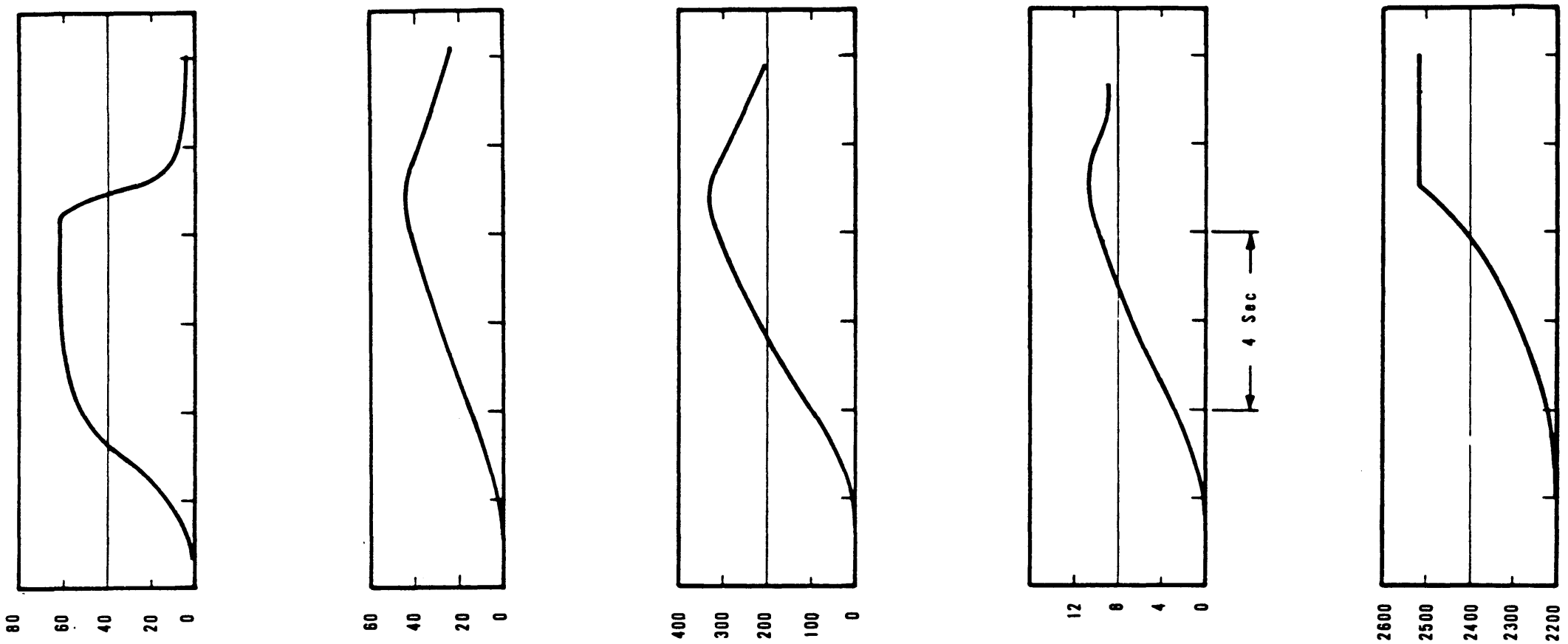


GPU Nuclear

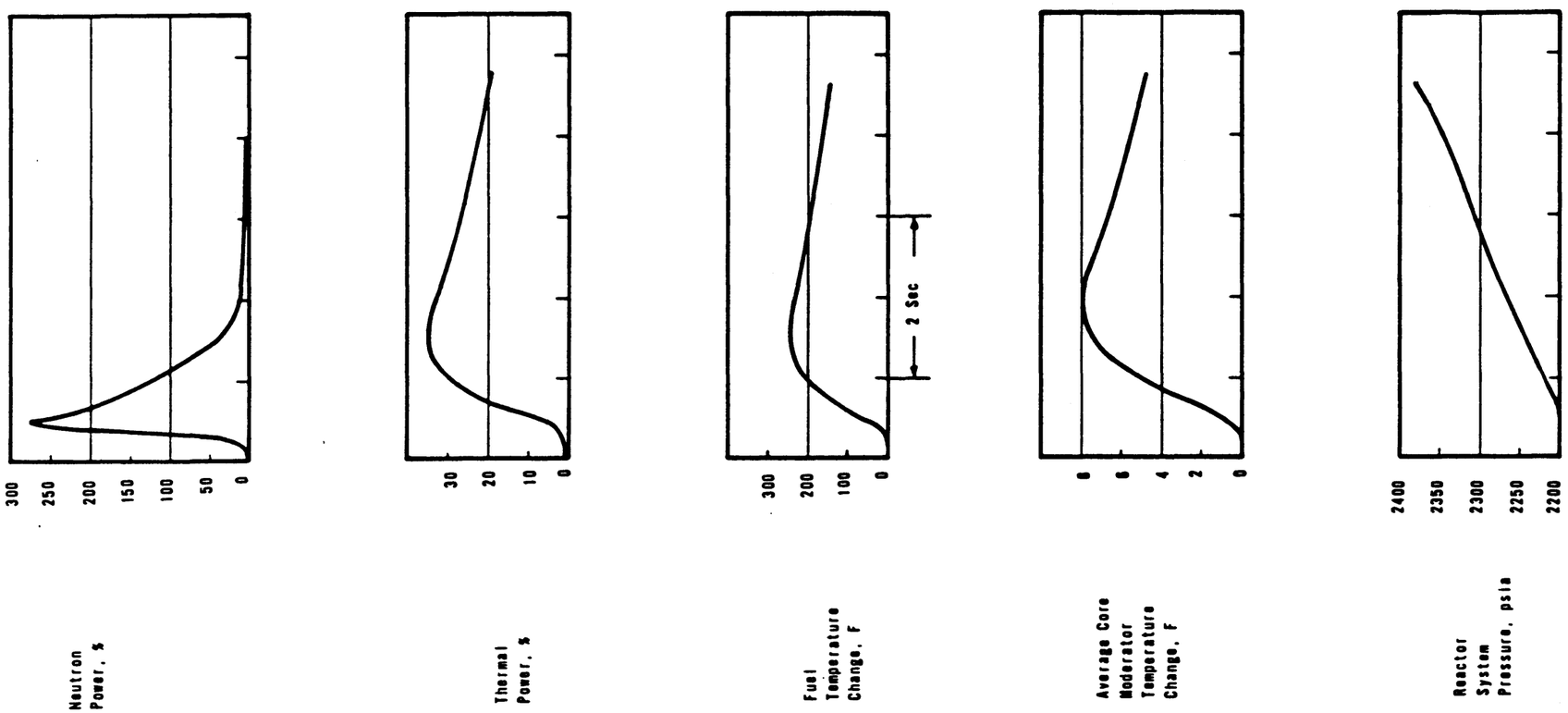
TMI Unit-1

Update -1

7/82

Startup Accident from 10^{-9} Rated Power using a 1.5 ΔK/K Rod Group High Pressure Reactor Trip is Actuated

Fig. 14.1-1



GPU Nuclear

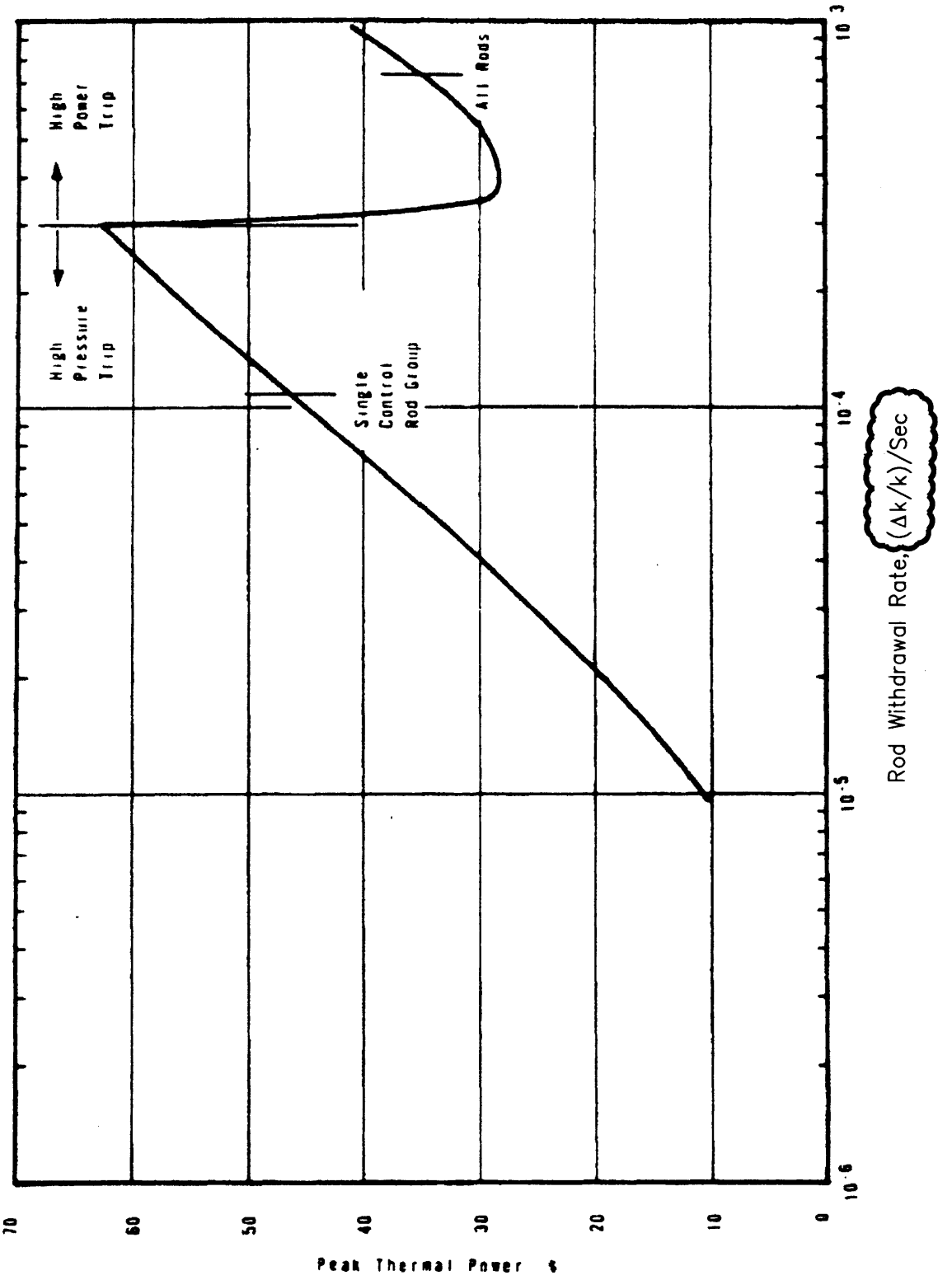
TMI Unit-1

Update -1

7/82

Startup Accident from 10^{-9} Rated Power using all Rods with a Worth of 10% ΔK/K; High Flux Reactor Trip is Actuated

