



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION I
2100 RENAISSANCE BOULEVARD, SUITE 100
KING OF PRUSSIA, PA 19406-2713

May 1, 2018

Mr. Timothy S. Rausch
President and Chief Nuclear Officer
Susquehanna Nuclear, LLC
769 Salem Blvd. NUCSB3
Berwick, PA 18603

SUBJECT: SUSQUEHANNA STEAM ELECTRIC STATION – INTEGRATED
INSPECTION REPORT 05000387/2018001 AND 05000388/2018001

Dear Mr. Rausch:

On March 31, 2018, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Susquehanna Steam Electric Station (SSES), Units 1 and 2. On April 13, 2018, the NRC inspectors discussed the results of this inspection with Derek Jones, Plant Manager, and other members of your staff. The results of this inspection are documented in the enclosed report.

No NRC-identified or self-revealing findings were identified during this inspection. NRC inspectors documented a licensee-identified violation which was determined to be of very low safety significance in this report. The NRC is treating this violation as a non-cited violation (NCV) consistent with Section 2.3.2.a of the Enforcement Policy.

If you contest the violation or significance of the NCV, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region I; the Director, Office of Enforcement; and the NRC Resident Inspector at Susquehanna.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and the NRC's Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* (10 CFR), Part 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

James A. Krafty, Acting Chief
Reactor Projects Branch 4
Division of Reactor Projects

T. Rausch

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Docket Numbers: 50-387 and 50-388
License Numbers: NPF-14 and NPF-22

Enclosure:
Inspection Report 05000387/2018001 and 05000388/2018001

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SUBJECT: SUSQUEHANNA STEAM ELECTRIC STATION – INTEGRATED INSPECTION
 REPORT 05000387/2018001 AND 05000388/2018001, DATED MAY 1, 2018

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Number(s): 50-387 and 50-388

License Number(s): NPF-14 and NPF-22

Report Number(s): 05000387/2018001 and 05000388/2018001

Enterprise Identifier: I-2018-001-0069

Licensee: Susquehanna Nuclear, LLC (Susquehanna)

Facility: Susquehanna Steam Electric Station, Units 1 and 2

Location: Berwick, Pennsylvania

Inspection Dates: January 1, 2018 to March 31, 2018

Inspectors: L. Micewski, Senior Resident Inspector
T. Daun, Resident Inspector
A. Turilin, Project Engineer
J. Furia, Senior Health Physicist
M. Modes, Senior Reactor Inspector

Approved By: J. Krafty, Acting Chief
Reactor Projects Branch 4
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring Susquehanna's performance at Susquehanna Steam Electric Station Units 1 and 2 by conducting the baseline inspections described in this report in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information. NRC-identified and self-revealed findings, violations, and additional items are summarized in the table below. Licensee-identified non-cited violations are documented in report section: 71111.15.

Additional Tracking Items

Type	Issue number	Title	Report Section	Status
LER	05000388/2017-008-01	Condition Prohibited by Technical Specifications Due to a Loose Terminal Block Associated with Primary Containment Isolation Valves	71153	Closed
LER	05000387(388)/2017-003-00	Loss of Secondary Containment Zone 3 Due to Fan Trip	71153	Closed

Plant Status

Unit 1 began the inspection period at 100 percent power. On January 8, 2018, operators reduced power to approximately 75 percent, performed rod pattern adjustments and returned the unit to 100 percent the same day. On February 15, 2018, operators' commenced end-of-cycle coast down for the remainder of the cycle. On March 30, 2018, operators commenced reducing power from 76 percent for a planned refueling outage. Unit 1 reached Mode 4 on March 31, 2018 and remained shut down for the remainder of the inspection period.

Unit 2 began the inspection period at 100 percent power. On February 12, 2018, operators reduced power to approximately 65 percent to perform maintenance on a reactor feed pump. Operators returned the unit to 100 percent the next day, and remained at or near 100 percent power for the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors performed plant status activities described in IMC 2515 Appendix D, "Plant Status" and conducted routine reviews using IP 71152, "Problem Identification and Resolution." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess Susquehanna's performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.01 - Adverse Weather Protection

Impending Severe Weather (1 Sample)

The inspectors evaluated readiness for impending adverse weather conditions for extremely cold temperatures on January 3, 2018 through January 6, 2018.

