

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT NO. 79-11

NOVEMBER, 1979

APPROVED:

D. L. Stewart  
MANAGER

7912140 338

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OPERATING DATA REPORT

DOCKET NO. 50-280  
DATE 04 DEC 79  
COMPLETED BY O.J. COSTELLO  
TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 1	
2. REPORTING PERIOD	791101 TO 791130	
3. LICENSED THERMAL POWER (MWT)	2441	
4. NAMEPLATE RATING (GROSS MWE)	847.5	NOTES
5. DESIGN ELECTRICAL RATING (NET MWE)	822	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775	
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A	

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY N/A  
(NET MWE)

10. REASONS FOR RESTRICTIONS, IF ANY N/A

THIS MONTH YR-TO-DATE CUMULATIVE

11. HOURS IN REPORTING PERIOD	720.0	8016.0	60840.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	720.0	2605.7	38305.6
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	3500.9	3500.9
14. HOURS GENERATOR ON-LINE	720.0	2600.1	37459.0
15. UNIT RESERVE SHUTDOWN HOURS	0.0	3504.6	3504.6
16. GROSS THERMAL ENERGY GENERATED (MWH)	1748073.0	6270436.0	86626389.0
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	567000.0	2020220.0	28327803.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	539327.0	1919679.0	26891398.0
19. UNIT SERVICE FACTOR	100.0 %	32.4 %	61.6 %
20. UNIT AVAILABILITY FACTOR	100.0 %	76.2 %	67.3 %
21. UNIT CAPACITY FACTOR (USING MDC NET)	96.7 %	30.9 %	57.0 %
22. UNIT CAPACITY FACTOR (USING DER NET)	91.1 %	29.1 %	53.8 %
23. UNIT FORCED OUTAGE RATE	0.0	67.5 %	24.6 %
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)	REFUELING JUNE 1, 1980, 6 WEEKS		

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP N/A

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION) FORECAST ACHIEVED

INITIAL CRITICALITY  
INITIAL ELECTRICITY  
COMMERCIAL OPERATION

OPERATING DATA REPORT

CORRECTED COPY FOR MONTH OF OCT.

DOCKET NO. 50-280
DATE 27 NOV 79
COMPLETED BY O.J. COSTELLO
TELEPHONE 804-357-3184

OPERATING STATUS

- 1. UNIT NAME
2. REPORTING PERIOD
3. LICENSED THERMAL POWER (MWT)
4. NAMEPLATE RATING (GROSS MWE)
5. DESIGN ELECTRICAL RATING (NET MWE)
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS

SURRY UNIT 1
791001 TO 791031

Table with 2 columns: Value and NOTES. Values include 2441, 847.5, 822, 811, 775, N/A.

- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)
10. REASONS FOR RESTRICTIONS, IF ANY

THIS MONTH YR-TO-DATE CUMULATIVE

Table with 4 columns: Item, THIS MONTH, YR-TO-DATE, CUMULATIVE. Rows include HOURS IN REPORTING PERIOD, NUMBER OF HOURS REACTOR WAS CRITICAL, REACTOR RESERVE SHUTDOWN HOURS, etc.

- 25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP

- 26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION) FORECAST ACHIEVED

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

OPERATING DATA REPORT

DOCKET NO. 50-281
DATE 04 DEC 79
COMPLETED BY O.J. COSTELLO
TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME SURRY UNIT 2
2. REPORTING PERIOD 791101 TO 791130
3. LICENSED THERMAL POWER (MWT) 2441
4. NAMEPLATE RATING (GROSS MWE) 847.5
5. DESIGN ELECTRICAL RATING (NET MWE) 822
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE) 811
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE) 775
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS N/A

Table with 2 columns: Item, Notes. Row 4: 847.5 | NOTES

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE) N/A
10. REASONS FOR RESTRICTIONS, IF ANY N/A

THIS MONTH YR-TO-DATE CUMULATIVE

Table with 4 columns: Item, This Month, Yr-to-Date, Cumulative. Rows 11-24 include metrics like Hours in Reporting Period, Energy Generated, and Service Factor.