Fig. 14.1-2



TMI - UNIT 1

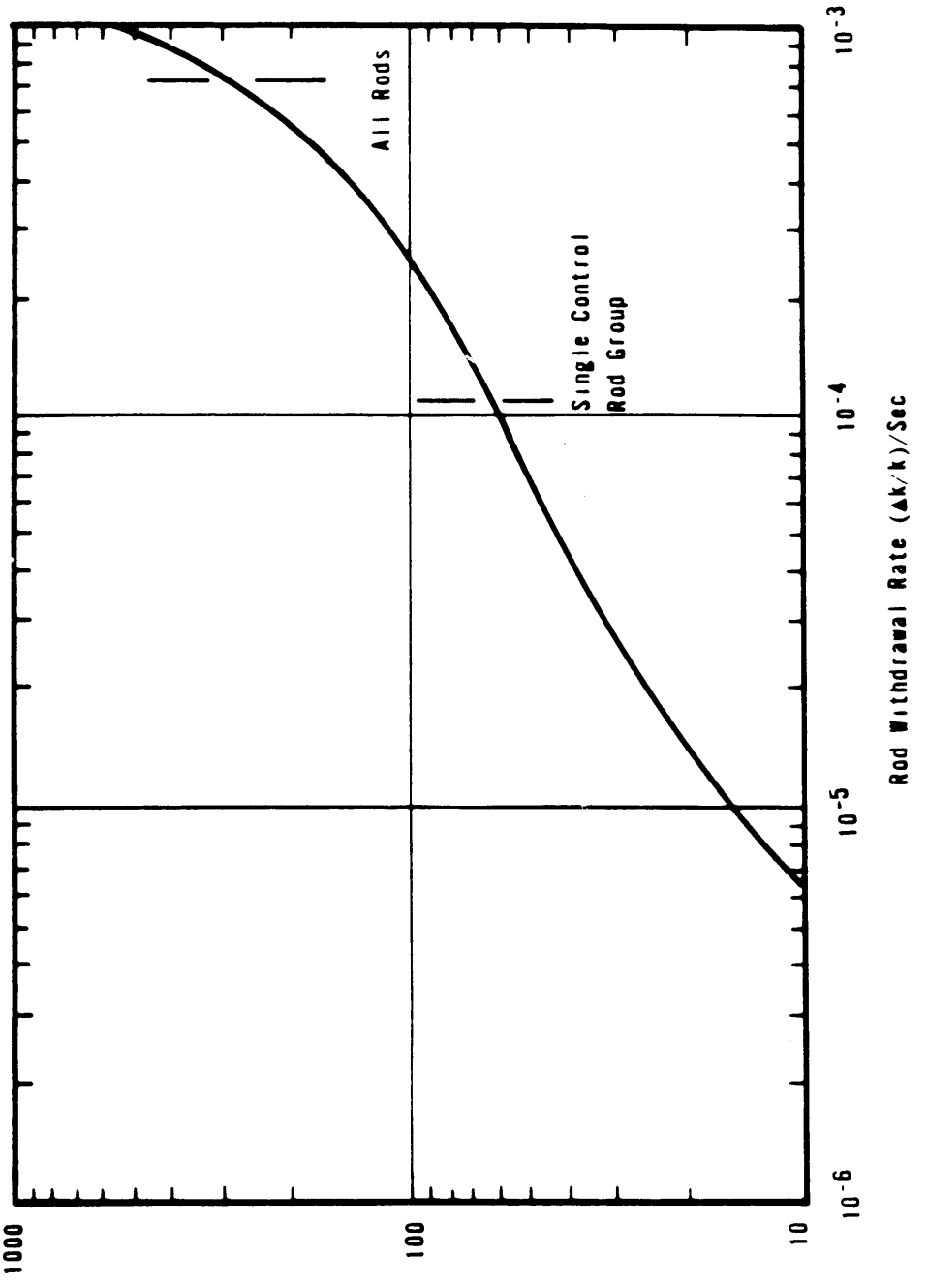
UPDATE - 14
4/98

GPU NUCLEAR

Peak Thermal Power vs. Rod Withdrawal Rate for a Startup Accident from 10^{-9} Rated Power

CAD FILE: 6555RI4.DWG

FIG 14.1-3



GPU Nuclear

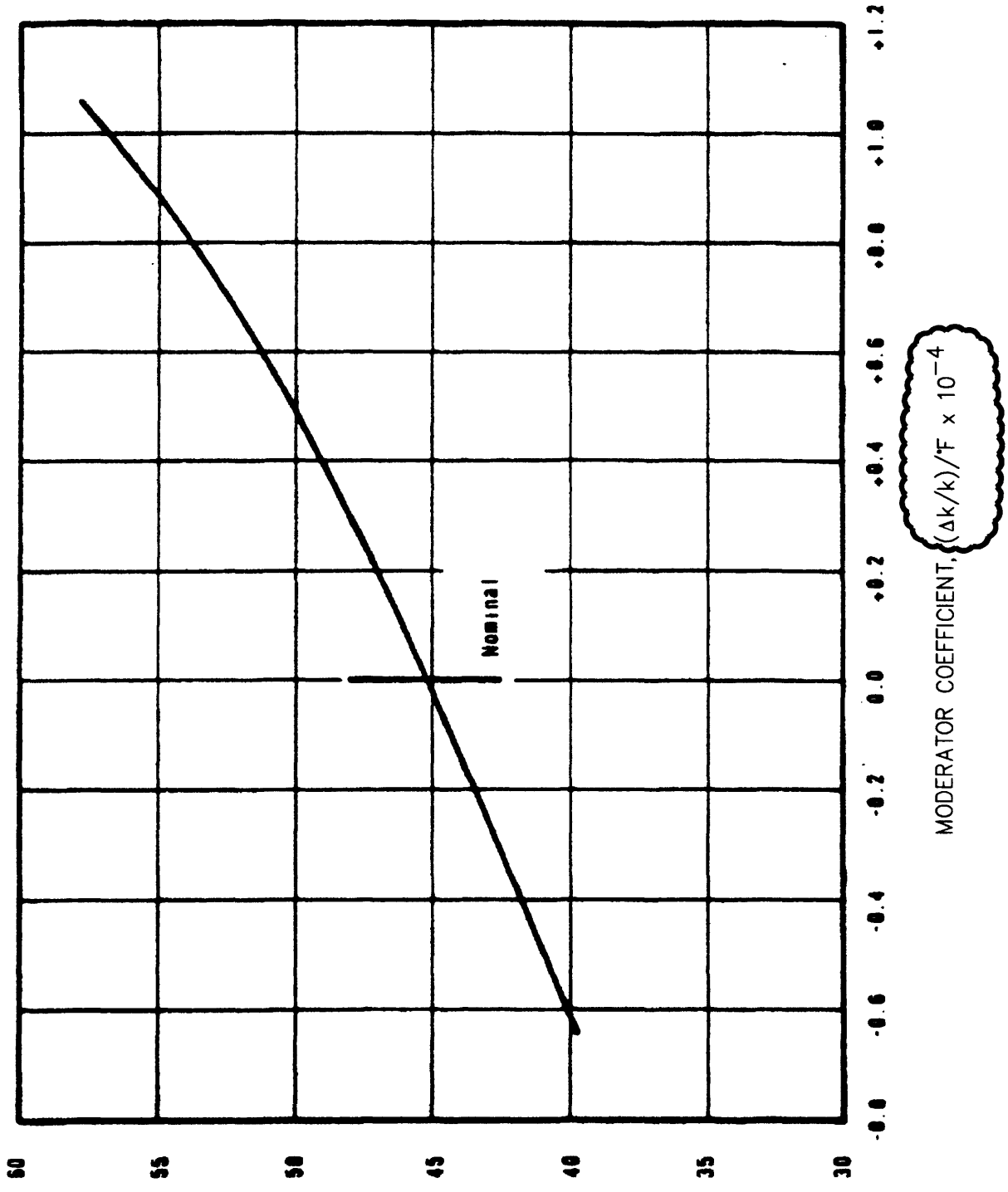
TMI Unit-1

Update -1

7/82

Peak Neutron Power vs. Rod Withdrawal Rate for a Startup Accident from 10^{-9} Rated Power

