory Alarm; 02/29/16
- AR 2633928; Security – Constant Alarm of XF05 Supervisory Alarm; 03/01/16
- AR 2741310; Security – Door 3A OOS for Hardware Constant Alarm; 11/15/16
- AR 2742160; Security – Door at CAS; 11/17/16
- AR 2633913; Security – F150 #3 Door Issue; 02/29/16
- AR 3967635; Security – ACP AVB(S) Out of Service – Failed Test; 01/28/17
- AR 4034884; CAP Issues: Adverse Trend Identified in a WGE; 08/24/2017
- AR 4020211; Unexpected Increase Observed between Unit 2 and Unit 3 Condenser Megawatt Electric after Circulating Water Flow Reversal; 09/15/2017
- AR 4043044; Action From SCMP Meeting for Second Quarter 2017; 07/21/2017
- AR 3983380; Work Order Performed Incorrectly; 03/08/2017
- AR 2593505; 2016 NOS Audit Performance and Effectiveness Review; 01/20/2017
- AR 2703314-09; RCR to Operations to Address WANO AFI; 10/03/2016
- AR 2703335; 2016 AFI MA.1; Maintenance Fundamentals, Quality Maint; 08/11/2016
- AR 2703314; 2016 AFI OP.1, Operations Fundamentals; 10/03/2016
- AR 3959767; TREND: Actions from Dec 2016 Semi-Annual Safety Culture MTG; 01/05/2017
- AR 399397; Biennial Self-Assessment & Benchmark Program Self-Assessment; 07/03/2017
- AR 4022417; CAP CA Closure Quality; 08/28/2017
- AR 4043977; Maintenance & Technical Training Gap to Excellence PIIM Focus; 08/21/2017
- AR 4040220; Actions Resulting from Semi-Annual Safety Culture Meeting; 08/16/2017
- AR 4086283; Preparation for NRC PI&R Inspection; 02/12/2018
- AR 4074551; 3Q SCMP Meeting; 11/16/2017

