UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report No. 50-281/79-39

Licensee: Virginia Electric and Power Company
Richmond, Virginia 23261

Facility Name: Surry Unit 2

Docket No. 50-281

License No. DPR-37

Inspection at Surry Site near Williamsburg, Virginia

Inspector: M. J. Gouge — Date Signed: 7/6/79

Approved by: F. S. Cantrell, Acting Section Chief, RCES Branch — Date Signed: 7/6/79

SUMMARY

Inspection on June 19-22, 1979

Areas Inspected

This routine, unannounced inspection involved 24 inspector-hours on-site in the areas of steam generator replacement program QA audits, control of nonconformance reports, and observation of welding activities.

Results

Of the three areas inspected, no apparent items of noncompliance or deviations were identified.

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DETAILS

1. Persons Contacted

Licensee Employees

*W. L. Stewart, Station Manager
*C. W. Rhodes, Resident Engineer Construction (SGRP)
*J. B. Goodson, Resident QC Engineer (SGRP)
D. Meeks, QA Engineer (SGRP)
C. W. Embler, Staff Engineer, Welding (SGRP)
R. Bradshaw, QA Engineer (SGRP)
Y. P. Mangum, Jr., Resident Engineer Project Control (SGRP)

Other licensee employees contacted included various construction craftsmen, technicians, and office personnel.

Other Organizations

Daniel Construction Company (DCC)
J. Latimer, Project Welding Engineer

NRC Resident Inspector
*D. J. Burke

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on June 22, 1979 with those persons indicated in Paragraph 1 above.

3. Licensee Action on Previous Inspection Findings

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Independent Inspection Effort

a. Steam Generator Replacement Program Quality Assurance (QA) Audits

Audit procedures for Surry Power Station are specified in section 18 of the Nuclear Power Station Quality Assurance Manual (NPSQAM). A QA local instruction, "Monitoring Audit Activities," has been issued to augment the NPSQAM and to meet the special requirements of the Steam Generator Replacement Project (SGRP). This local instruction was

approved May 14, 1979 and was used for the performance of audits during the period May-June 1979. The Region II inspector reviewed audit checklists, audit findings and project responses for monitoring audits conducted during the period May-June 1979. The monthly written summary of audits conducted, findings, and audits planned was reviewed. The inspector attended the monthly critique of audit findings and QA trends that the resident QC engineer held with SGRP management. The SGRP audit schedule for 1979 was reviewed for completeness. The SGRP audit program was being conducted in accordance with the QA local instruction, "Monitoring Audit Activities."

No items of noncompliance or deviations were identified.

b. Control of Nonconformance Reports (NCR's)

Control of NCR's is described in the Surry NPSQAM, Section 15s, Revision 1, Nonconformances (SGRP). Open NCR's 79-31 through 79-189 were reviewed for conformance to the above procedure. Closed NCR's through 79-90 were reviewed for acceptable disposition. All NCR's reviewed had been documented and controlled in accordance with the above procedure. The Quality Assurance unit is reviewing all open NCR's in an effort to resolve NCR's in a timely manner. The QA unit also reviews all NCR's for acceptable disposition and to determine if any significant trends are developing in the SGRP.

No items of noncompliance or deviations were identified.

6. Steam Generator Replacement Observation of Welding and Associated Activities

The applicable code as required by VEPCO'S licensee submittal (Steam Generator Project Surry Power Station Unit Nos. 1 and 2, Revision 9), is ASME Section XI, 1974 Edition with Addena through the Winter of 1976. Based on paragraph IWA-7000 of section XI, VEPCO is using the original construction code (USAS B31.1.0-1967) for pipe welding. Welder qualifications and procedure qualifications are being performed in accordance with the latest edition and addenda of ASME section IX in effect at the time of qualification. All welding materials were purchased and certified in accordance with Class I requirements of the ASME code.

The inspector observed in process welding of reactor coolant piping EP-4B joining loop B cold leg to steam generator B nozzle. Areas inspected included identification and documentation of QC hold points, proper preheat and interpass temperature, welder qualification to the procedure 8-8-B-22 (automatic TIG), welding technique, and the certified material test report (CMTR) for the welding filler materials. Welding was conducted in accordance with procedure 8-8-B-22 by qualified welders. The controlled weld joint record identified QC hold points and these hold points had been properly documented. The CMTR for welding filler material heat number Y4070L308 met the chemical requirements specified in SFA 5.9 Section II, Part C of the 1974 ASME code through Winter 1976 addenda for ER-308 material.

The Region II inspector observed the second repair to reactor coolant piping section EP-5A (weld #2). Areas inspected included proper documentation of the basic weld buildup and subsequent repairs, welder qualification to applicable procedures, welding technique, control of interpass temperature, and the CMTR for the welding filler materials. The basic weld buildup on weld #2 of EP-5A, repair 1 and repair 2 had been documented properly. The welder performing repair 2 was qualified to procedure 8-8-B-9, Rev. 0 for the thickness he was allowed to weld. Interpass temperature was properly controlled. The CMTR for welding filler material heat number 762904 (ER-316L) was reviewed. The CMTR met the chemical requirements for ER-316L material specified in SFA 5.9 referenced above.

No items of noncompliance or deviations were identified.