71111.04 - Equipment Alignment

Partial Walkdown (4 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 1, high pressure coolant injection during reactor core isolation cooling injection valve relay replacement on January 24, 2018
- (2) Unit 2, residual heat removal (RHR) during division 2 swing bus maintenance on January 30, 2018
- (3) Unit Common, 'E' emergency diesel generator (EDG) while substituted in for 'D' EDG during 'D' EDG mid-cycle overhaul on February 5, 2018
- (4) Unit Common, 'B' emergency service water (ESW) during 'D' subsystem pump maintenance and relay replacement on February 13, 2018

71111.05 - Fire Protection

Quarterly Inspection (5 Samples)

The inspectors evaluated fire protection program implementation in the following selected areas:

- (1) Unit Common, 'D' EDG room (fire zone 0-41D) on January 4, 2018
- (2) Unit 2, load center room II-510 and II-507 (fire zones 2-5F and 2-5G) on February 5, 2018
- (3) Unit Common, refuel floor (fire zone 0-8A) on March 1, 2018
- (4) Unit Common, computer room and computer maintenance room and office (fire zones 0-24E and 0-24F) on March 5, 2018
- (5) Unit 1, RHR 'B' pump room (fire zone 1-1E) on March 14, 2018

71111.06 - Flood Protection Measures

Internal Flooding (1 Sample)

The inspectors evaluated internal flooding mitigation protections in Unit 1, reactor building elevation 749' on February 8, 2018.

71111.07 - Heat Sink Performance

Heat Sink (Triennial) (3 Samples)

The inspectors evaluated heat exchanger/heat sink performance on the following components:

- (1) Unit 1 and 2, RHR heat exchangers, Section 02.02b
- (2) Unit 2 spray pond, Section 02.02d. Specifically, Sections 02.02d.3 and 02.02d.6 were completed.

71111.11 - Licensed Operator Regualification Program and Licensed Operator Performance

Operator Regualification (1 Sample)

The inspectors observed and evaluated licensed operator simulator training, which included a momentary loss of 'B' engineered safeguard system electrical bus, a high level in the scram discharge volume due to a failed closed drain valve, an Anticipated Transient Without Scram, and the failure of select components to automatically start as required on February 12, 2018.

Operator Performance (1 Sample)

The inspectors observed reactor shutdown activities from 76% to MODE 3 on Unit 1 on March 30 – 31, 2018.

71111.12 - Maintenance EffectivenessRoutine Maintenance Effectiveness (2 Samples)

The inspectors evaluated the effectiveness of routine maintenance activities associated with the following equipment and/or safety significant functions:

- (1) Unit 1, scram discharge volume vent and drain valves not opening when scram reset on January 31, 2018
- (2) Unit 1, fuel pool cooling system on March 16, 2018

71111.13 - Maintenance Risk Assessments and Emergent Work Control (5 Samples)

The inspectors evaluated the risk assessments for the following planned and emergent work activities:

- (1) Unit Common elevated risk during 'D' EDG post maintenance testing on February 9, 2018
- (2) Unit Common relay replacement and maintenance on 'D' ESW pump on February 13, 2018
- (3) Unit 1 repair of 'D' RHR pump local circuit breaker control switch on March 14, 2018
- (4) Unit 1 online functional test of RHR low pressure logic from remote shutdown panel on March 26, 2018
- (5) Unit 1 online risk associated with logic testing from the remote shutdown panel on March 29, 2018

71111.15 - Operability Determinations and Functionality Assessments (7 Samples)

The inspectors evaluated the following operability determinations and functionality assessments:

- (1) Unit Common, 'C' ESW pump field jumper installed not in accordance with design drawings on January 17, 2018
- (2) Unit Common, EDG operability when drywell unit cooling fan 2V413B is slow to stop on February 5, 2018
- (3) Unit 1, impact to equipment in reactor building elevation 749' with floor drain blockage on February 8, 2018
- (4) Unit Common, 'D' EDG air receivers ultrasonic testing results less than minimum wall thickness on February 27, 2018
- (5) Unit 1, division 2 core spray with non-restrained ESW replacement piping staged next to room cooler on March 20, 2018
- (6) Unit 1, reactor core isolation cooling oil filter clogging on March 21, 2018
- (7) Unit Common, Zone 3 secondary containment increased inleakage past hatch 1H24 on March 23, 2018

7111.19 - Post Maintenance Testing (6 Samples)

The inspectors evaluated post maintenance testing for the following maintenance/repair activities:

- (1) Unit Common, 'D' EDG air receivers internal inspection on February 15, 2018
- (2) Unit Common, 'C' EDG electronic governor replacement on March 6, 2018
- (3) Unit 1, 'B' standby liquid control pump and actuator overhaul on March 13, 2018
- (4) Unit Common, 'D' EDG mid-cycle overhaul on March 13, 2018
- (5) Unit 1, 'D' RHR pump lateral switch repairs on March 15, 2018
- (6) Unit 1, division 1 RHR service water installation of pulsation dampener on March 22, 2018

7111.20 - Refueling and Other Outage Activities (Partial Sample)

The inspectors evaluated Unit 1 refueling outage 1R20 activities from March 30, 2018 to March 31, 2018. The inspectors completed inspection procedure Sections 03.01.a and 03.01.c1.