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP 5/5/80

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION) FORECAST ACHIEVED

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

# UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-280  
 UNIT NAME SURRY UNIT 1  
 DATE DEC. 1, 1979  
 COMPLETED BY O. J. COSTELLO  
 TELEPHONE (804) 357-3184

REPORT MONTH NOVEMBER, 1979

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting Down Reactor	Licensee Event Report #	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
None during this reporting period.									

1  
**F:** Forced  
**S:** Scheduled

2  
**Reason:**  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

3  
**Method:**  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

4  
**Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)**

5  
**Exhibit I - Same Source**

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.

50-280

UNIT NAME

SURRY 1

DATE

NOV. 2, 1979

COMPLETED BY

O. J. COSTELLI

TELEPHONE

(804) 357-3184

REPORT MONTH OCT. 1979

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
79-8	10-01-79	F	574.6	D	1				Continuation of shutdown due to Show Cause Order issued by the NRC concerning possible error in piping stress analysis performed by Stone & Webster.  The Show Cause Order was modified Aug. 22nd to allow interim operation when required modifications are completed. Shutdown began on 3/19/79.  *Reactor tripped on feed flow/steam flow mismatch coincident with low steam generator level signal while feeding steam generators in manual during power increase following startup.
79-9	10-24-79	F	2	II*	3				

1  
F: Forced  
S: Scheduled

2  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance of Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

3  
Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Other (Explain)

4  
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5  
Exhibit I - Same Source

# UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH NOVEMBER 1979

DOCKET NO.	50-281
UNIT NAME	SURRY UNIT 2
DATE	DEC. 1, 1979
COMPLETED BY	O. J. COSTELLO
TELEPHONE	(804) 357-3184

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
79-9	11-01-79	/S	720	C	1				Continuation of shutdown for refueling and steam generator replacement which began on 2/4/79.

<sup>1</sup>  
F: Forced  
S: Scheduled

<sup>2</sup>  
Reason:  
A-Equipment Failure (Explain)  
B-Maintenance of Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup>  
Methods:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Other (Explain)

<sup>4</sup>  
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
Exhibit I - Same Source

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO.1

MONTH: NOVEMBER, 1979

<u>DATE</u>	<u>TIME</u>	<u>HOURS</u>	<u>LOAD, MW</u>	<u>REDUCTIONS, MW</u>	<u>MWH</u>	<u>REASON</u>
			None during this reporting period.			
MONTHLY TOTAL					0	

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 2

MONTH: NOVEMBER, 1979

<u>DATE</u>	<u>TIME</u>	<u>HOURS</u>	<u>LOAD, MW</u>	<u>REDUCTIONS, MW</u>	<u>MWH</u>	<u>REASON</u>
				None during this reporting period.		
<b>MONTHLY TOTAL</b>					0	

DOCKET NO 280  
UNIT SURRY I  
DATE 12-1-79  
COMPLETED BY O J COSTELLO

AVERAGE DAILY UNIT POWER LEVEL

MONTH: NOVEMBER 79

DAY	AVERAGE DAILY POWER LEVEL (MWE=NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE=NET)
1	751.8	16	753.6
2	750.8	17	750.5
3	755.0	18	752.3
4	757.5	19	753.5
5	757.7	20	752.7
6	753.8	21	751.1
7	753.5	22	751.9
8	753.9	23	751.3
9	756.5	24	753.0
10	682.5	25	752.0
11	699.5	26	751.8
12	752.4	27	751.1
13	755.8	28	750.4
14	758.2	29	749.8
15	755.6	30	752.5

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE=NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 % LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