Fig. 14.1-4



TMI - UNIT 1

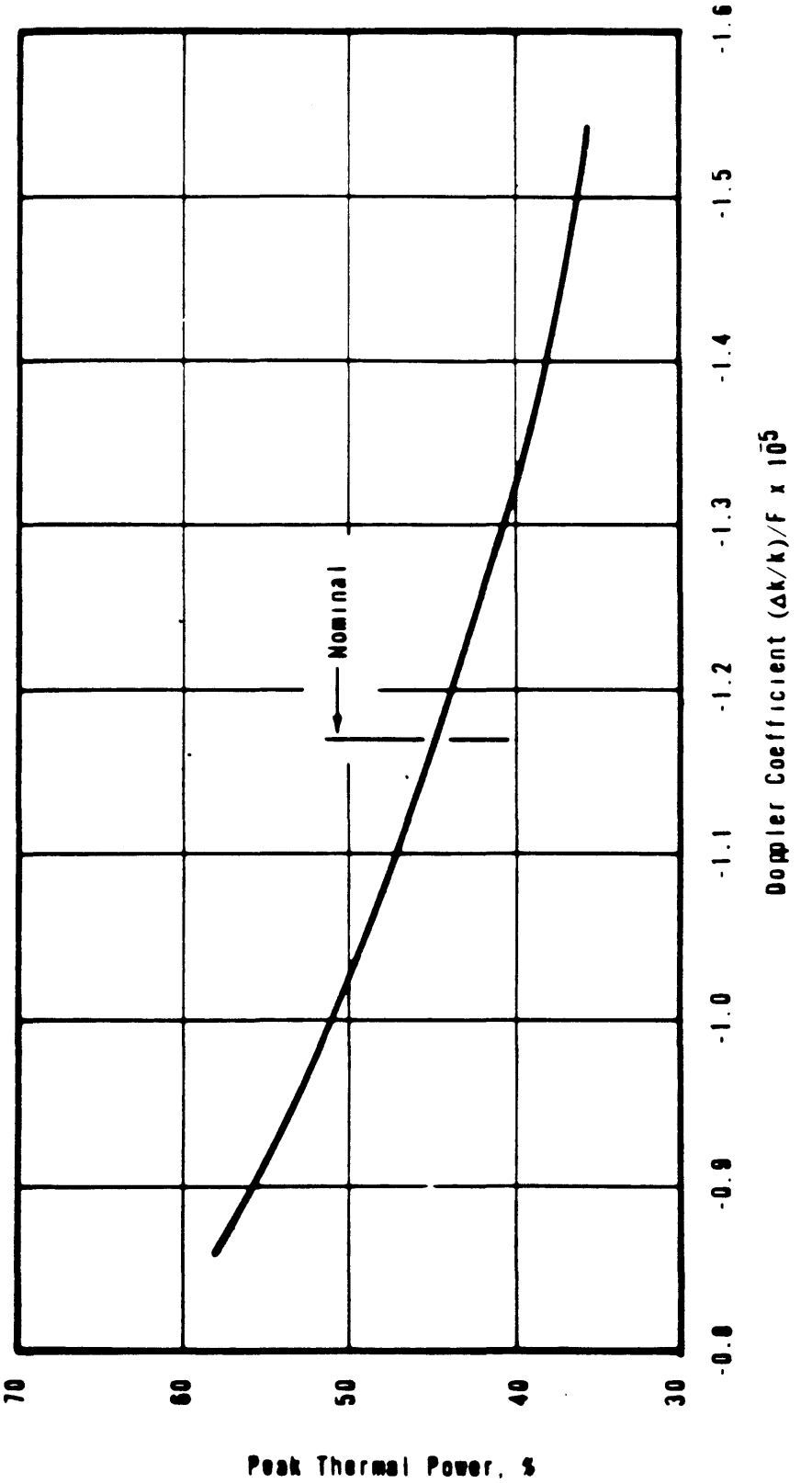
UPDATE - 14
04/98

GPU NUCLEAR

Peak Thermal Power vs. Moderator Coefficient for a Startup Accident using a 1.5% Δk/k Rod Group at 1.09×10^{-4} (Δk/k)/S FROM 10^{-9} Rated Power

CAD FILE: 6556RI4.DWG

FIG 14.1-5



GPU Nuclear

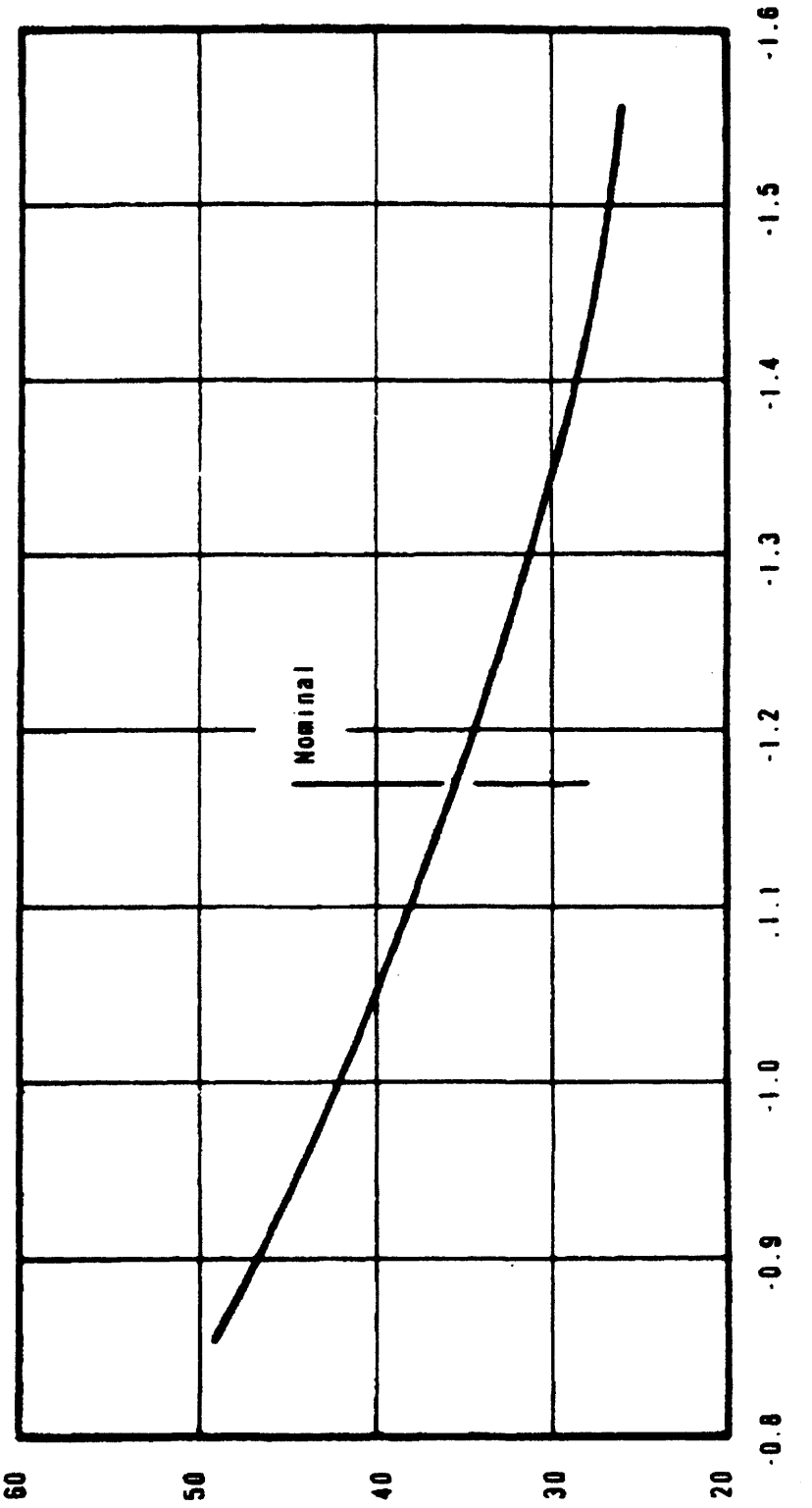
Update - 8R

TMI Unit 1

3/90

Peak Thermal Power Versus Doppler Coefficient for a Startup Accident Using a 1.5% ΔK/K Rod Group at 1.09 $X10^{-4}(\Delta K/K)/S$ From 10^{-9} Rated Power

Fig. 14.1-6



TMI - UNIT 1

UPDATE - 14
04/98

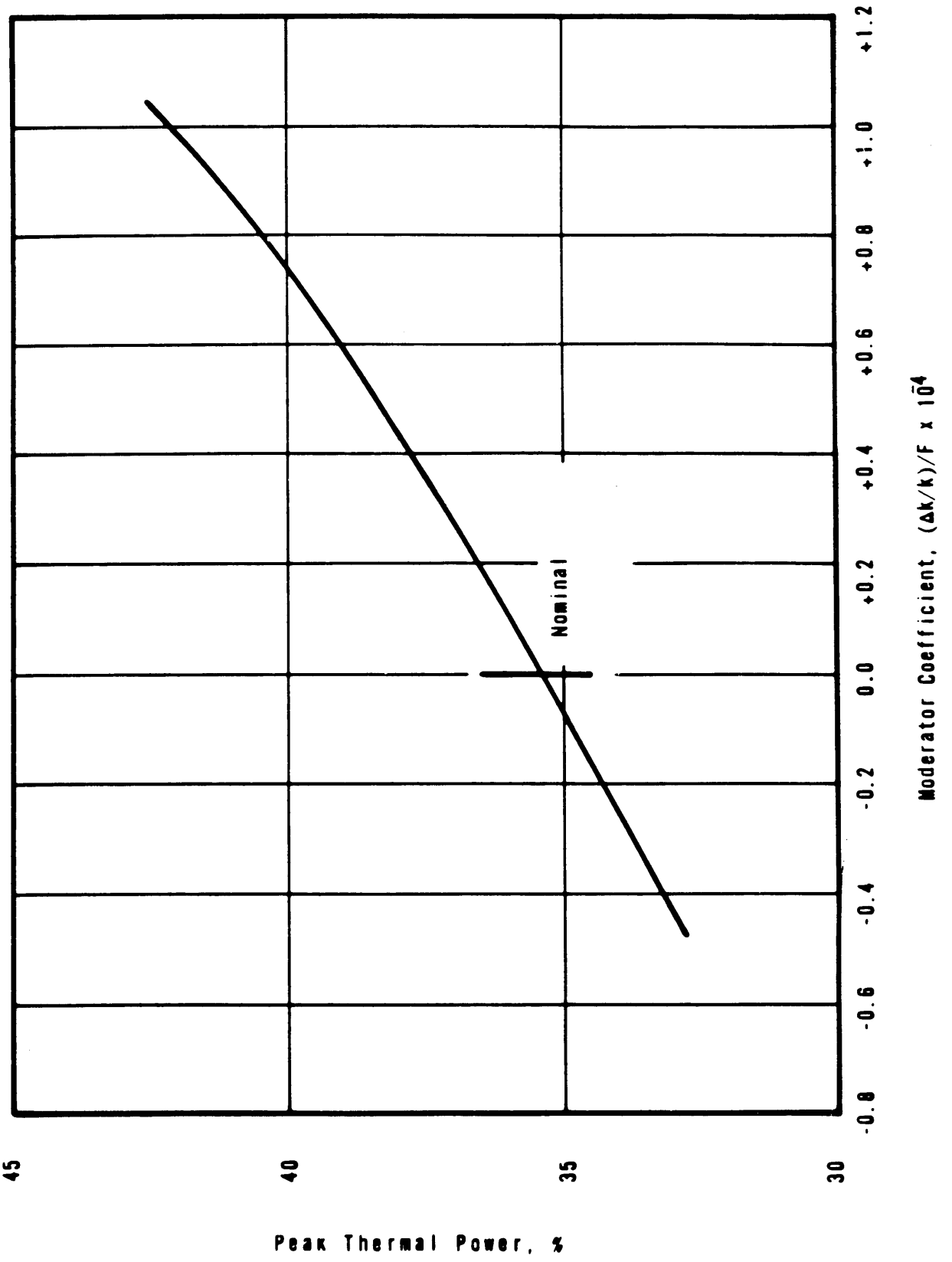
GPU NUCLEAR

Peak Thermal Power vs. Doppler Coefficient for a Startup Accident Using All Rods at 7.25 x 10^{-4} $(\Delta k/k)$/S from 10^{-9} Rated Power

