



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report Nos. 50-280/79-39 and 50-281/79-58

Licensee: Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Facility Name: Surry 1 and 2

License Nos. DPR-32 and DPR-37

Inspection at Surry Site near Williamsburg, Virginia

Inspector: <u>G. R. Jenkins</u> for R. W. Zavadoski	<u>7/19/79</u> Date Signed
Approved by: <u>G. R. Jenkins</u> G. R. Jenkins, Acting Section Chief, FF&MS Branch	<u>7/19/79</u> Date Signed

SUMMARY

Inspection on June 26, 1979 - June 29, 1979

Areas Inspected

This routine unannounced inspection involved 28 inspector-hours onsite in the areas of health physics practices in the containments and outside areas, control of contaminated material, personnel exposures for the SGRP to date, issuance and use of RWP's, and solid radwaste shipments.

Results

Of the six areas inspected, no apparent items of noncompliance or deviations were identified in five areas; two apparent items of noncompliance were found in one area failure to have shipping documents, (50-280/79-39-01 and 50-281/79-58-01) paragraph 12, failure to follow a certificate of compliance, (50-280/79-39-02 and 50-281/79-58-02) paragraph 12).

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DETAILS

1. Persons Contacted

Licensee Employees

- R. M. Smith, Health Physics Supervisor
- *G. E. Kane, Operating Supervisor
- *J. W. Patrick, Mechanical Supervisor
- *P. P. Nottingham, III, Ass't. Health Physics Supervisor (SGRP)
- *F. L. Rentz, Resident Q. C. Engineer
- *J. Goodson, Resident Q. C. Engineer (SGRP)
- *E. P. Dewandel Staff Assistant
- *C. W. Rhodes, SGRP

Other licensee employees contacted included 10 construction craftsman, 7 technicians, 5 operators, and 4 mechanics.

*Denotes those present at exit interview

2. Exit interview

The inspection scope and findings were summarized on June 29, 1979, with those persons indicated in Paragraph 1 above. Items discussed included two items of non compliance on solid radwaste shipments discussed in paragraph 12. Licensee management acknowledged the items of noncompliance.

3. Licensee Action on Previous Inspection Findings

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Unit No. 2 Containment Tours

Throughout the course of the inspection, the inspector, accompanied by licensee's representatives and alone, made numerous entries into the Unit No. 2 containment to observe work in progress and work practices. The inspector observed that considerable effort had gone into removing radioactive wastes from the containment. He also noted workers using waiting areas and workers being directed by foreman/health physics technicians to use waiting areas. The inspector observed health physics practices within the containment and found health physics technicians thoroughly covering jobs and requesting additional shielding where necessary.

Workers were trying to minimize their exposures by finding the lowest radiation fields in which to work. The inspector noticed that the forty-seven foot elevation in the containment had no significant radiation fields

greater than 5 millirem/hour and that there was unnecessary posting of waiting areas. The inspector brought this to the attention of licensee management and they promptly removed signs designating waiting areas on the forty-seven foot elevation in order to avoid confusion on the part of the workers who were using waiting areas on the elevations other than the forty-seven foot level. The inspector noted no items of noncompliance and had no further questions.

6. Unit No. 1 Containment Tour

The inspector, accompanied by a licensee's representative, toured the Unit No. 1 containment. Unit No. 1 is presently shut down for repairs and inspection. The inspector noted that housekeeping in the containment had improved (IE Rpt. Nos. 50-280/79-09 and 50-281/79-10, Paragraph 6.B) but still was unsatisfactory for power operation. This item was brought to the attention of plant management who stated that the containment would be cleaned prior to power ascension. The inspector had no further questions at this point.

7. Outside Area Tour

The inspector, accompanied by licensee's representative and alone, toured the RCA outside area. The inspector noted that the majority of solid wastes had been shipped off site for burial. The inspector also observed the storage of radioactive material outside and found the packaging and labeling to be satisfactory. The inspector had no further questions in this area.

8. Exposures to Date

The inspector reviewed a draft copy of Progress Report No. 2 for the Steam Generator Repair Project for Surry Unit No. 2. The draft report shows that as of May 31, 1979, 1007 Man-Rem had been expended on the project compared to an estimated exposure of 1094 Man-Rem. Licensees' representatives estimate that as of June 29, 1979, the project was 45% complete and 65% of the exposures have been expended.

9. Issuance and Use of Radiation Work Permits (RWP's)

The inspector observed the issuance of RWP's for special jobs, the use of standard RWP's, including the quizzing and checking before entering the radiation control area of each individual by a health physics technician as to which RWP an individual was working under.

A random survey taken by the inspector of numerous individuals inside the RCA revealed that the individuals were cognizant of their RWP number and requirements. The inspector found no items of noncompliance and had no further questions in this area.

10. Project Area Tour

The inspector, accompanied by a licensee representative, and equipped with Region II instrumentation, toured each warehouse storage area, construction shed, and selected office space in the steam generator repair project area. The purpose of the tour was to verify licensee's control over contaminated material and wastes. The inspector did not identify any contaminated material or wastes, found no items of noncompliance and had no further questions.

11. Control of Contaminated Tools

The inspector observed the issuance and return of contaminated tools from the tool room inside the Unit No. 2 containment. Discussions with tool room workers revealed that the majority of tools were accounted for. By their estimate, a normal rate of attrition prevailed. Discussions with the compactor operators inside the Unit 2 containment revealed that an insignificant number of tools were found in the trash. The inspector also observed the use and storage of tools at various job locations within the containment and found no items of noncompliance. The inspector had no further questions.

12. Solid Radwaste Shipments

The inspector reviewed records for solid radwastes for the period April 1, 1979 thru June 28, 1979. The inspector noted that several shipments had been made by the plant in a Chem-Nuclear cask designated by Chem-Nuclear as Model No. 18-450. Licensee's representatives stated that the cask was used to ship relatively hot radioactive drums (i.e., drums which met low specific activity requirements and contained less than Type A quantities for Group III material, 3 curies by 10 CFR 71.4.(q) but had surface radiation readings of several rem/hour). Although technically the drums could have been shipped in a rag-top trailer truck, they were shipped in a cask to take advantage of the cask's shielding properties. On April 26, 1979, a Chem-Nuclear Cask Model No. 18-450 was used to ship 1.765 curies of radioactive waste weighing 9800 pounds. Chem-Nuclear Cask Model No. 18-450 is licensed by Certificate of Compliance No. 9122 (issued November 1978, and expires October 1983), issued under the provisions of 10 CFR 71.12.(b). Licensee's representatives stated that they did not have a copy of the Certificate of Compliance nor all the documents referred to in the certificate.

The inspector informed licensee's representatives that failure to have a certificate of compliance and referenced documents was contrary to the provisions of 10 CFR 71.12.b.1.i and an item of noncompliance (50-280/79-39-01 and 50-281/79-58-01). Further, provision 5.b.2 of Certificate of Compliance No. 9122, limits the maximum quantity of material for the cask to 8000 pounds. The inspector informed licensee's representatives that loading the Chem-Nuclear Cask, Model No. 18-450, on April 26, 1979, with 9800 pounds was contrary to the provisions of 10 CFR 71.12.b.1.ii and was an item of noncompliance (50-280/79-39-02 and 50-281/79-58-02).