7111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Routine (4 Samples)

- (1) Unit Common, ESW logic system functional test on January 2, 2018
- (2) Unit Common, 'E' EDG monthly run on January 3, 2018
- (3) Unit Common, 'B' EDG integrated surveillance test on January 29 and 30, 2018
- (4) Unit 1, RHR service water flow surveillance on March 22, 2018

Inservice (1 Sample)

- (1) Unit Common, quarterly ESW flow verification Loop 'A' on February 22, 2018

7114.06 - Drill EvaluationDrill/Training Evolution (1 Sample)

The inspectors evaluated the emergency director, dose assessment personnel, and communicators' emergency preparedness implementation during five tabletop training exercises between January 26, 2018 and March 9, 2018. The inspectors observed two different scenarios. One scenario involved a simulated airborne radiological release following core damage due to a large amount of foreign material. The second scenario involved a steam leak at the reactor feed pump turbine, that was unable to be isolated due to main steam isolation valve failure.

RADIATION SAFETY

71124.03 - In-Plant Airborne Radioactivity Control and Mitigation

The following inspection activities were completed to close-out the 2016/2017 Reactor Oversight Process biennial cycle:

Use of Respiratory Protection Devices (1 Sample)

The inspectors evaluated respiratory protection.

Self-Contained Breathing Apparatus for Emergency Use (1 Sample)

The inspectors evaluated the licensee's self-contained breathing apparatus program.

Problem Identification and Resolution (1 Sample)

The inspectors evaluated whether problems associated with radiation monitoring and exposure control (including operating experience) were identified at an appropriate threshold and properly addressed in the corrective action program.

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below for the period from January 1, 2017 through December 31, 2017. (6 Samples)

- (1) Unit 1 and 2 Unplanned Scrams
- (2) Unit 1 and 2 Unplanned Scrams with Complications
- (3) Unit 1 and 2 Unplanned Power Changes

71153 - Follow-up of Events and Notices of Enforcement Discretion

Licensee Event Reports (2 Samples)

The inspectors evaluated the following licensee event reports which can be accessed at <https://lersearch.inl.gov/LERSearchCriteria.aspx>:

- (1) Licensee Event Report (LER) 05000388/2017-008-01, Condition Prohibited by Technical Specifications Due to a Loose Terminal Block Associated with Primary Containment Isolation Valves, on March 22, 2018. The review for this event is documented in Inspection Report 05000387/2017003 and 05000388/2017003.
- (2) LER 05000387(388)/2017-003-00, Loss of Secondary Containment Zone 3 Due to Fan Trip, on March 29, 2018. The review for this event is documented in Inspection Report 05000387/2017001 and 05000388/2017001.

INSPECTION RESULTS

Licensee Identified Non-Cited Violation	71111.15
<p>This violation of very low safety significant was identified by the licensee and has been entered into the licensee corrective action program and is being treated as a Non-Cited Violation, consistent with Section 2.3.2 of the Enforcement Policy.</p>	
<p>Violation: Susquehanna Unit 1 TS section 5.4.1 requires that “written procedures shall be implemented covering the applicable procedures recommended in Regulatory Guide 1.33, Revision 2, Appendix A, February 1978.” Susquehanna’s implementing instruction NDAP-QA-0503, General Housekeeping, Transient Material and Internal Cleanliness, Revision 45 implements aspects of the Regulatory Guide administrative procedures requirements. NDAP-QA-0503 section 6.1.5.h requires, in part, that “transient equipment shall be located such that it will not impact safety related equipment during a seismic event. Locate all items at a distance greater than the height of the item from safety related equipment.” Additionally, TS 3.5.1 Action Statement I directs immediate entry into Limiting Condition for Operation (LCO) 3.0.3 if one core spray subsystem is inoperable with one low pressure coolant injection (LPCI) subsystem inoperable. LCO 3.0.3 requires action to be taken within 1 hour to place the unit in MODE 2 within 7 hours and MODE 3 within 13 hours.</p>	
<p>Contrary to the above, from December 1, 2017 to December 3, 2017, Susquehanna staged a 540 pound, ten foot long replacement pipe on 34 inch high stands within 34 inches of the safety related Unit 1, “B” Core Spray room cooler. Susquehanna concluded that the room cooler was inoperable because the pipe could have reasonably contacted and damaged the flexible conduit for the power cable to the room cooler during a seismic event. Additionally, from 7:48 a.m. on December 2, 2017 to 1:35 p.m. on December 3, 2017, maintenance was performed on the Unit 1, division 2 LPCI swing bus motor generator which rendered the division 2 LPCI system inoperable. During this time, Susquehanna did not perform the required actions of LCO 3.0.3 and remained in MODE 1.</p>	
<p>Significance/Severity Level: This violation is of very low safety significance (Green), since this finding did not represent a loss of system, a loss of function of at least a single train for greater than its TS allowed outage time, or a loss of a non-TS train.</p>	
<p>Corrective Action Reference(s): CR-2017-20227; CR-2018-01717; CR-2018-02250</p>	

