

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO: Dennis L. Ziemann,

FROM: FPL  
Miami, FL. 33101  
Robert E. Uhrig

DATE OF DOCUMENT

03-21-77

DATE RECEIVED

03-25-77

LETTER  
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*1 signed*

DESCRIPTION

Ltr. Ref. our 01-17-77 ltr and their 10-17-77 ltr...Furnishing info regarding the ability of St. Lucie Unit #1 and others to accommodate a refueling accident within containment...

( 2 pages )

**ACKNOWLEDGED**

PLANT NAME: ST LUCIE UNIT # 1  
jcm

**DO NOT REMOVE**

ENCLOSURE

SAFETY

FOR ACTION/INFORMATION

ENVIRO

ASSIGNED AD:		ASSIGNED AD:
BRANCH CHIEF:	<i>Ziemann (5)</i>	BRANCH CHIEF:
PROJECT MANAGER:	<i>Silver</i>	PROJECT MANAGER:
LIC. ASST. :	<i>Diggs</i>	LIC. ASST. :

INTERNAL DISTRIBUTION

<input checked="" type="checkbox"/> REG FILE	SYSTEMS SAFETY	PLANT SYSTEMS	SITE SAFETY &
<input checked="" type="checkbox"/> NRC PDR	HEINEMAN	TEDESCO	ENVIRO ANALYSIS
<input checked="" type="checkbox"/> I & E (2)	SCHROEDER	BENAROYA	DENTON & MULLER
<input checked="" type="checkbox"/> OELD		LAINAS	
<input checked="" type="checkbox"/> GOSSICK & STAFF	ENGINEERING	IPPOLITO	ENVIRO TECH.
<input checked="" type="checkbox"/> MIPC	MACARRY	KIRKWOOD	ERNST
<input checked="" type="checkbox"/> CASE	BOSNAK		BALLARD
<input checked="" type="checkbox"/> HANAUER	SIHWEIL	OPERATING REACTORS	YOUNGBLOOD
<input checked="" type="checkbox"/> HARLESS	PAWLICKI	STELLO	
			SITE TECH.
<input checked="" type="checkbox"/> PROJECT MANAGEMENT	REACTOR SAFETY	OPERATING TECH.	GAMMILL
<input checked="" type="checkbox"/> BOYD	ROSS	EISENHUT	STAPP
<input checked="" type="checkbox"/> P. COLLINS	NOVAK	SHAO	HULMAN
<input checked="" type="checkbox"/> HOUSTON	ROSZTOCZY	BAER	
<input checked="" type="checkbox"/> PETERSON	CHECK	BUTLER	SITE ANALYSIS
<input checked="" type="checkbox"/> MELTZ		GRIMES	VOLLMER
<input checked="" type="checkbox"/> HELTEMES	AT & I		BUNCH
<input checked="" type="checkbox"/> SKOVHOLT	SALTZMAN		J. COLLINS
	RUTBERG		KREGER

EXTERNAL DISTRIBUTION

CONTROL NUMBER

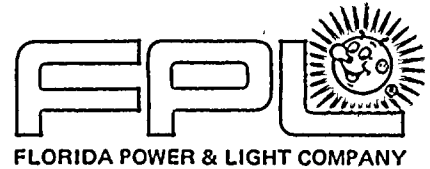
<input checked="" type="checkbox"/> LPDR: <i>Pt Pierce, Fla</i>	NAT. LAB:	BROOKHAVEN NAT. LAB.
<input checked="" type="checkbox"/> TIC:	REG V.IE	ULRIKSON (ORNL)
<input checked="" type="checkbox"/> NSIC:	LA PDR	
<input checked="" type="checkbox"/> ASLB:	CONSULTANTS:	
<input checked="" type="checkbox"/> ACRS 16 CYS HOLDING/SENT	<i>AS CATED</i>	

*770870204 MAY 67*



1-2

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REGULATORY DOCKET FILE COPY

March 21, 1977  
L-77-85

Office of Nuclear Reactor Regulation  
Attn: Dennis L. Ziemann, Chief  
Operating Reactors Branch #2  
Division of Operating Reactors  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Ziemann,

Re: St. Lucie Unit 1  
Docket No. 50-335  
Postulated Refueling Accident



Your letter of 17 January 1977, relayed to us a concern expressed by Mr. Robert D. Pollard regarding the ability of St. Lucie Unit No. 1 and others to accommodate a refueling accident within containment. This issue was previously reviewed by the Staff during the development of the St Lucie Unit No. 1 Technical Specifications. (See Florida Power & Light Company letter L-75-506 dated 17 October 1975, at item 42 of the attachment.) The upper limit dose was evaluated and provided at that time. The maximum thyroid dose is 40 rem and the corresponding whole body dose is 0.17 rem, which are small fractions of the 10 CFR 100 guideline limits of 300 rem thyroid and 25 rem whole body. The principal assumptions used in this evaluation were in accordance with Regulatory Guide 1.25 and are as follows:

- i) A rupture of an entire fuel assembly and release of the gap inventory in accordance with RG 1.25 was assumed.
- ii) The effective DF of shielding water was 100 for iodine and 1 for Kr-85.
- iii) No credit for dilution or mixing within the containment or reactor shield building annulus was assumed.
- iv) No credit for charcoal filters was assumed.
- v) Containment isolation was not assumed.
- vi) The 2-hour accident meteorological dilution factor was assumed.

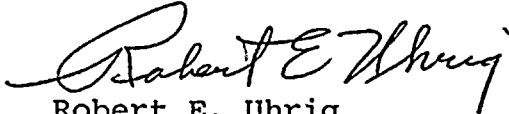
770870204

To: Dennis L. Ziemann, NRC  
Re: St. Lucie Unit No. 1  
Docket No. 50-335  
Postulated Refueling Accident

March 21, 1977  
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Since the bounding evaluation assumed that radionuclides emerging from the pool are transported immediately to the receptor under adverse meteorological conditions with the resultant doses being small fractions of 10 CFR 100 guideline values, and a more realistic evaluation would yield significantly smaller doses, Mr. Pollard's concern is clearly not relevant to the St. Lucie facility. Accordingly, we believe further evaluation of this issue is unwarranted. If you require any additional clarifying data, we would be pleased to furnish it.

Yours very truly,



Robert E. Uhrig  
Vice President

REU/LLL/bb

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