TITLE LINE 2

CAD FILE: 6557RI4.DWG

FIG 14.1-7

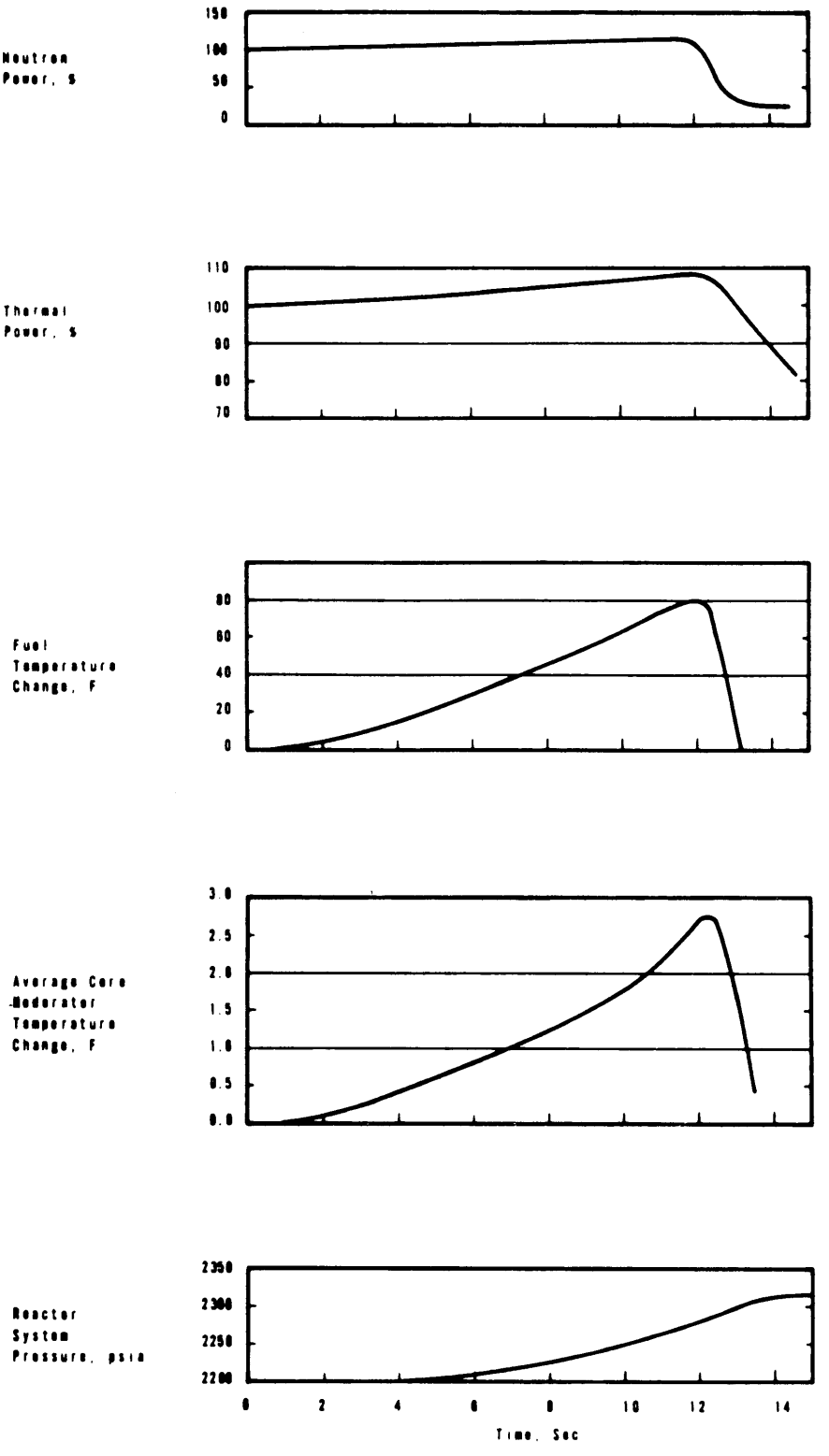


GPU Nuclear Update -1

TMI Unit-1 7/82

Peak Thermal Power vs. Moderator Coefficient for a Startup Accident Using all Rods at 7.25×10^{-4} $(\Delta K/K)/S$ From 10^{-9} Rated Power

Fig. 14.1-8



GPU Nuclear

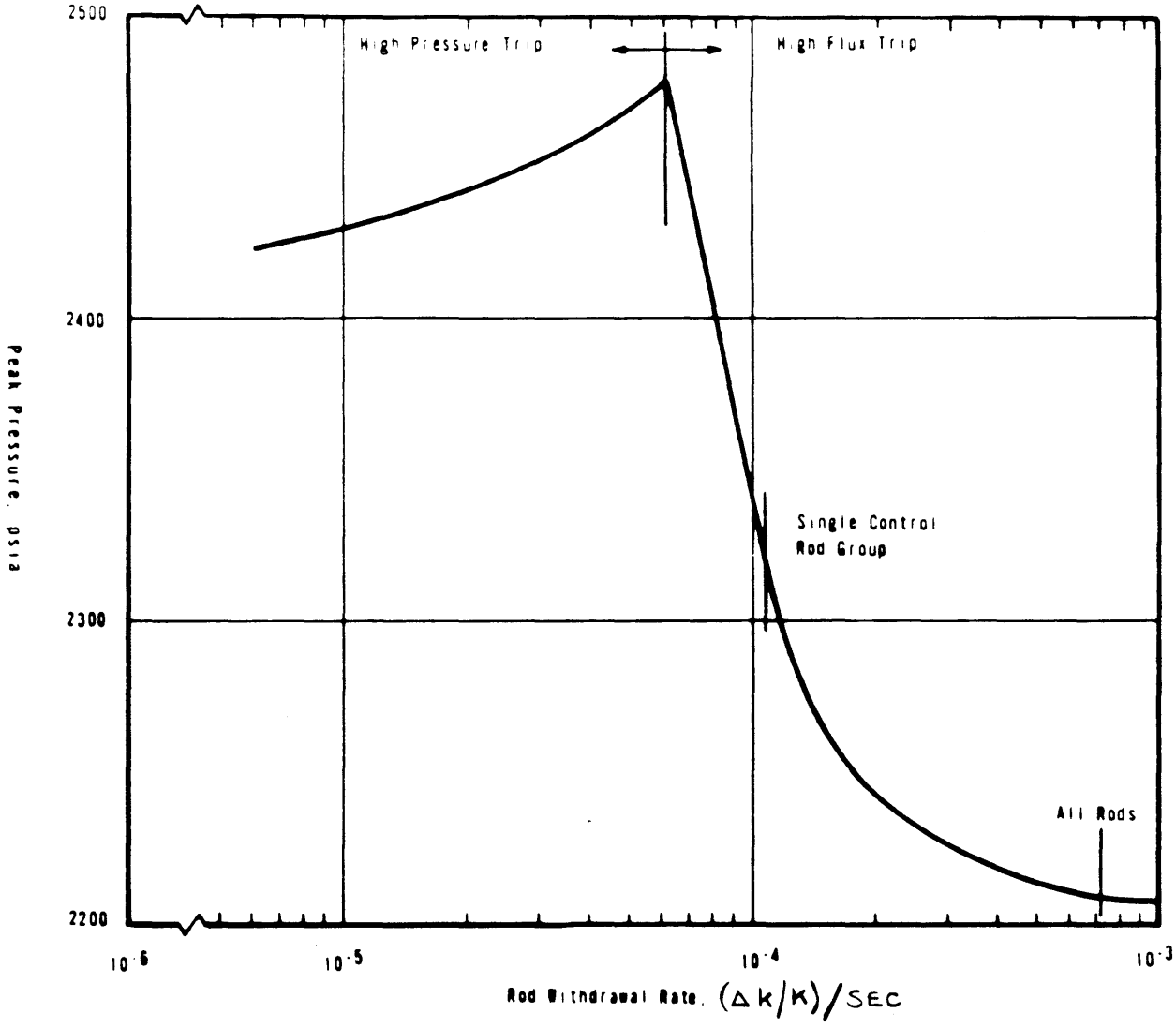
Update -1

TMI Unit-1

7/82

Rod Withdrawal Accident from Rated Power Using a 1.5% ΔK/K Rod Group at 1.09×10^{-4} (ΔK/K)/S; High Flux Reactor Trip is Actuated

Fig. 14.1-9



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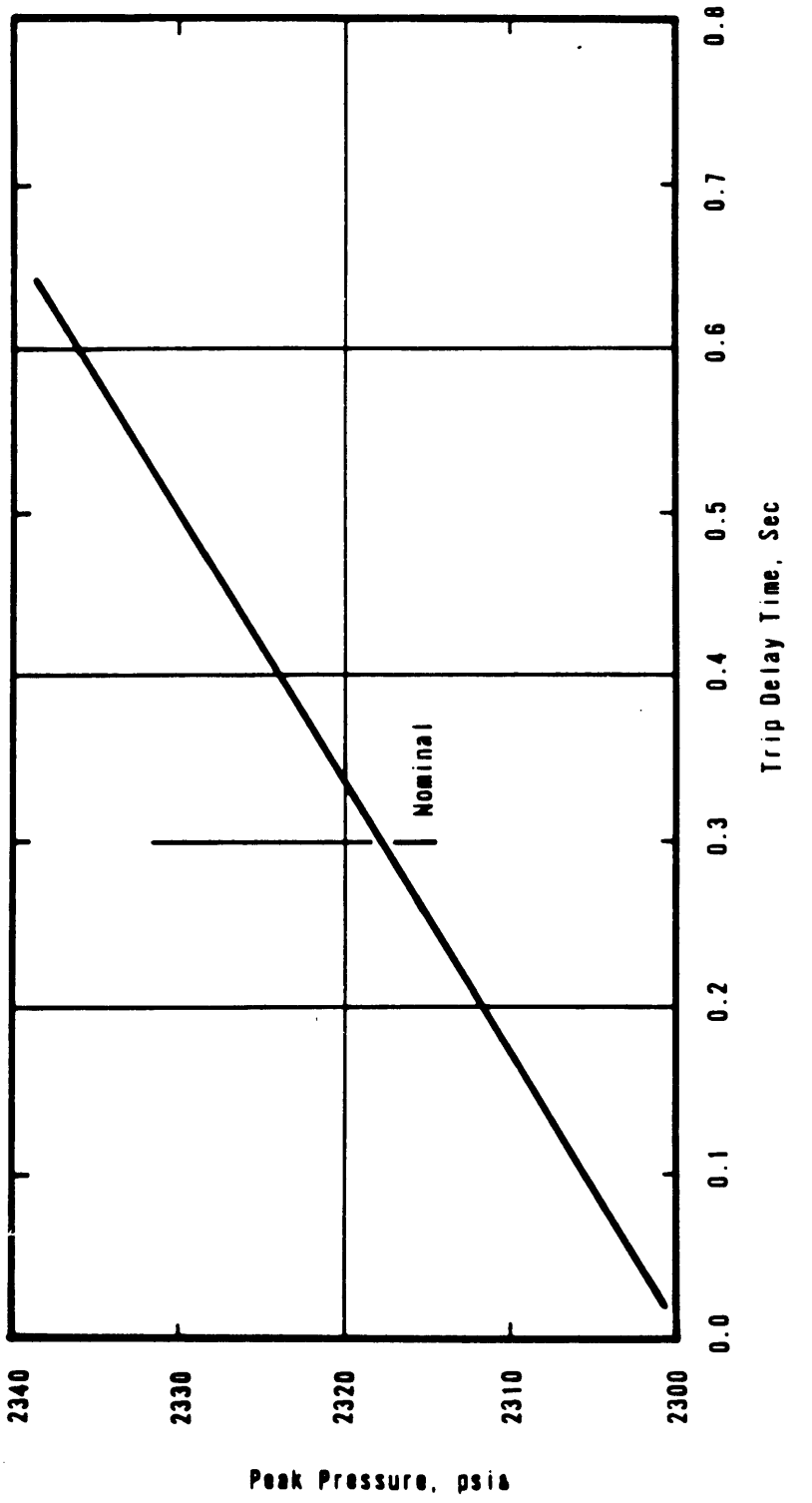
TMI Unit 1