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On April 13, 2018, the inspectors presented the quarterly resident inspector inspection results to Derek Jones, Plant Manager, and other members of the staff.

DOCUMENTS REVIEWED**71111.01**Procedures

NDAP-00-1913, Seasonal Readiness, Revision 10

OI-AD-029, Emergency Load Control, Revision 17

ON-NATPHENOM-001, Severe Weather/Natural Phenomena, Revision 56

NDAP-00-0030, Severe Weather/Natural Disaster Preparation, Revision 12

71111.04Condition Reports (*initiated in response to inspection)

CR-2017-14989 CR-2018-00378 CR-2018-00931 CR-2018-01073

CR-2018-02126 CR-2018-02452 CR-2018-02492 CR-2018-02494

CR-2018-02500

Drawings

FF62008, Common Fire Barrier Upgrade Conduit Interface at Concrete Q-Decking or Penetration Seal Sketch, Sheet 26, Revision 1

FF62008, Common Fire Barrier Upgrade Conduit Interface at Concrete Q-Decking or Penetration Seal Sketch, Sheet 1, Revision 2

FF62009, Unit 1 Fire Barrier Upgrade E1K758 Wrapped Raceway Sketch, Sheet 361, Revision 0

M-182, Common P&ID Diesel Generator 'E' Building Air Flow Diagram, Sheet 2, Revision 9

M-134, Common P&ID Diesel 'E' Auxiliaries (Starting Air Systems), Sheet 6, Revision 7

M-5200, Common Diesel Generator 'E' Bldg., General Arrangement Sections, Sheet 2, Revision 2

M-156, Unit 1 HPCI Lubricating and Control Oil P&ID, Sheet 1, Revision 9

M-156, Unit 1 P&ID HPCI Turbine- Pump, Sheet 1, Revision 36

Miscellaneous

Level of Effort Checklist, CR-2017-14989

71111.05AQMiscellaneous

FP-013-198, Diesel Generator Bay 'D' Fire Zone 0-41D Elevations 677', 660' and 710, Revision 4

FP-213-258, Load Center Room (II-510) Load Center Room (II-507) Fire Zone 2-5F, 2-5G Elevation 749'-1", Revision 5

FP-013-141, Computer Maintenance Area (C-205, 206, 207) and Corridor (C-200, 204) Fire Zones 0-24F, 0-24B Elevation 698'-0", Revision 7

FP-013-140, Computer Room (C-202) Fire Zone 0-24E Elevation 698'-0", Revision 7

FP-113-105, RHR Pump Room 'B' (I-13) Fire Zone 1-1E Elevation 645'-0", Revision 3

FP-013-132, Common Refueling Floor 9I-810, II-810) Fire Zone 0-8A Elevation 818'-1", Revision 4

71111.06Condition Reports (*initiated in response to inspection)

CR-2018-02051

DrawingsC-2729, Unit 1 Reactor Building Station Flood Barrier Plan of Elevation 749'-1", Sheet 1,
Revision 1MiscellaneousEC-FLOD-1001, Evaluation of Response to INPO ER 11-1, Recommendations 3 & 4 for Station
Flooding, Revision 2EC-FLOD-0001, Internal Flooding Evaluations for Moderate Energy Pipe Cracks and Sprinkler
system Actuations, Revision 3**71111.07**Procedures

NEIM-00-1165, Cooling Tower Structural Inspections, Revision 1

NDAP-QA-1163, Structural Monitoring Program, Revision 4

NSEI-AD-103, System Engineering Instruction for Outage Inspection of the Cooling Tower,
Revision 1

NSEP-AD-0002, Station Health Process, Revision 16

Maintenance Orders/Work Orders

PCWO 2144095

PCWO 2102869

PCWO 1995536

CalculationsNuclear Engineering Calculation Cover Sheet NEPM-QA-0221-1, Calculation EC-016-1003,
Heat Sink - Maximum Water Loss Design Basis Analysis, Revision 6