- AR 4082859; TREND IR: Safety OSHA Trends; 12/11/2017
- AR 3993704; Biennial "Self-Assessment & Benchmark Program" Self-Assessment; 07/03/2017
- AR 347338; DG Ventilation Calculation Weaknesses; 06/24/2005
- AR 672183; 2/3A SBTG Flow Swinging Excessively; 09/17/2007
- AR 893211; While Placing C/O 71240, MO 2-2301-6 did not Fully [Position] Closed; 03/15/2009
- AR 1070607; NRC CDBI: LPCI and CS Pump Seals should be Safety-Related; 05/18/2010
- AR 1350392; No Scheduled Replacement for AOV Positioners; 04/05/2012
- AR 1355545; 3A CCSW Vault Cooler Drain Line Partially Plugged; 04/18/2012
- AR 1620074; OPEX: Monticello Event Requires Possible Actions at Dresden; 02/12/2014
- AR 1624708; Trng ID: DOA 4450-03 Requires Revision; 02/22/2014
- AR 2518780; DEOP 500-02 for Re-Opening MSIVs Not Adequate; 06/24/2015
- AR 2537446; Evaluate Creation of a DOA for Recirc Pump Failure (Training); 08/04/2015
- AR 2586123; Piping Break Near U2 G X Area Cooler; 11/12/2015
- AR 2615854; Dose Rates Higher than Expected during 3B CFS Transfer; 01/22/2016
- AR 2699953; 2A EHC PP Discharge Filter Housing; 08/02/2016
- AR 2701933; Initiate Abandonment of 2/3 Radwaste Reboiler; 08/08/2016
- AR 2738160; Turbine Runback Temperature Switch Found Out of Tolerance; 11/07/2016
- AR 2740503; 2/3 Cribhouse Screen Refuse Pit Pump Motors Wetted; 11/13/2016
- AR 3998073; NSO Questions Recent Procedure Changes; 04/14/2017
- AR 4027245; 1-7319-B3 Tripped Free when Restarting Chem Cleaning; 6/30/2017
- AR 4046450; CA/CAPR Doc Not being Stand Alone or Completed Properly; 8/28/2017
- AR 4049215; Unexpected Alarm U3 West LPCI/CS Sump Lvl Hi; 09/05/2017
- AR 4049289; Recommendation of Clean Out All Four Corner Room Sumps; 09/06/2017
- AR 2639257; Level-1 PCE; 03/11/2016
- AR 2702494; 3 Level 1 PCEs and 1 Level 2 PCE from Maximum Recycle; 08/09/2016
- AR 2637388; RP FLS Identified B-25 Boxes Not RAM Tagged; 03/08/2016
- AR 2637586; Sealand #5198 Failed Shipping Inspection; 03/08/2016
- AR 2638621; U2 Service Air CO Monitor Giving False Readings; 03/10/2016
- AR 2640724; Respirator Issuance for Security; 03/15/2016
- AR 2646936; U3 Drywell CAM Heat Trace Is Reading Below Set Value; 03/29/2016
- AR 2650493; Legacy Leak from CRD Vent Filter; 04/04/2016
- AR 2650946; D2 CO Monitor Does Not Calibrate Post Maintenance; 04/05/2016
- AR 2657259; Locked HIGH RAD and HIGH RAD Door/Barrier Deficiencies; 04/18/2016
- AR 2659116; Trend IR: Level 1 PCE; 04/21/2016
- AR 2659179; U2 CO Monitor Calibration Failed; 04/21/2016
- AR 2664017; ACE on PCEs; 05/02/2016
- AR 2664017-02; ACE: Dry Cask PCEs on 04/28/2016; 05/27/2016
- AR 2683473; U2 LHRA Door Latch Needs Repair; 06/20/2016
- AR 2686422; Repair Door Seal 2/3 Reactor Interlock Door; 06/28/2016
- AR 2702494; 3 Level 1 PCEs and 1 Level 2 PCE from Maximum Recycle; 08/09/2016
- AR 2702494-03; ACE: Four PCEs Occurred While Performing Decontamination Outside of Maximum Recycle; 09/12/2016
- AR 2703713; RP FASA Identifies Deficiency; 08/12/2016
- AR 2728202; Deficiencies Found during RP-DR-463 Surveillance; 10/14/2016
- AR 2736271; Trend IR: Dresden Exceeds D3R24 PCE Goal; 11/02/2016
- AR 2740952; RP-AB-460 Needs an Enhancement to Preclude Confusion; 11/14/2016
- AR 3943636; Initiate Review of D3R24 PCEs; 11/21/2016
- AR 3943636-02; Radiation Protection Investigation: D3R24 Personnel Contamination Events (PCEs); 02/23/2017