DOCKET NO 50-281  
UNIT SURRY II  
DATE 12-1-79  
COMPLETED BY O J COSTELLO

AVERAGE DAILY UNIT POWER LEVEL

MONTH: NOVEMBER 79

DAY	AVERAGE DAILY POWER LEVEL (MWE=NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE=NET)
1	0.0	16	0.0
2	0.0	17	0.0
3	0.0	18	0.0
4	0.0	19	0.0
5	0.0	20	0.0
6	0.0	21	0.0
7	0.0	22	0.0
8	0.0	23	0.0
9	0.0	24	0.0
10	0.0	25	0.0
11	0.0	26	0.0
12	0.0	27	0.0
13	0.0	28	0.0
14	0.0	29	0.0
15	0.0	30	0.0

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE=NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 % LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

-9-  
SUMMARY OF OPERATING EXPERIENCE

NOVEMBER, 1979

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT 1

- November 1 - This reporting period begins with the unit at 100% power.
- November 2 - Reduced reactor power to 99% while investigating validity of secondary calorimetric data.
- November 3 - Evaluation of calorimetric data completed satisfactorily. Reactor power raised to 100%.
- November 10 - At 1158 indications of gross condenser tube leakage were received in the control room (i.e., the condensate sodium recorder "pegged" upscale). Immediately, preparations were made to remove the leaking condenser waterbox from service and the steam generator (S/G) blowdown was increased to 65 gpm on all S/G's. Shortly after the waterbox with the highest leakage (as indicated by installed conductivity cells) was removed, it became apparent that condenser tube leakage was not the source of sodium in the main condensate system. An investigation of possible sources commenced and at 1350 reactor power was reduced to 90% in an attempt to reduce sodium carryover in the main steam exiting the S/G's. The unit reached 90% power at 1445. At 1520 a further reduction to 75% power was commenced. At 1610 the sodium contamination source was eliminated. The source was traced to the auxiliary steam drains and an improperly installed jumper hose. The unit reached 75% reactor power at 1700.
- November 11 - At 0423 secondary chemistry had shown significant improvement and the decision was made to begin a power increase to 90% reactor power. The unit reached 90% reactor power at 0535. Secondary chemistry continued to improve and at 0730 a reactor power increase to 100% began. At 0845 the unit reached 100% reactor power.
- November 30 - This reporting period ends with the unit at 100% reactor power.

UNIT 2

- November 1 - This reporting period begins with the unit at cold shutdown, all fuel removed from the reactor and steam generator replacement in progress.
- November 30 - This reporting period ends with the unit at cold shutdown.

AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

NOVEMBER, 1979

None during this reporting period.

FACILITY CHANGES REQUIRING  
NRC APPROVAL

NOVEMBER, 1979

There were none during this reporting period.

FACILITY CHANGES THAT  
DID NOT REQUIRE NRC APPROVAL

NOVEMBER, 1979

There were none during this reporting period.

TESTS AND EXPERIMENTS REQUIRING

NRC APPROVAL

NOVEMBER, 1979

There were none during this reporting period.

TESTS AND EXPERIMENTS THAT  
DID NOT REQUIRE NRC APPROVAL

NOVEMBER, 1979

There were none during this reporting period.

OTHER CHANGES, TESTS AND EXPERIMENTS

NOVEMBER, 1979

There were none during this reporting period.

CHEMISTRY REPORT

NOVEMBER , 19 79

T.S.6.6.A.11

PRIMARY COOLANT ANALYSIS	UNIT NO. 1			UNIT NO. 2		
	MAXIMUM	MINIMUM	AVERAGE	MAXIMUM	MINIMUM	AVERAGE
Gross Radioact., µCi/ml	5.26E-1	3.26E-1	4.19E-1	2.08E-2	1.49E-4	8.72E-3
Suspended Solids, ppm	0.1	0.0	0.1	0.1	0.1	0.1
Gross Tritium, µCi/ml	1.36E-1	7.33E-2	4.33E-2	*	*	*
Iodine-131, µCi/ml	2.59E-2	3.43E-3	8.79E-3	*	*	*
I-131/I-133	0.8301	0.2066	0.4190	*	*	*
Hydrogen, cc/kg	40.2	25.7	30.2	*	*	*
Lithium, ppm	1.03	0.60	0.83	*	*	*
Boron-10, ppm +	78.8	62.7	71.5	367.5	216	244
Oxygen-16, ppm	0.000	0.000	0.000	4.0	1.5	2.4
Chloride, ppm	0.05	0.05	0.05	0.08	0.05	0.05
pH @ 25°C	6.99		6.88	5.23	4.89	5.10