Update - 8R

3/90

Peak Pressure vs. Rod Withdrawal Rate for a Rod Withdrawal Accident from Rated Power

Fig. 14.1-10

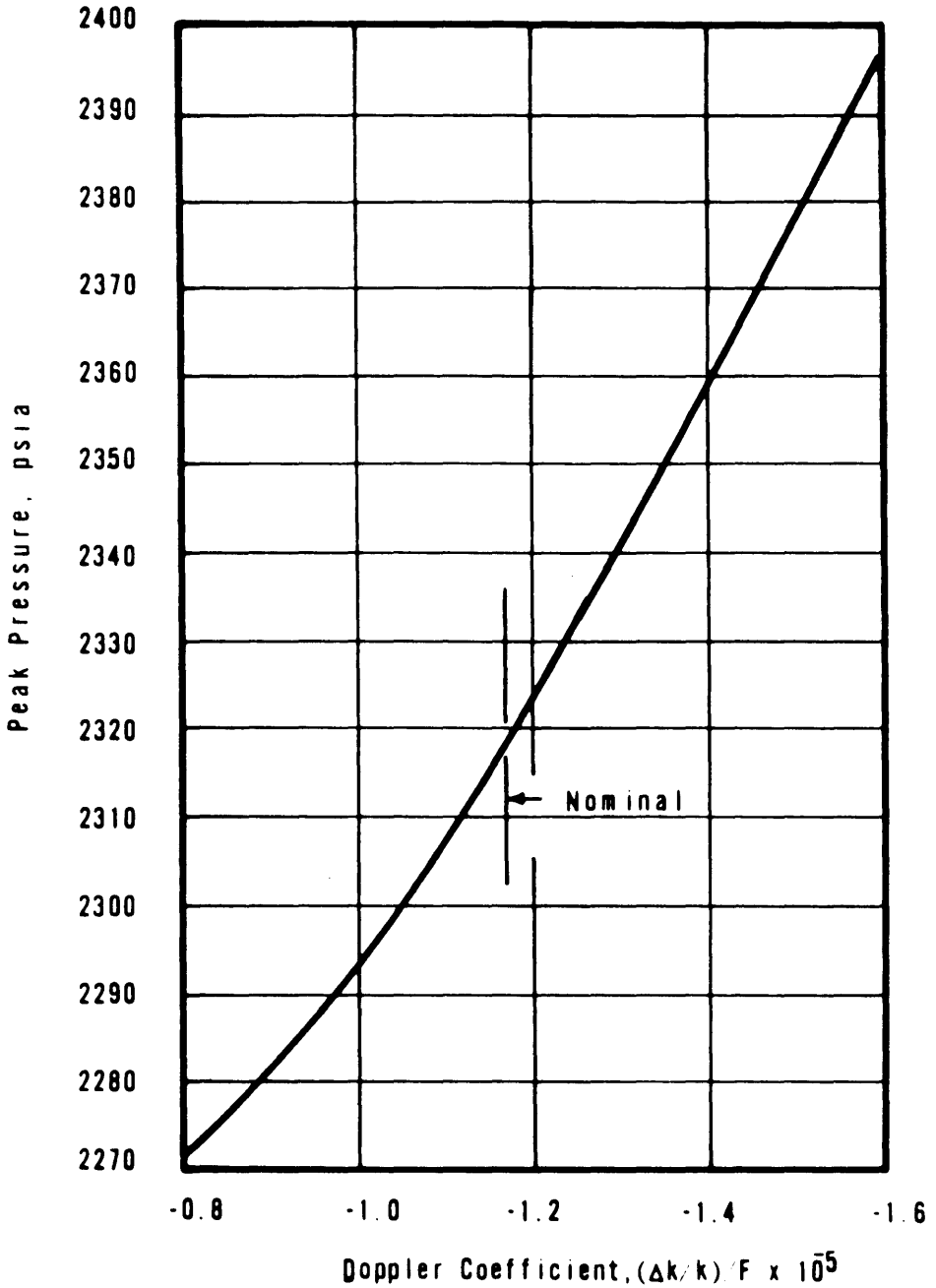


GPU Nuclear Update -1

TMI Unit-1 7/82

Peak Pressure vs. Trip Delay Time
For a Rod Withdrawal Accident from
Rated Power Using a 1.5% ΔK/K
Rod Group

Fig. 14.1-11



GPU Nuclear

TMI Unit-1

Peak Pressure vs. Doppler Coefficient for a Rod Withdrawal Accident from Rated Power Using a 1.5% △K/K Rod Group

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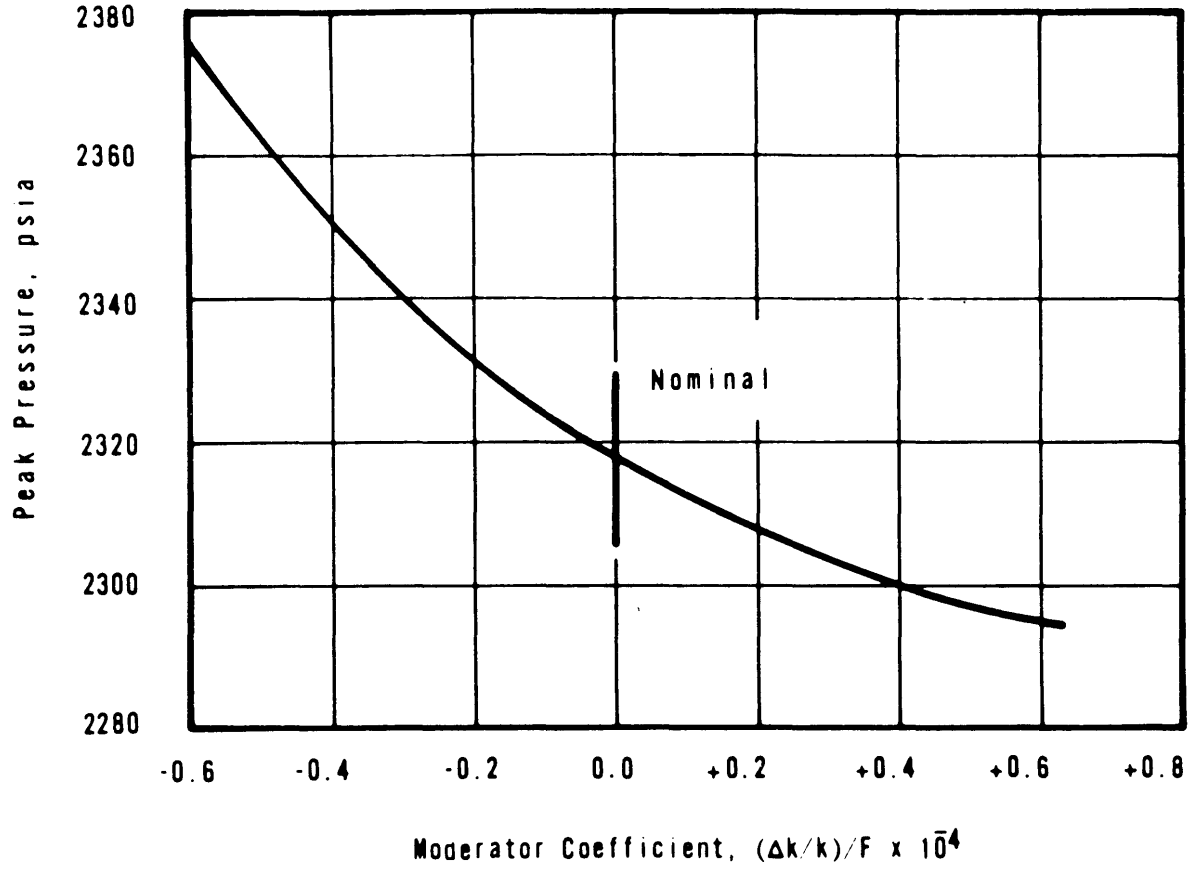
Fig. 14.1-12