- AR 3975315; Bay 5 in Barreling Area Has Increased Dose Emitting from It; 02/17/2017
- AR 4044311; Extra Dose Received for 3A CPF Transfer; 08/22/2017
- AR 4070223; U3 Drywell CAM Issues; 11/02/2017
- AR 2670087; Steam Leak on HPCI Piping Upstream of 2-2301-29 AOV; 05/16/2016
- AR 2672852; ECOLOCHEM Failed to Trip on High Conductivity; 05/23/2016
- AR 2695912; EACE 2672852 Rejected by MRC with a Grade of 1; 07/22/2016
- AR 2703314; WANO AFI OP.1 Operations Fundamentals; 08/11/2016
- AR 2703314-09; RCE: WANO AFI OP.1 Operations Fundamentals; 10/03/2016
- AR 2703841; Corrective Action Determined Ineffective during EFR; 08/12/2016
- AR 2738040; LLRT Exceeds Warning Limit 3-8501-1B; 11/07/2016
- AR 2738559; D3R24LL: TS 3.6.4.1 Entry / ENS Notification (RB D/P > -0.25"); 11/08/2016
- AR 4040248; U2 EDG Oil Temperature Low; 08/08/2017
- AR 4076133; Unexpected Turbine/Generator Trip During Startup; 11/18/2017
- AR 2735005; Plant Walkdown Issues Identified; 10/31/2016
- AR 3952013; Bolts Missing from U2 EDG; 12/12/2016
- AR 3962319; Combustible Material in Cable Tray; 01/12/2017
- AR 2634423; U3 EDG Missing Screws; 03/01/2016
- AR 2634881; U2 EDG Missing Screws; 03/02/2016
- AR 2636169; 3 Pipe Hangers Broken on 3-4699-961; 03/04/2016
- AR 2638684; U2 EDG Louver Door Is Missing 3 of 13 Screws; 03/10/2016
- AR 2639082; 2B TBCCW Cooling Water Pump Has High Vibes; 03/11/2016
- AR 2651986; 3B H2 Analyzer Failed Surveillance; 04/07/2016
- AR 2653534; 3A CCSW Pump Discharge Valve Could Not Be Closed; 04/10/2016
- AR 2659864; U2 RBCCW Supply Header to SDC Room Broken Pipe Hanger; 04/22/2016
- AR 2691967; U2 A SBO LB Jacket Water Pump Coolant Leak; 07/12/2016
- AR 2738559; D3R24LL: TS 3.6.4.1 Entry/ENS Notification (RB D/P > -0.25"); 11/08/2016
- AR 2738559-11; ACE: Loss of Secondary Containment and Unplanned Entry into TS 3.6.4.1; 12/28/2016
- AR 4015141; IA Valve to 2-1601-60 Found Out of Position; 05/26/2017
- AR 4015141-05; CAPE: U2 Torus Air Operated Valve to Reactor Building Vent System Failed to Operate; 07/06/2017
- AR 4108807; Through Wall Leaks on Radwaste Service Water Piping; 02/27/2018
- AR 2651479; EOP Project – UFSAR Change for New HPCI Temperature Limit; 04/06/2016
- AR 2732769; NRC Report 2016003 3rd Quarter 2016; 10/26/2016
- AR 2652711; NOS ID: C urrent X/Q Values Not Used for EAL Threshold Calculations; 04/08/2016
- AR 3968580; Receipt of NRC 4Q2016 Report and NCV for EP; 01/31/2017
- AR 2666210; NRC: Eagle Air Compressor CO Monitor OOS; 05/06/2016
- AR 2697341; NRC Report 2016-002 2nd QTR 2016; 07/27/2016
- AR 2735594; ED Dose Rate Alarm; 11/01/2016
- AR 2735651; ED Dose Rate Alarm; 11/01/2016
- AR 3968586; Receipt of NRC 4Q2016 Report and NCV for RP; 01/31/2017
- AR 04003943; Receipt of NRC 1Q2017 Inspection Report; 04/28/2017
- AR 02501498; TSSR 3.8.1.10: EDG Largest Load Reject; 05/15/2015
- AR 4041350; Root Cause Investigation Report for Issues in M&T Training; 09/29/2017
- AR 2670087; Root Cause for Unit 2 HPCI Piping Steam Leak; 09/06/2016
- AR 2633687; Apparent Cause Investigation Report for 2016 NOS Maintenance Audit; 03/31/2016
- AR 4022757; Perform OPEX Review IN 2017-03; 08/17/2017
- AR 4128239; 2018 PI & R; Docmntn Incomplete – Resolution D2R25 IR 4068782; 04/18/2018
- AR 4130673; 2018 PI & R: Conduit Support DWG for U2 1D INBD MSIV LS; 04/25/2018