+ Boron-10 = Total Boron x 0.196

NON-RADIOACTIVE CHEMICAL  
RELEASES, POUNDS  
T.S. 4.13.A.8

Phosphate	<u>0.0</u>	Boron	<u>268</u>
Sulfate	<u>834</u>	Chromate	<u>.24</u>
50% NaOH	<u>1.050</u>	Chlorine	<u>0.0</u>

Remarks: \*Unit #2 is in cold shutdown condition - RHR System refilled 11-21-79.

DESCRIPTION OF ALL INSTANCES WHERE  
THERMAL DISCHARGE LIMITS WERE EXCEEDED

NOVEMBER, 1979

Due to impairment of the circulating water system on the following days the thermal discharge limits were exceeded as noted.

November 1, 1979	*Exceeded 17.5° ΔT across station
November 2, 1979	*Exceeded 17.5° ΔT across station
November 10, 1979	*Exceeded 15.0° ΔT across station
November 27, 1979	*Exceeded 15.0° ΔT across station
November 28, 1979	*Exceeded 15.0° ΔT across station
November 30, 1979	*Exceeded 15.0° ΔT across station

\*Indicates dates where station ΔT was <15.0°F across the station for sometime during the day.

The ΔT excursions were allowable under Technical Specifications 4.14.B.2. There were no reported instances of significant adverse environmental impact.

On November 10, 1979, the temperature change at the station discharge exceeded 3°F per hour while removing 1B condenser waterbox from service for suspected tube leakage. This event was reported in accordance with Technical Specifications 4.14.

FUEL HANDLING

NOVEMBER, 1979

There was no fuel received or shipped during this reporting period.





PROCEDURE REVISIONS THAT CHANGED THE  
OPERATING MODE DESCRIBED IN THE FSA

NOVEMBER, 1979

None during this reporting period.

DESCRIPTION OF PERIODIC TESTS WHICH WERE NOT  
COMPLETED WITHIN THE TIME LIMITS  
SPECIFIED IN TECHNICAL SPECIFICATIONS

NOVEMBER, 1979

The following monthly Periodic Tests were not conducted in August. Subsequently these test have been completed satisfactorily.

<u>NUMBER</u>	<u>TITLE</u>	<u>DESCRIPTION</u>
PT-24.4H	Fire Protection Systems (Hose Houses)	To check equipment and hydrants for operability.
PT-24.8	Fire Protection Systems (Fire Lockers)	To assure lockers contain equipment on inventory and operability of gear.

The following Periodic Tests were not completed within the time limits specified in the Technical Specifications.

PT-38.11	Primary Coolant Crud Analysis was not completed on Unit #1 when scheduled (11-9-79). The test performs a radio chemical analysis of the reactor coolant to evaluate certain corrosion products present. The test was conducted on 11-19-79, three days beyond the 25% grace period allowed.
PT-38.29	Service Air and Breathing Air Compressor Sampling on Unit #1 was not conducted when scheduled (5-31-79). The test is to assure the quality of air used in self contained breathing apparatus, respirators, hoods, and suits meets required specifications. This semi-annual test was conducted on 10-25-79 three (3) months beyond the 25% grace period. When conducted, the test indicated the air fulfilled specified requirements.

INSERVICE INSPECTION

NOVEMBER, 1979.