GPU Nuclear

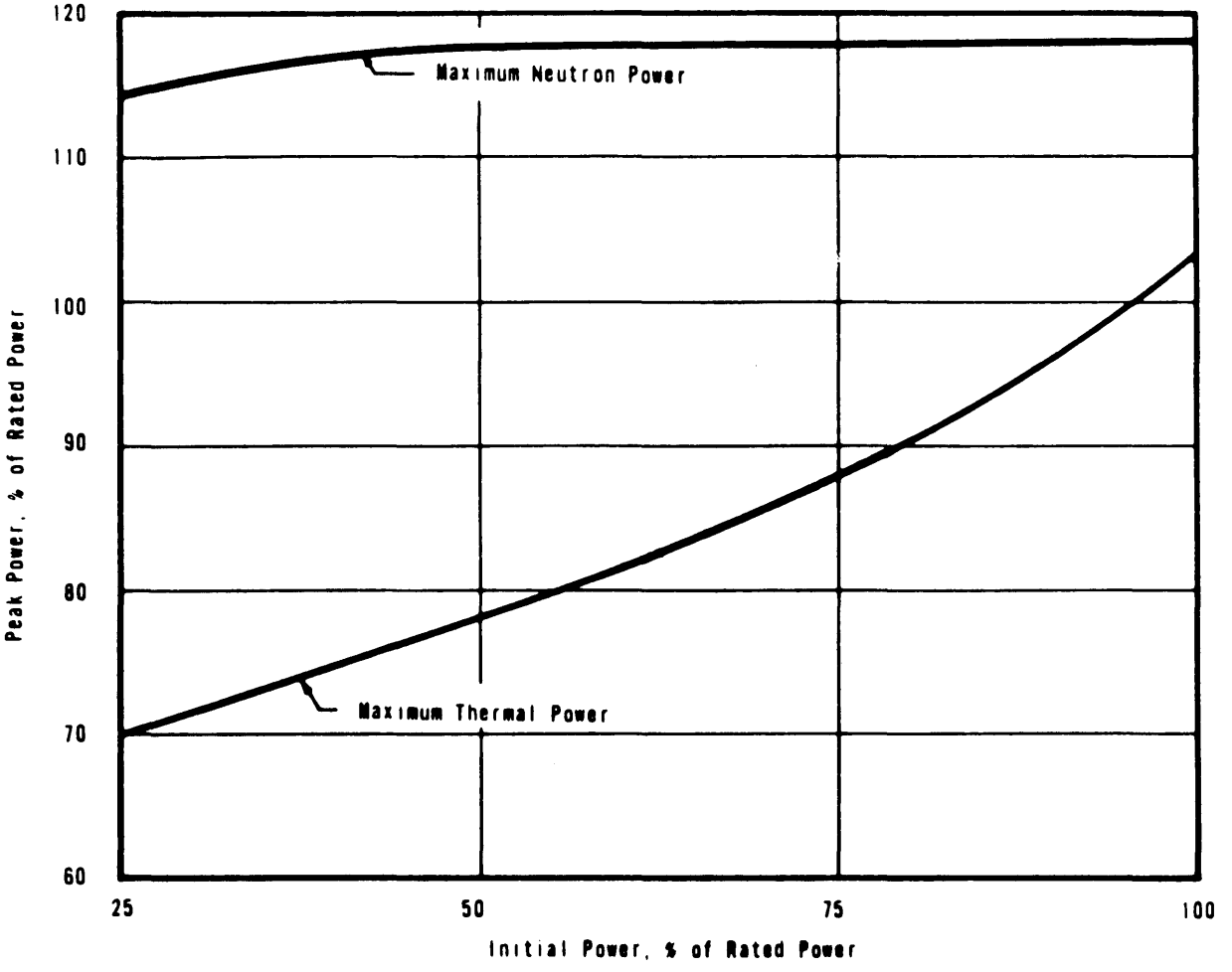
TMI Unit-1

Update -1

7/82

Peak Pressure vs. Moderator Coefficient for a Rod Withdrawal Accident from Rated Powr Using a 1.5% △K/K Rod Group

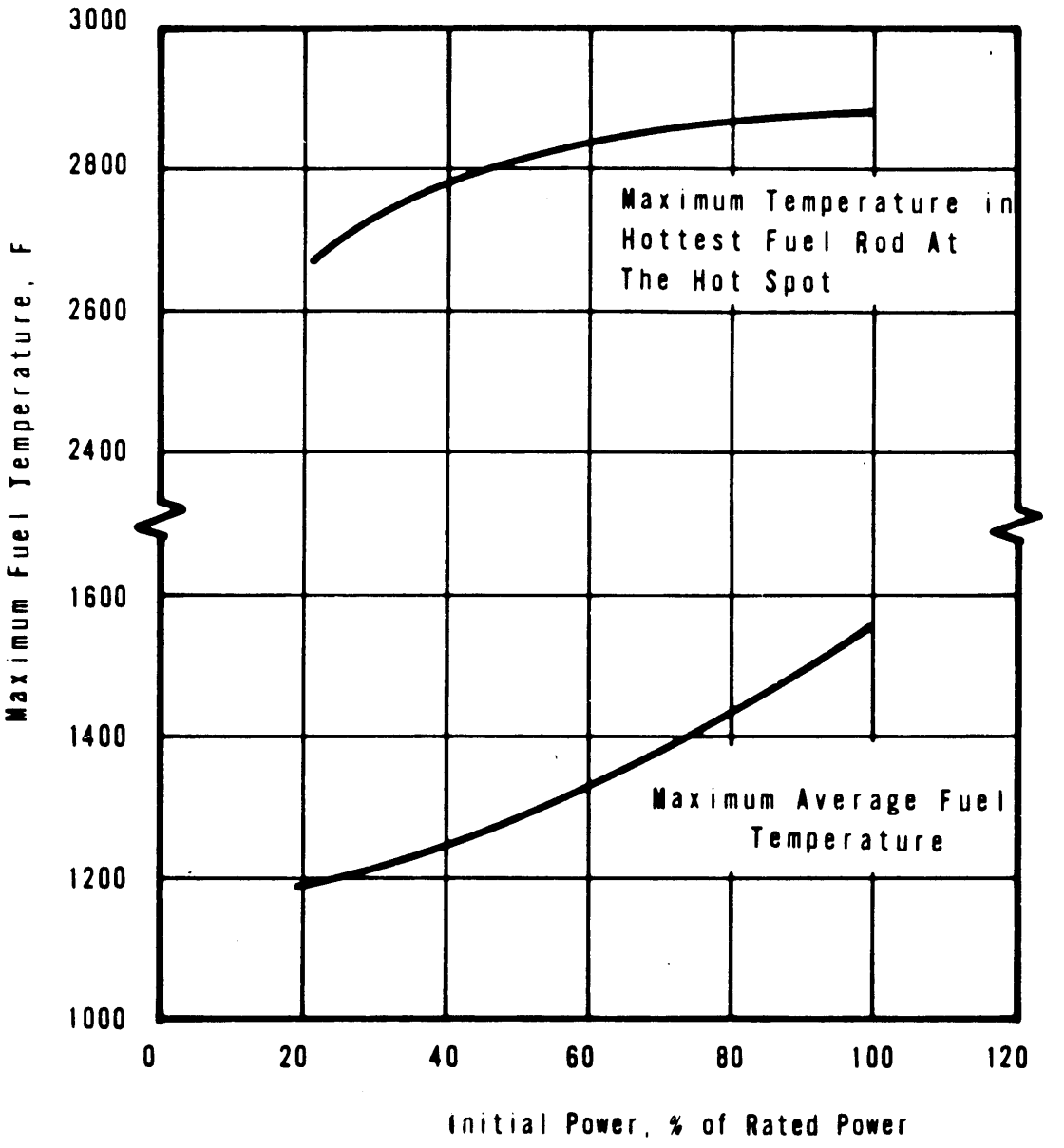
Fig. 14.1-13



GPU Nuclear | Update -1
TMI Unit-1 | 7/82

Maximum Neutron and Thermal Power for an All-Rod Withdrawal Accident from Various Initial Power Levels

Fig. 14.1-14



GPU Nuclear

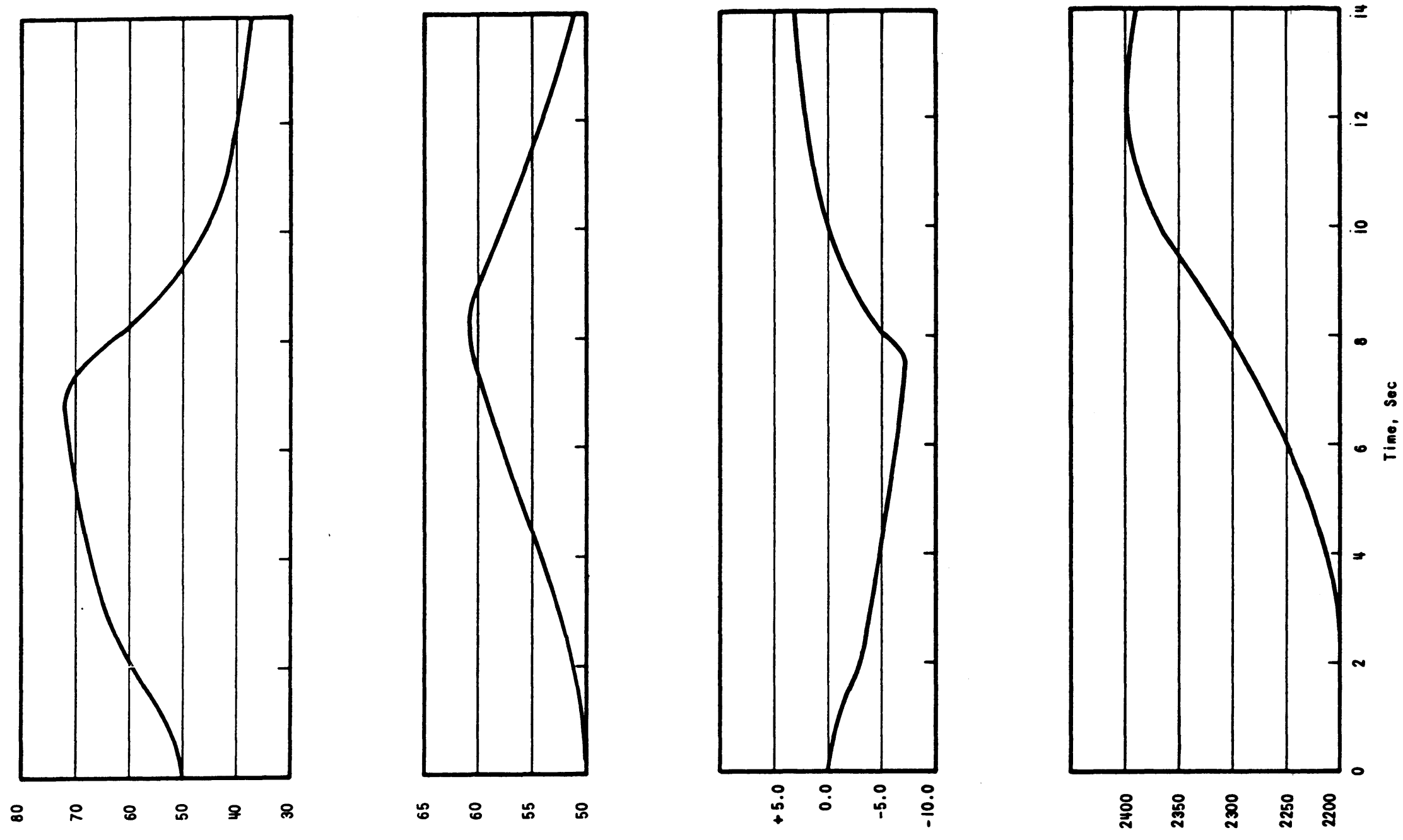
Update -1

TMI Unit-1

7/82

Peak Fuel Temperature in Average Rod and Hot Spot for an All-Rod Withdrawal Accident from Various Initial Power Levels