- AR 3987373; Quarterly IST Stroking AOV 2(3)-220-44 Valve In Containment; 03/20/2017
- Action Request 2521197; Software Error Potentially Adverse to DW Support M-1156D-2; dated 06/29/2015
- AR 2592034; NRC RIS 2015-13, Seismic Stability Analysis Methodologies for Spent Fuel Dry Cask Loading Stack-up Configuration; 12/10/2015
- AR 2733434; RWP DR-03-16-00520 for Drywell Snubber; dated 10/27/2016
- AR 2735326; Support Needed for Small Bore Piping in U3 Drywell; 11/01/2016
- AR 2735725; Disconnected Unistrut Observed for CRD Piping in Drywell; 11/01/2016
- AR 4068782; MSIV 2-0203-1D Limit Switch Conduit Support Nut Missing; 10/30/2017
- AR 0981877; LR-NRC Identified-SBLC Tank Wall Thickness Below Min; 10/20/2009
- AR 2684933; Maintenance EMD ACE 2670555 Rejected by MRC; 06/23/2016
- AR 4102554; U3 EDG Maintenance Tasks for Mechanical Inspection; 02/09/2018
- AR 2643951; Maintenance Needed on 2-0220-58A; 03/22/2016
- AR 3983097; Abbreviated Maintenance on D2 EDG Air Housing; 03/08/2017
- AR 3977686; NOS Id In-Storage Maintenance Not Done on HPCI AOP Motor; 02/23/2017
- AR 2671632; Flex Equipment Maintenance; 05/19/2016
- AR 2657517; Create WO for MOV 2-1301-3 Maintenance; 04/18/2016
- AR 4104998; Abbreviated Maintenance on 2B Service Water Coupling; 02/16/2018
- AR 4098930; MSIV Seismic Classification; 01/30/2018
- AR 4098927; MSIV Limit Switch Support Seismic Qualification; 01/30/2018
- AR 4069785; TRM Snubber 2-3019E-58 Failed its Functional Test (D2R25); 11/01/2017
- AR 2642339; Event 52003 – Switches Not Environmentally Qualified; 07/14/2016
- AR 2642339; Event Number 52212, Part 21 Notification – Flowserve Swing Check Valves; 12/12/2016
- AR Event No. 52407 – Part 21, Non-Conforming Cover Machining on Solenoid Valve; 06/13/2017
- AR 2642339; 47818 – Part 21 Report – HK Circuit Breaker Studs Failed to Meet Specification; 06/20/2017
- AR 2503008; Interim Part 21 Report – STAAD Software Errors; 06/05/2015
- AR 2563163; NRC Information Notice 2015-09: Mechanical Dynamic Restraint (Snubber) Lubricant Degradation not Identified due to Insufficient Service Life Monitoring; 11/23/2015
- AR 2704548; NRC Information Notice 2016-11; Potential for Material Handling Events to Cause Internal Flooding; 11/08/2016
- AR 02688163; U3 HPCI AUX Oil Pump on Fire; 06/27/2016

Self-Assessments/Benchmarks/Audits

- NOSA-DRE-18-02; Security Programs Audit Report; 02/07/18
- NOSA-DRE-17-02; Security Programs Audit Report; 02/07/17
- NOSA-DRE-17-04; Correct Action Program Audit Report; 03/03/2017
- 3Q17 Dresden Safety Culture Monitoring Panel (SCMP) Report; 11/16/2017
- 4Q17 Dresden Safety Culture Monitoring Panel (SCMP) Report; 02/15/2018
- Semi-Annual Safety Culture Review; 08/16/2017
- NOSA-DRE-17-06; Radiation Protection Audit Report, Dresden Station; 06/28/2017
- NOSA-DRE-17-08; Operations Functional Area Audit, Dresden Station; 10/18/2017
- AR 3977906; FASA: (RP) Hazard Assessment, ALARA Planning & Controls; 08/30/2017
- AR 3977939; Self Assessment: 2017 Contamination Control; 08/31/2017
- AR 3978032; Self Assessment: 2017 Control of Radioactive Material; 01/26/2018
- NOSA-DRE-16-09; Inservice Inspection, Inservice Testing, and Appendix J Audit Report; 09/28/2016
- NOSA-DRE-16-05; Engineering Programs and Station Blackout Audit Report; 05/11/2016

- NOSA-DRE-17-05; Engineering Design Control Audit Report; 07/19/2017
- NOSA-DRE-18-01; Maintenance Functional Area Audit Report; 02/21/2018
- Dresden Organizational Effectiveness Survey; 07/25/20173964393; Maintenance Self-Assessment – Predictive Maintenance; 04/18/2017
- 3993787; Operations Self-Assessment – Operator Fundamentals; 10/13/2017
- 3993796; Operations Self-Assessment – Configuration Control; 10/13/2017
- 4032862; Maintenance Self-Assessment – Maintenance Work Preparation; 09/26/2017
- Maintenance Monthly Trending Report; Dec. 2016, Dec. 2017, and Jan. 2018
- Operations Monthly Trending Report; Dec. 2016, Dec. 2017, and Jan. 2018
- Dresden Organizational Effectiveness Survey; 07/25/20173964393; Maintenance Self-Assessment – Predictive Maintenance; 04/18/2017
- 3993787; Operations Self-Assessment – Operator Fundamentals; 10/13/2017
- 3993796; Operations Self-Assessment – Configuration Control; 10/13/2017
- 4032862; Maintenance Self-Assessment – Maintenance Work Preparation; 09/26/2017