The Unit #2 radiography of the feedwater lines (IE Bulletin 79-13 Revision 2) was completed. One weld (#7) on "A" Steam Generator feed-line was rejected (small area of rejectable porosity). This weld was repaired and accepted. All other welds were accepted. Several of the welds on the feedwater lines were replaced as part of the S/G Replacement Project. These welds were replaced prior to the IE Bulletin 79-13 implementation in Unit #2. NDE information on the original welds was unobtainable. All new welds were radiographed to the S/G Replacement Project construction code in force and not to the requirements of IE Bulletin 79-13.

The monthly visual inspections required by IE Bulletin 79-17 Revision 1 (Pipe cracks in stagnant borated water systems) is in progress in both Units 1 and 2. No rejectable indications have been reported at this time. The inspection is still in progress.

UNIT2-12/03/79  
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTDWNTH
11/20/79	CH	PIPING	3/4 CH-406-150	REMOVE ARC STRIKE	REMOVED ARC STRIKE PERFORMED EXAM	2	911092145	120
11/21/79	FW	PIPING	14-WFPD-109-60	NDT FEEDWATER PIPING	NO REJECTABLE INDICATIONS	2	907240830	2856
11/21/79	FW	PIPING	14-WFPD-117-60	NDT FEEDWATER PIPING	ONE REJECTABLE WELD-REPAIRED	2	907240832	2856
11/21/79	FW	PIPING	14-WFPD-113-60	NDT FEEDWATER PIPING	NO REJECTABLE INDICATIONS	2	907270833	2856
								*****
DEPT TOTAL								8688

23  
REPORTABLE OCCURRENCES PERTAINING TO  
ANY OUTAGE OR POWER REDUCTIONS

NOVEMBER, 1979

There were none during this reporting period.

Maintenance of Safety Related Systems During  
Outage or Reduced Power Periods

UNIT #1

Mechanical Maintenance

UNIT #1

Mechanical Maintenance

None during this reporting period.