Fig. 14.1-15

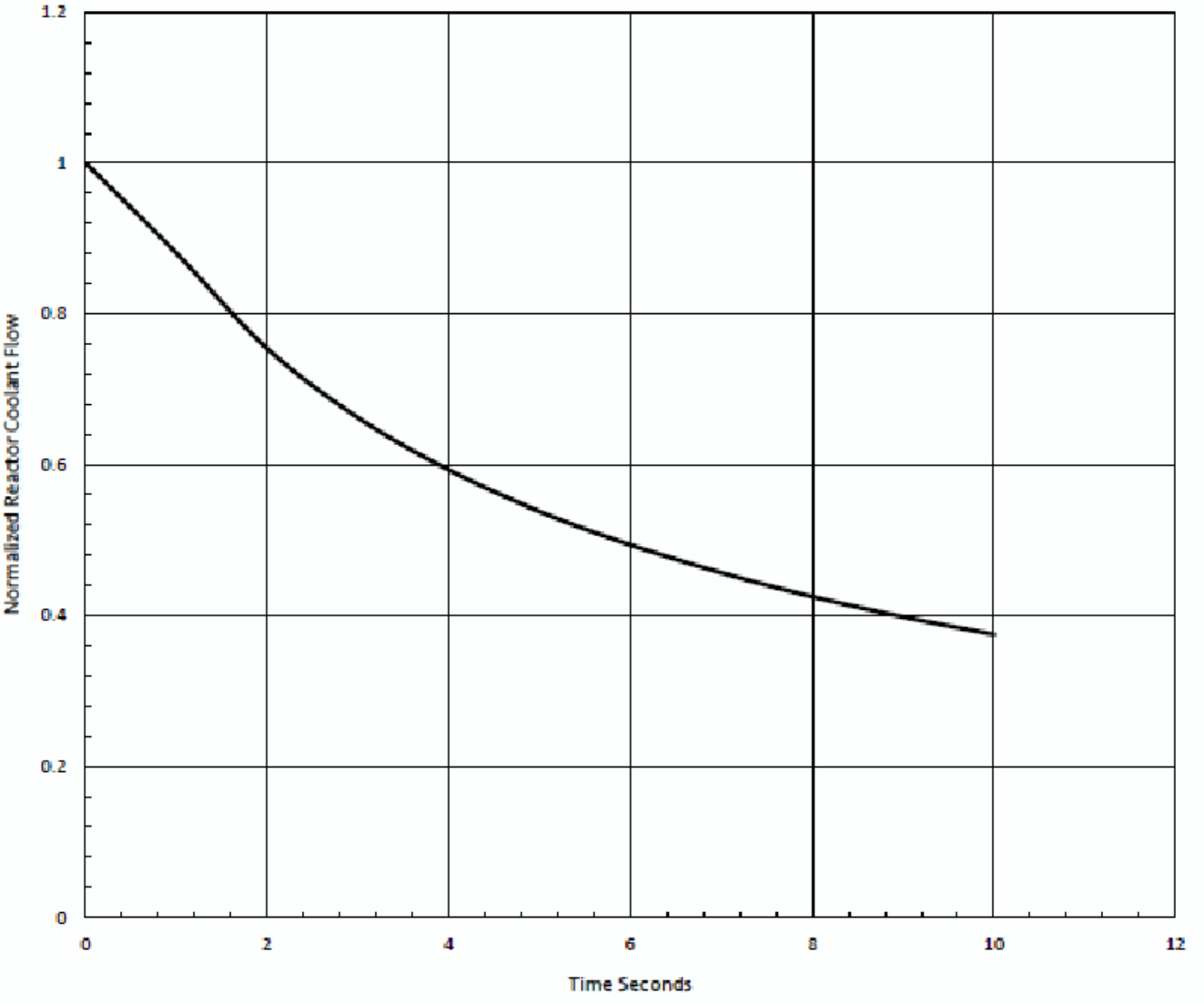


GPU Nuclear Update -1

TMI Unit-1 7/82

Pump Startup from 50% Power and 50% Flow

Fig. 14.1-16



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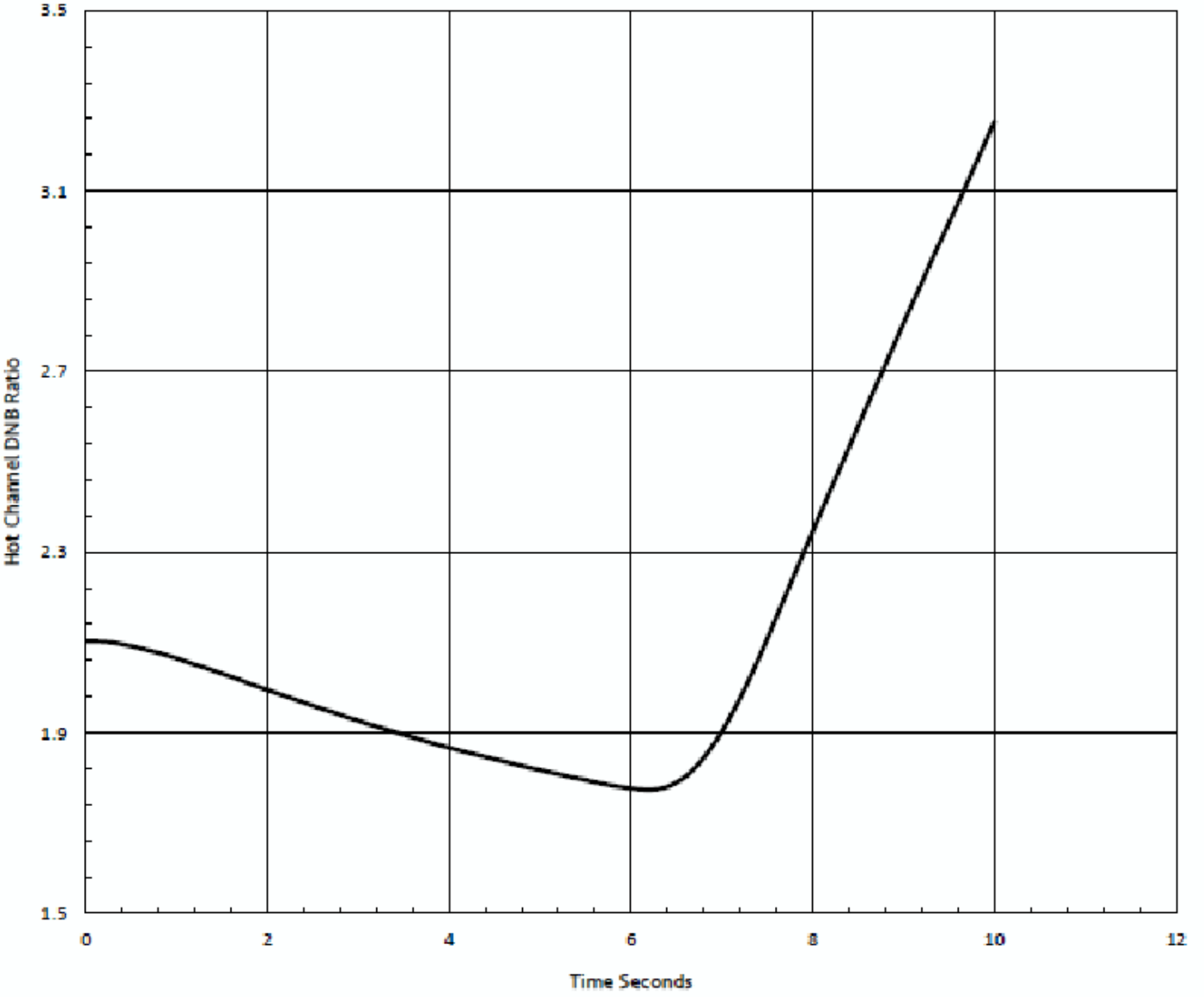
TMI Unit-1

