



May 2, 2018

NRC 2018-0016
10 CFR 50.71(e)
10 CFR 54.37(b)

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

Periodic Update of the Final Safety Analysis Report

In accordance with the requirements of 10 CFR 50.71(e) and 10 CFR 54.37(b), this letter submits a periodic update of the Point Beach Nuclear Plant Final Safety Analysis Report (FSAR).

Enclosure 1 to this letter includes 1 DVD-ROM containing the non-public version of the Point Beach FSAR, including the description of changes. The FSAR is being submitted in its entirety, constituting a total replacement copy. Enclosure 1 contains security related information as defined by 10 CFR 2.390(d) and should be withheld from public disclosure. The enclosure reflects changes since the last periodic update of September 1, 2016.

Enclosure 2 to this letter includes 1 DVD-ROM containing the public version of the Point Beach FSAR.

Enclosure 3 to this letter includes a report describing how the effects of aging of newly-identified structures, systems or components (SSCs) will be managed, as required by 10 CFR 54.37(b).

This letter contains no new Regulatory Commitments and no revisions to existing Regulatory Commitments.

A053
NRR

~~Security Related Information - Withhold Under 10 CFR 2.390.~~
Enclosure 1 Contains ~~Security Related Information~~,
Upon Separation of Enclosure 1 this letter is Non-Security Related.

Document Control Desk
Page 2

I declare under penalty of perjury that this submittal accurately presents changes made since the previous submittal that reflect information and analyses submitted to the NRC or prepared pursuant to NRC requirements, and changes made under the provisions of 10 CFR 50.59.

Executed on May 2, 2018.

Sincerely,

NextEra Energy Point Beach, LLC



Robert Craven
Plant General Manager

Enclosures (3)

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW (less enclosures)

~~Security Related Information - Withhold from Public Disclosure Under 10 CFR 2.390~~

ENCLOSURE 1

NEXTERA ENERGY POINT BEACH, LLC
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

2017 UPDATE OF THE FINAL SAFETY ANALYSIS REPORT
NOT FOR PUBLIC DISCLOSURE

~~Security Related Information - Withhold Under 10 CFR 2.390.~~
Enclosure 1 Contains ~~Security Related Information,~~
Upon Separation of Enclosure 1 this letter is Non-Security Related.

ENCLOSURE 2

**NEXTERA ENERGY POINT BEACH, LLC
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2**

**2017 UPDATE OF THE FINAL SAFETY ANALYSIS REPORT
REDACTED - PUBLIC VERSION**

ENCLOSURE 3

NEXTERA ENERGY POINT BEACH, LLC POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

REPORT CONSISTENT WITH 10 CFR 54.37(b) ON HOW EFFECTS OF AGING OF NEWLY-IDENTIFIED STRUCTURES, SYSTEMS, OR COMPONENTS ARE MANAGED

This update follows the guidance regarding the appropriate level of detail for reports under 10 CFR 54.37(b) that is presented in Frequently Asked Questions (FAQs) About License Renewal" Inspection Procedure (IP) 71003, "Post-Approval Site Inspection for License Renewal." This report provides summary information as required by 10 CFR 54.37(b) for the period between July 15, 2016 through March 30, 2018.

Regulatory Requirements and Guidance

10 CFR 54.37(b)

After the renewed license is issued, the FSAR update required by 10 CFR 50.71(e) must include any systems, structures, and components newly identified that would have been subject to an aging management review or evaluation of time-limited aging analysis in accordance with §54.21. This FSAR update must describe how the effects of aging will be managed such that the intended function(s) in §54.4(b) will be effectively maintained during the period of extended operation.

RIS 2007-16, Revision 1

Newly Identified Systems, Structures, and Components (SSCs)

The intent of 10 CFR 54.37(b) is to capture those SSCs that, if they had been identified at the time of the license renewal application, would have been subject to an aging management review or evaluation of TLAAs. In the context of 10 CFR 54.37(b), newly identified SSCs that should be included in the next FSAR update required by 10 CFR 50.71(e) are those SSCs that meet one of the two following conditions:

(1) *There is a change to the current licensing basis (CLB) that meets the following criteria:*

- The change impacts SSCs that were not in scope for license renewal when the NRC approved the license renewal application.*
- The SSCs would have been in the scope of license renewal based on the CLB change if 10 CFR 54.4(a) were applied to the SSCs.*

(2) *SSCs were installed in the plant at the time of the license renewal review that, in accordance with the CLB at the time, should have been included in the scope of license renewal per 10 CFR 54.4(a) but were not identified as in scope until after issuance of the renewed license.*

SSCs that are plant additions or modifications installed after the renewed license is issued are not subject to the provisions of 10 CFR 54.37(b).

Identification of SSCs under 10 CFR 54.37(b)

The language of 10 CFR 54.37(b) does not limit how or who finds newly identified SSCs. A licensee may identify SSCs that should be within the scope of its license renewal program at any time. The NRC staff may also discover newly identified SSCs. One way to identify these SSCs is through the LR-ISG process.

Newly Identified SSC

In 2018, using the guidance of RIS 2007-16, Rev. 1, PBNP staff reviewed changes to the plant that had taken place since the last 54.37(b) review of the Current Licensing Basis (July 2016). This review identified 143 components that would be considered "newly identified" and subject to 10 CFR 54.37(b) reporting requirements.

Of these 143 components, 136 are associated with existing plant components that are now being credited for NFPA-805. The remaining 7 are components that are now being credited for flood mitigation. 117 of the 143 are Active components, not requiring aging management. The remaining 26 are passive, long lived components (doors, valves, pumps) are already represented within the aging management tables submitted with the PBNP LRA.

These 26 passive, long-lived components have been evaluated and assigned to existing Aging Management Programs as required. Appropriate aging management strategies have been invoked to adequately detect and manage the applicable aging effects throughout the period of extended operation.

A list of the 143 newly identified components is not provided in this document, consistent with the level of detail provided in the LRA.

May 2, 2018

**Document Control Desk
U. S. Nuclear Regulatory Commission
One White Flint North
Mail Stop 08H 4A
11555 Rockville Pike
Rockville, MD 20852-2738**

Manual: FSAR

Manual No.: 80

Issue Date: May 2, 2018

1. UFSAR 2017 Update.

**AFTER UPDATING YOUR MANUAL, PLEASE SIGN AND RETURN THIS RECEIPT
ACKNOWLEDGMENT TO: POINT BEACH NUCLEAR PLANT, DISTRIBUTION
CENTER, 6590 NUCLEAR ROAD, TWO RIVERS, WI 54241.**

If you have any questions, please call Document Control at ext. 7638.

*****Special Distribution Notes*****

**1-Public CD and
1-Non-Public CD**

Distribution Center - PBNP

Date _____