Maintenance of Safety Related Systems During  
Outage or Reduced Power Periods

UNIT #2

Mechanical Maintenance

UNIT2=12/03/79  
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSEVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTLWNTM
11/01/79	IA	COMPRESS	2-IA-C=1	OIL LEAK	REPAIRED OIL LEAK	2	910221117	206
11/02/79	EE	COMPRESS	EDG 2	LIFT INNERSTAGE RELIEF VALVE	REPLACED INTAKE AND DISCHARGE VALVES	2	905110415	29
11/02/79	PR	SNUBBER	2-RC-HSS-137	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161530	4054
11/02/79	PR	SNUBBER	2-RC-HSS-136	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161531	4054
11/02/79	PR	SNUBBER	2-RC-HSS-135	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161532	4056
11/02/79	PR	SNUBBER	2-RC-HSS-134	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161533	3213
11/02/79	PR	SNUBBER	2-RC-HSS-133	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161534	3213
11/02/79	PR	SNUBBER	2-RC-HSS-132	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161535	3213
11/02/79	PR	SNUBBER	2-RC-HSS-131	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161536	3213
11/02/79	PR	SNUBBER	2-RC-HSS-130	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161537	3213
11/02/79	PR	SNUBBER	2-RC-HSS-129	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161538	3213
11/02/79	PR	SNUBBER	2-RC-HSS-128	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161539	3213
11/02/79	PR	SNUBBER	2-RC-HSS-127	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161540	3213
11/02/79	PR	SNUBBER	2-RC-HSS-126	REBUILD SNUBBER WITH EP MATERIAL	REBUILT SNUBBER	2	905161541	3216
11/05/79	MS	VALVE		GAGE SAFETY VALVES	COMPLETED	2	911041355	2
11/06/79	FW	PMP MTR	2-FW-P=3B	NO LEAK OFF THRUST BEARING+PACKING	ADJUSTED	2	911042204	24
11/07/79	RH	VALVE	2-RH=2	DC 79-05 REPLACE STUDS WITH 416SS	VALVE BONNET WAS NOT REMOVED	2	910241000	160
11/07/79	RH	VALVE	2-RH=8	DC 79-05 REPLACE STUDS WITH 416SS	VALVE BONNET WAS NOT REMOVED	2	910241001	160
11/07/79	RH	VALVE	2-RH=24	DC 79-05 REPLACE STUDS WITH 416SS	VALVE BONNET WAS NOT REMOVED	2	910241005	163
11/07/79	RH	VALVE	MOV-RH=2720A	DC 79-05 REPLACE STUDS WITH 416SS	VALVE BONNET WAS NOT REMOVED	2	910241007	146
11/07/79	RH	VALVE	MOV-RH=2720B	DC 79-05 REPLACE STUDS WITH 416SS	VALVE BONNET WAS NOT REMOVED	2	910241008	146
11/07/79	SI	VALVE	2-SI-147	REPLACE STUDS	REPLACE STUDS	2	911061503	11
11/13/79	CC	PUMP	2-CC-F-1C	EXCESSIVE AXIAL MOVEMENT	REBUILT PUMP	2	910082345	792
11/13/79	SI	VALVE	2-SI-130	REPLACE STUDS	REMOVED OLD STUDS+REPLACED UPGRADED	2	911061502	120
11/14/79	CH	VALVE	2-CH=263	VALVE HANDWHEEL SHEARED AND REMOVED	REPLACED VALVE HAND WHEEL	2	908182105	5
11/15/79	DG	VALVE	2-DG=10	REPLACE DIAPHRAGM	REPAIRED VLV	2	902161003	2747
11/15/79	DG	VALVE	2-DG=48	REPLACE DIAPHRAGM	REPLACED DIAPHRAGM	2	902161004	2747
11/15/79	DG	VALVE	2-DG=50	REPLACE DIAPHRAGM	REPLACED DIAPHRAGM	2	902161005	2747
11/15/79	DG	VALVE	2-DG=51	REPLACE DIAPHRAGM	REPLACED DIAPHRAGM	2	902161006	2747
11/15/79	DG	VALVE	2-DG=64	REPLACE DIAPHRAGM	REPAIRED VLV	2	902161017	2747
11/16/79	DG	VALVE	2-DG=59	REPLACE DIAPHRAGM	REPLACE DIAPH VALVE PROGRAM	2	902161013	2770
11/16/79	DG	VALVE	2-DG=60	REPLACE DIAPHRAGM	REPLACE DIAPHRAGM VALVE PROGRAM	2	902161014	2770
11/16/79	DG	VALVE	2-DG=72	REPLACE DIAPHRAGM	REPLACE DIAPHRAGM VALVE PROGRAM	2	902161020	2770
11/16/79	DG	VALVE	2-DG=75	REPLACE DIAPHRAGM	REPLACE DIAPH VALVE PROGRAM	2	902161022	2770
11/16/79	DG	VALVE	2-DG=76	REPLACE DIAPHRAGM	REPLACE DIAPH VALVE PROGRAM	2	902161023	2770
11/16/79	DG	VALVE	2-DG=77	REPLACE DIAPHRAGM	REPLACE DIAPH VALVE PROGRAM	2	902161024	2770
11/16/79	DG	VALVE	2-DG=80	REPLACE DIAPHRAGM	REPLACE DIAPHRAGM VALVE PROGRAM	2	902161025	2770
11/16/79	DG	VALVE	2-DG=86	REPLACE DIAPHRAGM	REPLACE DIAPHRAGM VALVE PROGRAM	2	902161029	2770
11/16/79	RH	VALVE	2-RH=23	INSPECT VALVE	INSPECTED VALVE	2	908171615	48
11/19/79	DG	VALVE	2-DG=67	REPLACE DIAPHRAGM	REPLACED DIAPHRAGM	2	902161019	2747
11/20/79	CH	VALVE	2-CH=303	LEAKS THRU ON PT16.4	REPAIRED VALVE	2	907101130	24
11/20/79	CH	VALVE	HCV=2310A	REPAIR POSITION INDICATOR	ADJUSTED	2	911160805	5
11/20/79	CH	VALVE	FCV=2160	REPAIR POSITION INDICATOR	REPAIRED	2	911160806	5
11/20/79	CH	VALVE	HCV=2311	REPAIR POSITION ON INDICATOR	ADJUSTED	2	911160808	5
11/21/79	SI	VALVE	2-SI=109	REPLACE STUDS	REPLACED STUDS	2	911061501	408
11/21/79	CC	INSTR	FI=CC=218B	INSTALL FLOW INDICATOR	REPAIRED	2	911200828	1
11/22/79	CC	VALVE	2-CC=953	VALVE IS MISSING HANDWHEEL	REPLACED VALVE HANDWHEEL	2	911200821	37

