NEXTera® ENERGY POINT BEACH

May 2, 2018

NRC 2018-0017
10 CFR 50.71(e)

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

10 CFR 50.59 Evaluation and Commitment Change Summary Report

NextEra Energy Point Beach (NextEra), LLC, is submitting the 10 CFR 50.59 Evaluation and Commitment Change Summary Report for the Point Beach Nuclear Plant (PBNP), Units 1 and 2, for calendar years 2016 and 2017.

Sincerely,

NextEra Energy Point Beach, LLC

Thomas P. Schneider for
THOMAS P. SCHNEIDER
Licensing Manager
Eric Schultz

Enclosure

cc: Administrator, Region III, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC

ENCLOSURE

NEXTERA ENERGY POINT BEACH, LLC
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

10 CFR 50.59 & COMMITMENT CHANGE
SUMMARY REPORT FOR 2016 & 2017

10 CFR 50.59 Evaluations

Modification: EC 286906, Point Beach Unit 2 Cycle 36 Core Reload Modifications.

Activity Description:

Engineering Change (EC) 286906 is a design change for the Unit 2 Cycle 36 reactor core reload analysis following Westinghouse WCAP-9272 methodology.

Summary of Evaluation:

The 50.59 evaluated the following:

- The use of the PARAGON computer code for Unit 2 Cycle 36 is an activity that uses a revised or different method of evaluation for performing safety analyses than what is described in the FSAR.

Conclusion:

No activity requiring prior NRC approval per 10 CFR 50.59 was identified and no Technical Specification change is involved. [EVAL 2017-001]

Modification: EC 284250, Point Beach Unit 1 Cycle 37 Core Reload Modifications.

Activity Description:

Engineering Change (EC) 284250 is a design change for Unit 1 Cycle 37 reactor core reload analysis following Westinghouse WCAP-9272 methodology.

Summary of Evaluation:

The 50.59 evaluated the following:

- The use of the PARAGON computer code for Unit 1 Cycle 37 is an activity that uses a revised or different method of evaluation for performing safety analyses than what is described in the FSAR

Conclusion:

No activity requiring prior NRC approval per 10 CFR 50.59 was identified and no Technical Specification change is involved. [EVAL 2015-002]

FSAR Change: EC 286205, FSAR Chapter 11 Changes are needed to be consistent with the elimination of Technical specification 3.7.14, Primary Auxiliary Building Ventilation (VNPAB) in regards to control room dose.

Activity Description:

EC 286205 is a change to FSAR Section 11.2 to be consistent with Technical Specification 3.7.14, Primary Auxiliary Building Ventilation (VNPAB) deletion. The waste gas decay tank (WGDT), volume control tank (VCT), and charcoal decay tank (CDT), rupture calculations revisions assumed the control room atmospheric dispersion factors based on VNPAB being out of service.

Summary of Evaluation:

The 50.59 evaluated the following:

- Adverse impact on the current WGDT/VCT and CDT rupture control room dose analyses as described in the FSAR.

Conclusion:

No activity requiring prior NRC approval per 10 CFR 50.59 was identified and no Technical Specification change is involved. [EVAL 2016-001]

Modification: EC 282784, NFPA-805 Modification to Condenser Steam Dump and Moisture Separator/Reheaters Steam Supply Instrument Air Isolation (LAR MOD-14)

Activity Description:

EC 282784, MOD-14 addresses the potential fail open scenarios associated with multiple spurious operation concerns by installing solenoid valves in the instrument air lines supplying the condenser steam dump valves (CSDVs) and Reheat Steam Inlet Valves, to the Moisture Separator/Reheaters (MSRs). A manually activated switch for each Unit was installed in a common control switch panel to operate the new solenoid valves. Each switch operates the new solenoid valves on the respective Unit. This power supply is dependent on equipment in the main control room and cable spreading room, and an uninterruptible power supply provides adequate power supply to the solenoid valves for a period of at least six hours to meet the NFPA-805 requirements for the design.

Summary of Evaluation:

The 50.59 evaluated the following:

- The adverse impacts on condenser steam dump load rejection design functions and methods of controlling design functions and moisture separator reheater steam reheat design functions.

Conclusion:

No activity requiring prior NRC approval per 10 CFR 50.59 was identified and no Technical Specification change is involved. [EVAL 2015-001]

Modification: EC 289216, Branch River Project Switchyard Change Package

Activity Description:

EC 289216 is for the changes to Branch River Substation. American Transmission Company (ATC) is constructing Branch River Substation. Because of this new substation, two 345kV lines leaving Point Beach Nuclear Plant (PBNP) Switchyard, Line 111 to Sheboygan Energy Center Switchyard and Line 121 to Forest Junction Substation, are bisected and re-terminated at four new line positions at Branch River Substation. The 345kV lines between PBNP and Branch River remain lines 111 and 121. The line segment between Branch River and Sheboygan Energy Center becomes Line W-14. The line segment between Branch River and Forest Junction becomes Line W-15.

Summary of Evaluation:

The 50.59 evaluated the following:

- The interconnection of Lines 111 and 121 at the Branch River substation is considered an adverse impact on the independence or isolation between the various sources of electrical power.
- The location of Lines 111 and 121 within close proximity at the Branch River substation is considered an adverse impact on the independence or isolation between the various sources of electrical power.

Conclusion:

No activity requiring prior NRC approval per 10 CFR 50.59 was identified and no Technical Specification change is involved. [EVAL 2018-001]

COMMITMENT CHANGE EVALUATIONS

Steam Generator Storage Building: The original commitment was to ensure no combustibles would be stored in the Steam Generator Storage Building. The commitment was specifically to ensure that a fire in the building would not cause a release of radioactive corrosion products from the used Steam Generator Lower Assemblies (SGLAs). The commitment was modified to allow storage of contaminated combustible materials in the south half of the building. This change canceled the commitment.

Justification for Change: The commitment was specifically for prevention of a radioactive release from the used SGLAs. The used SGLAs were disposed of offsite. The Steam Generator Storage Building was repurposed and is used for FLEX equipment. Therefore, the commitment is no longer applicable. (CCE 2016-002)

Operating Experience for Pre-Job Briefs: The original commitment was to implement a new electronic method for access of operating experience (OE) for pre-job briefs for Maintenance. The change canceled this commitment.

Justification for Change: INPO has enhanced the means for searching of OE that has simplified how individuals can search and obtain the needed OE for inclusion within pre-job briefs. Therefore, station electronic methods for accessing OE for pre-job briefs for Maintenance or other station personnel is no longer needed. (CCE 2017-001)

Procedure Guidance: The original commitment was adding guidance to the sites maintenance rule monitoring procedure for considering unavailability of support systems versus supported systems. The change canceled this commitment.

Justification for Change: Guidance for unavailability of support systems is included in the Fleet procedure and industry guidance within NUMARC 93-01. (CCE 2017-002)