Procedures

- PI-AA-125; Corrective Action Program (CAP) Procedure; Revision 6
- PI-AA-120; Issue Identification and Screening Process; Revision 8
- EI-AA-101-1001; Employee Concerns Program Process; Revision 15
- DAP 13-03; Unit 2 Reactor Building Trackway Interlock Door Access Control; Revision 25
- ER-AA-700; License Renewal Implementation Program; Revision 6
- ER-AA-700-1003; Screening and Evaluation of Potential Aging Issues; Revision 4
- ER-AA-2002; System and Equipment Health Monitoring; Revision 21
- PI-AA-125-1001; Root Cause Analysis Manual; Revision 3
- PI-AA-125-1003; Corrective Action Program Manual; Revision 4
- PI-AA-126; Self-Assessment Benchmark Program; Revision 2
- PI-AA-126-1001; Self-Assessments; Revision 2
- PI-AA-126-1001-F-01; Self-Assessment; Revision 2
- RP-AA-350; Personnel Contamination Monitoring, Decontamination and Reporting; Revision 18
- DMP 5800-18; Load Handling of Heavy Loads and Lifting Devices; Revision 28
- MA-AA-716-022; Control of Heavy Loads Program; Revision 12
- MA-AA-716-021; Rigging and Lifting Program; Revision 28

Five-Year System Review: Nuclear Instrumentation (Oscillation Power Range Monitors)

- AR 1515941; NRC Questions PINV for U3 OPRM 1/2 Scram; 05/20/2013
- AR 1600984; OPRM Will Not Communicate with Maintenance Terminal; 12/26/2013
- AR 2409245; 'A' Channel Half Scram Reset Prevented by OPRM 7; 11/09/2014
- AR 2423937; OPRM Power Supply LED Failed to Light; 12/11/2014
- AR 2737798; OPRM #7 Power Supply; 11/06/2016
- AR 4022909; Unexpected Unit 2 OPRM Trouble Alarm; 06/17/2017
- AR 4056222; PMC: OPRM Review; 09/27/2017
- AR 4112084; OPRM 6 Power Supply Failure; 03/07/2018

Five-Year System Review: Nuclear Instrumentation (Intermediate and Local Range Monitors)

- Intermediate Range Monitor Engage Health Report
- Engineering Change 379745; Evaluate / Justify Disconnecting Shorted LPRMs / DOP 0700-08; 12/02/2010
- Work Order 4652483; U3 IRM 15 Spiking; 02/26/2018
- Work Order 4746973; IRM 15 Does Not Pass Channel Check; 02/22/2018
- Work Order 4747662; IRM 14 Inop; 03/23/2018

- AR 2454544; DOP 0500-10 Change Evaluation for IRM Bypass; 02/18/2015
- AR 2618377; Adverse Trend SRM & IRM System; 01/28/2016
- AR 4003841; OIO: Informal Benchmarking of LaSalle; 04/28/2017
- AR 4057251; PMC: Historical TIP Issue; 09/29/2017
- AR 4068406; IRM 13 1/2 Scram; 10/30/2017
- AR 4068407; IRM 14 1/2 Scram During S/D; 10/30/2017
- AR 4088847; RM Self Assessment Deficiency (S/D NI Reliability); 12/30/2017
- AR 3977613; SPC 03949773-02: Unit 2 LPRM 32-49B Failure Analysis Results; 02/23/2017

Other

- Analysis No. M-1156D-2/D2-FW-02C; Pipe Support Qualification; Revision 6
- Calculation No. 8122D2-537; Elevation 537' – Drywell Structural Steel; Revision 11
- EC 621951; Historical Operability for Snubber 2-3019E-58 Failing and Locking; Revision 0
- EC 622674; Reportability Investigation for Locked Snubber 2-3091E-58 for Full Design Basis Operability; Revision 0
- Work Order 390652; MSIV Limit Switch Relocation - U2; 11/13/2017
- EC 390651; MSIV Limit Switch Relocation; Revision 3