## UNIT2-12/03/79

## (MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERVD	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTDWTM
11/26/79	SI	VALVE	2-SI-85	REPLACE STUDS	REPLACED STUDS	2	911131015	168
11/26/79	SI	VALVE	2-SI-243	REPLACE STUDS	REPLACED STUDS	2	911131016	168
11/26/79	SI	VALVE	2-SI-240	REPLACE STUDS	REPLACED STUDS	2	911131018	168
11/26/79	SI	VALVE	2-SI-242	REPLACE STUDS	REPLACED WITH UPGRADED STUDS	2	911131020	144
11/26/79	SI	VALVE	2-SI-94	REPLACE STUDS	REPLACED STUDS	2	911131025	168
11/26/79	SI	VALVE	2-SI-238	REPLACE STUDS	REPLACED WITH UPGRADED BOLTS+NUTS	2	911131026	144
11/27/79	RC	VALVE	2-RC-141	REPLACE STUDS	REPLACED STUDS	2	911051620	501
11/27/79	RC	VALVE	2-RC-158	REPLACE STUDS	REPLACED STUDS	2	911051622	507
11/27/79	CH	PUMP	2-CH-P-2C	REPAIR OIL LEAK ON SIGHT GLASS	REPLACED OIL FEEDER	2	911240535	5
11/28/79	SI	VALVE	2-SI-82	REPLACE STUDS	REPLACED DEGRADED STUDS+NUTS	2	911131019	192
11/28/79	SI	VALVE	2-SI-88	REPLACE STUDS	REPLACED DEGRADED STUDS+NUTS	2	911131023	192
11/29/79	SI	VALVE	2-SI-91	REPLACE STUDS	REPLACED STUDS	2	911131021	31
11/29/79	SI	VALVE	2-SI-239	REPLACE STUDS	REPLACE STUDS	2	911131022	34
11/29/79	BR	VALVE	MOV-BR-101B	REPACK	REPACKED VALVE	2	911271443	25
12/01/79	BR	PUMP	BR-P-1A	INLET + DISCHARGE FLANGES LEAK	REPAIRED	2	911271445	26

DEPT TOTAL

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Maintenance of Safety Related Systems During  
Outage or Reduced Power Periods

UNIT #1

Electrical Maintenance

DEPT=ELEC

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UNIT1-12/03/79

(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTDWTM
11/19/79	CH	INSTR		DEFECTIVE RELAY	REPLACE COIL	1	911141446	120 ***** 120
DEPT TOTAL								