Design Calculation 22A2597, Heat Exchanger, Residual Heat Removal, Revision 1

Miscellaneous

Spray Pond Level Readings for February 2 - 27, 2018

RHR Chemistry Surveillance, February 3, 2017 and February 9, 2017

2E205A, 2017 Chemistry Sample Report

2E2058, 2015 Final Eddy Current Results Map

71111.11Miscellaneous

GO-100-004

GO-100-005

OP-AD-300

71111.12Condition Reports (*initiated in response to inspection)

CR-2017-11560 CR-2018-04438 CR-2018-04439

Miscellaneous

EACE for CR 2017-11560, Revision 0

ACT-01-CR-2017-11560

ACT-05-CR-2017-11560

71111.13Condition Reports (*initiated in response to inspection)

CR-2018-03945

Miscellaneous

Protected Equipment, Unit 1 'B' RHR Pump

SO-149-020 On-line Function Test of RHR LP B from RSDP

Scheduler's Evaluation for Susquehanna Unit 1 & 2 March 24- 28, 2018

71111.15Condition Reports (*initiated in response to inspection)

CR-2018-00301 CR-2018-01870 CR-2018-02051 CR-2018-02237

CR-2018-02238* CR-2018-02372 CR-2018-02398 CR-2018-02494

CR-2018-02513 CR-2018-03152 CR-2018-04322*

Action Requests

AR-2017-20603 AR-2018-02561

Drawings

E-146, Common Schematic Diagram ESW Pump C, Sheet 5, Revision 27

E109-13, Unit 1 Medium Voltage Switchgear Safeguard Connection Diagram Bus 1C IA20303,
Sheet 54, Revision 21

E-146, Common Schematic Diagram ESW Pump 0P504C, Sheet 6, Revision 50

E-347-3, Unit 1 Connection Diagram 4.16KV Switchgear 1A203, Sheet A, Revision 36

E-347-3, Unit 1 Connection Diagram 4.16KV Switchgear 1A203, Sheet C

FF-105800, Design report of Air Starting Receiver

ACT-01-CR-2018-02398

M-150, Unit 1 P&ID RCIC Turbine Lube Oil, Revision 2

Miscellaneous

NDAP-QA-0703-1, Prompt Operability Determination CR-2018-02051

BOP-UT-18-008, Supplemental Report

NDE-UT-014, UT Thickness Examination

SE-270-001, Revision 16, Attachment A

SO-170-001, Revision 3, Attachment A

SE-070-011, Revision 16, Attachment A

SE-270-011, Revision 15, Attachment A

SE-070-011, Revision 19, Attachment A

Level of Effort Checklist, CR-2018-02250

EC-034-0016, Verification of Reactor Building Coolers Selection, Revision 0
 EC-RISK-0521, Reactor Building- Effects of Loss of HVAC on Components, Revision 4
 EC-LOCA-0500, COTTAP Analysis RB Post DBA Temperature Issue, Revision 5
 EC-034-0567, Power Uprate Impact Review- Reactor Building HVAC System, Revision 5

71111.19

Procedures

OT-024-147, Diesel Generator 'C' Restoration, Revision 0
 SO-153-004, Quarterly Standby Liquid Control Flow verification, Revision 45
 SO-149-008, 2 YR- RHR Logic System Functional Test (Div 2) Online

Condition Reports (*initiated in response to inspection)

CR-2018-02398 CR-2018-03945

Work Orders

2041900
 2066604

71111.22

Procedures

SO-024-B01, Diesel Generator 'B' Integrated Surveillance Test, Revision 1
 SO-024-014, Monthly Diesel Generator 'E' Operability Test, Revision 50
 SO-054-001C, 24 Month Logic System Functional test of the ESW System Pump 'C' Start and
 Reset Logic, Revision 1
 SO-054-A03, Quarterly ESW Flow Verification Loop A, Revision 17
 SO-116-A03, Quarterly RHR Service Water System Flow Verification Division 1, Revision 14

Condition Reports (*initiated in response to inspection)

CR-2018-00102 CR-2018-00167 CR-2018-01984 CR-2018-01990

71114.06

Condition Reports (*initiated in response to inspection)

CR-2018-03161 CR-2018-03162 CR-2018-03194

71151

Miscellaneous

DI-2017-08601

71124.03

Procedures

HP-TP-759, Inspection and Testing of Firehawk SCBA Protective Equipment, Revision 5

Miscellaneous

Respiratory Protection/TEDE ALARA Worksheets for RWP 20180034, LRW Filter and
 RWP 20180035, Carwash