Maintenance of Safety Related Systems During  
Outage or Reduced Power Periods

UNIT #2

Electrical Maintenance

## UNIT2-12/03/79

## (MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERVD	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTDWNM
11/02/79	EPDC	CHARGER	2A2	CHECK FOR INSULATION BREAKDOWN	CHECKED WIRING TESTED SAT	2	905231430	40
11/03/79	EPDC	CHARGE	2B-1	INSTALL NEW CAPACITORS	INSTALLED NEW CAPACITORS LOAD TESTED	2	910290725	100
11/05/79	SW	MOV	MOV-SW-206A	MOV PMS	COMPLETED AS PER EMP-P-MOV-45	2	901251535	6600
11/05/79	SW	MOV	MOV-SW-206B	MOV PMS	COMPLETED AS PER EMP-P-MOV-45	2	901251536	1600
11/05/79	CH	PUMP	2-CH-P-1B	AUX OIL THERMALS OUT	LOAD CHECKED SAT	2	909290210	792
11/06/79	FW	MOV	MOV-FW-251D	MOV PMS	COMPLETED AS PER EMP-P-MOV-45	2	901251409	6504
11/14/79	CH	HT	PNL0-6	LOW ALARM	ALARM HAD CLEARED	2	911111810	26
11/14/79	CH	HT	PNL0-6	LOW ALARM	ALARM HAD CLEARED	2	911111811	26
11/19/79	CH	MOV	MOV-CH-2370	DISCONNECT FOR MECHANIC	COMPLETED AS PR EMP-C-MOV-11	2	903082151	5400
11/19/79	EPDC	BATT		PERFORM PT 23.4A	COMPLETED EMP-P-EPDC-44 PT 23.3	2	911091400	33
11/21/79	CH	MOV	MOV-2370	MOV PMS	COMPLETED AS PER EMP-P-MOV-45	2	901251456	6725
11/26/79	CH	MOV		DISCONNECT FOR MECH MOV-2206B	DISCONNECTED+RECONNECTED TESTED SAT	2	900190954	173
11/28/79	RC	VALVE	HCV-RC-2556C	CONNECT SOV	TESTED VALVE VALVE TESTED OK	2	911201331	184
11/28/79	RC	VALVE	HCV-RC-2557C	CONNECT SOV	TESTED VALVE VLV TEST OK	2	911201334	184

DEPT TOTAL

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Maintenance of Safety Related Systems During  
Outage or Reduced Power Periods

UNIT #1

Instrument Maintenance

UNIT #1

Instrument Maintenance

None during this reporting period.

Maintenance of Safety Related Systems During  
Outage or Reduced Power Periods

UNIT #2

Instrument Maintenance

DEPT=INST

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UNIT2-12/03/79  
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSEVDT	SYS	COMP	MARKNO	SUMMARY	WKPERF	U	MR	TOTDWN TM
11/07/79	RC	INSR		CALIBRATE PRESS TRANSMITTER	CALIBRATED TRANSMITTERS	2	911060000	30
11/21/79	GW	INSTR	H2-GW-203	COMPLETE CALIBRATION	CAL+OP CHECK	2	911141051	144
11/28/79	RH	INSTR	FCV-RH-2605	REPAIR VALVE	REPLACED RELAY ADJUSTMENT E/P	2	911241200	94
DEPT TOTAL								268

NOVEMBER, 1979

There was no single release of radioactivity or radiation exposure specifically associated with an outage that accounted for more than 10% of the allowable annual values in 10CFR20.

PROCEDURE DEVIATIONS REVIEWED BY STATION NUCLEAR  
SAFETY AND OPERATING COMMITTEE AFTER TIME LIMITS  
SPECIFIED IN TECHNICAL SPECIFICATIONS

NOVEMBER, 1979

<u>NUMBER</u>	<u>UNIT</u>	<u>TITLE</u>	<u>DEVIATION</u>
OP-7.1 Valve Checkoff Sheet	1	Safety Injection System	1-SI-21 Left Closed

This procedure was deviated October 17, 1979 and reviewed by the Station Nuclear Safety and Operating Committee November 7, 1979.

OP-32A Valve Checkoff Sheet	1	Steam Generator Blowdown System	1-BD-37 Left Closed.
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This procedure was deviated October 15, 1979, and reviewed by the Station Nuclear Safety and Operating Committee November 7, 1979.