Percent Reactor Coolant Flow as a Function of Time After Loss of Pump Power

Rev. 24 04/18

Fig. 14.1-17



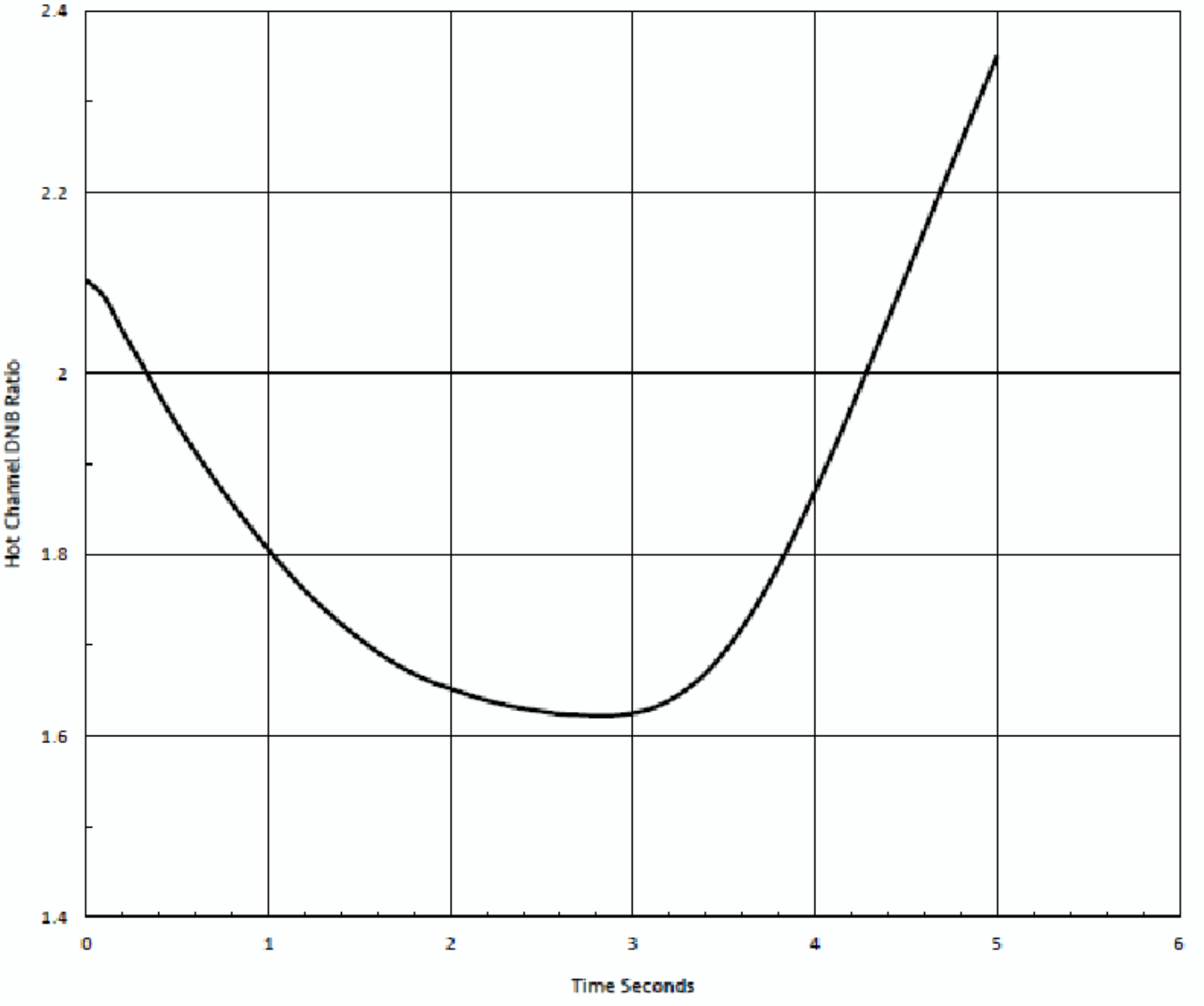
GPU Nuclear

TMI Unit-1

DNB Ratio Versus Time for a 4 Pump Coastdown Accident from 102% of Rated Power

Rev. 24 04/18

Fig. 14.1-18



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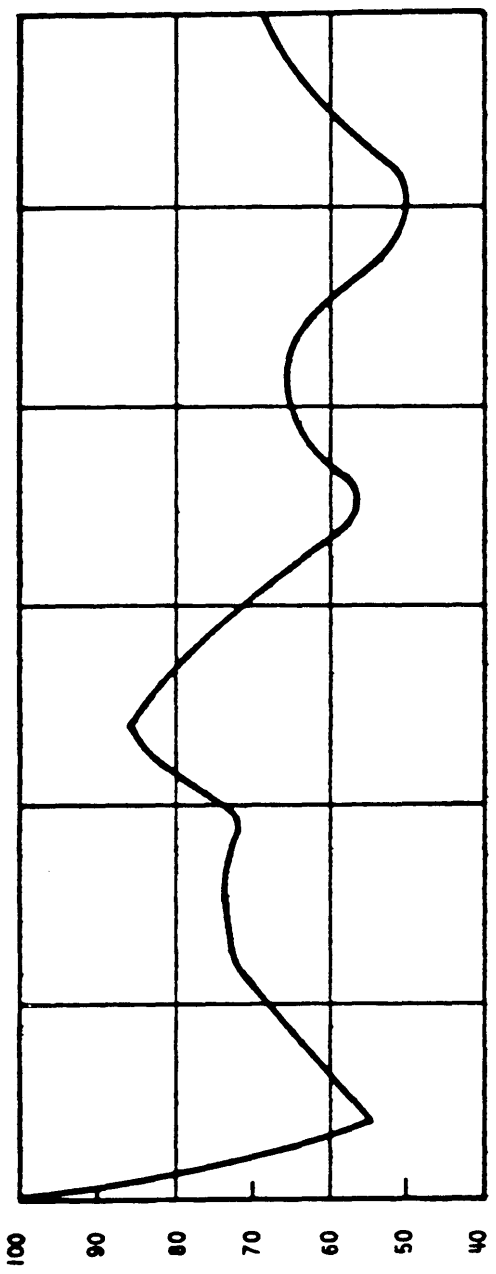
TMI Unit-1

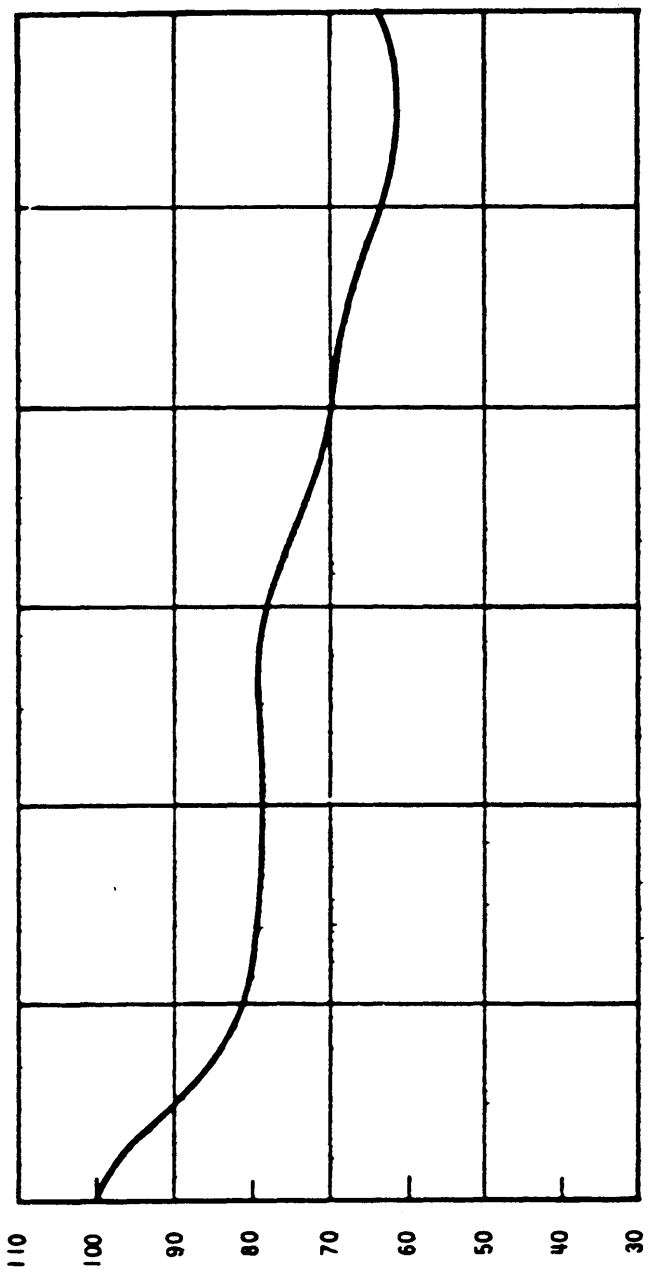
DNB Ratio Versus Time for a Locked Rotor Accident from 102% of Rated Power

Rev. 24 04/18

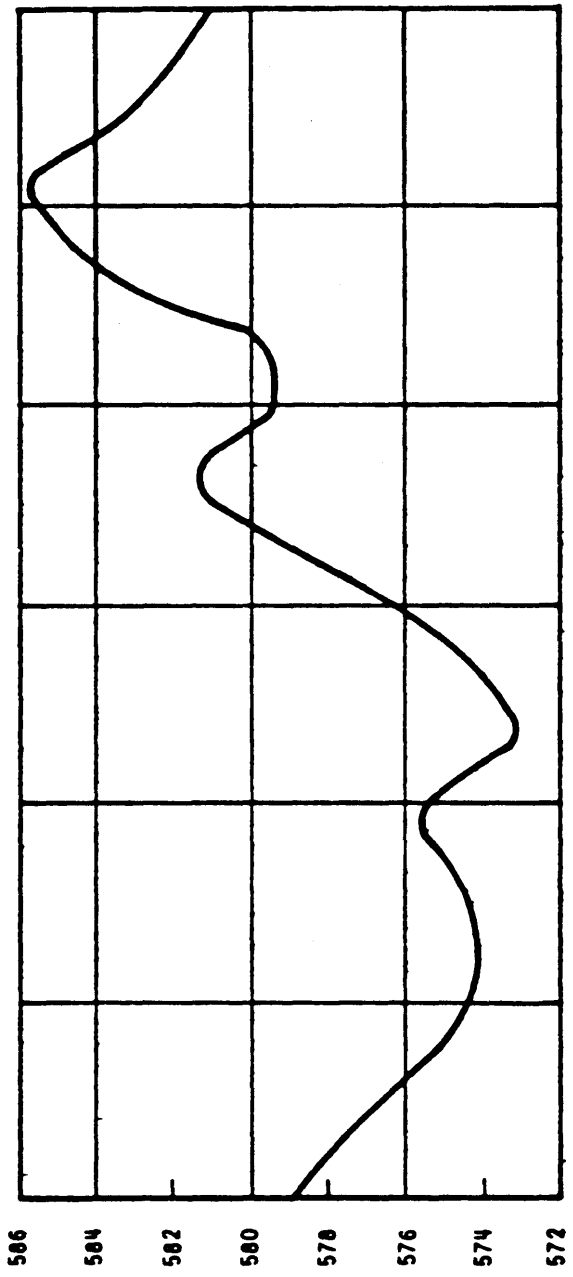
Fig. 14.1-19



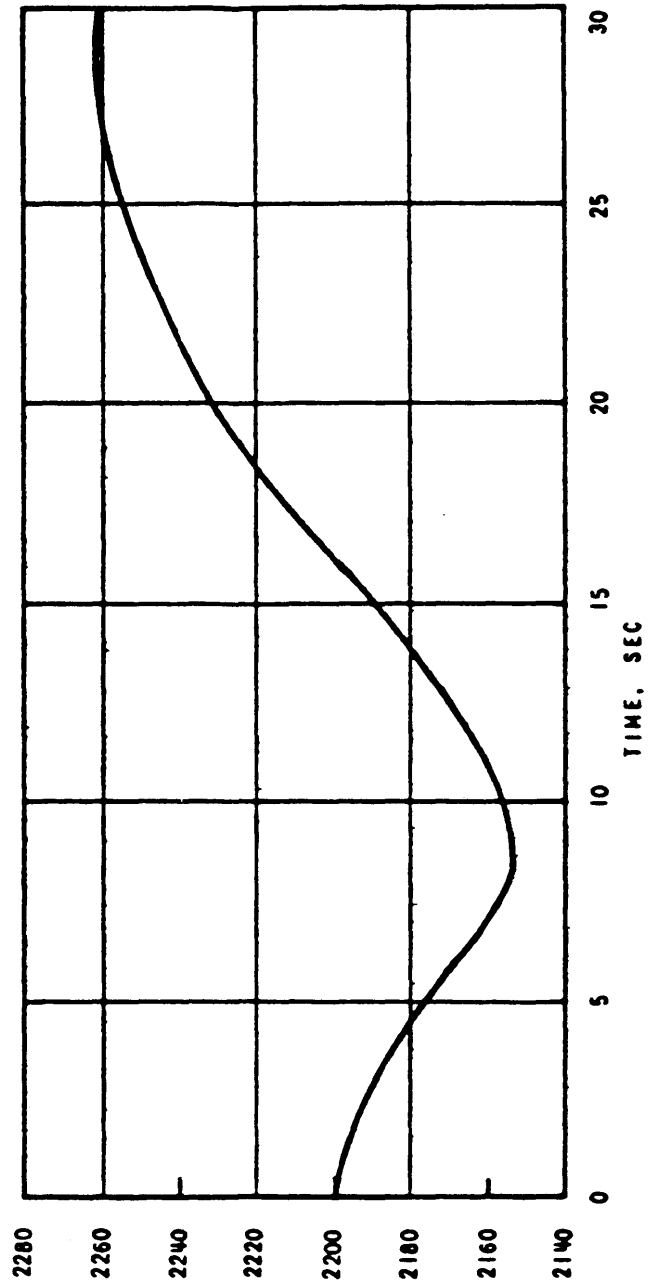
NEUTRON POWER %



THERMAL POWER %



AVERAGE CORE MODERATOR TEMPERATURE, F



REACTOR SYSTEM PRESSURE psia

GPU Nuclear

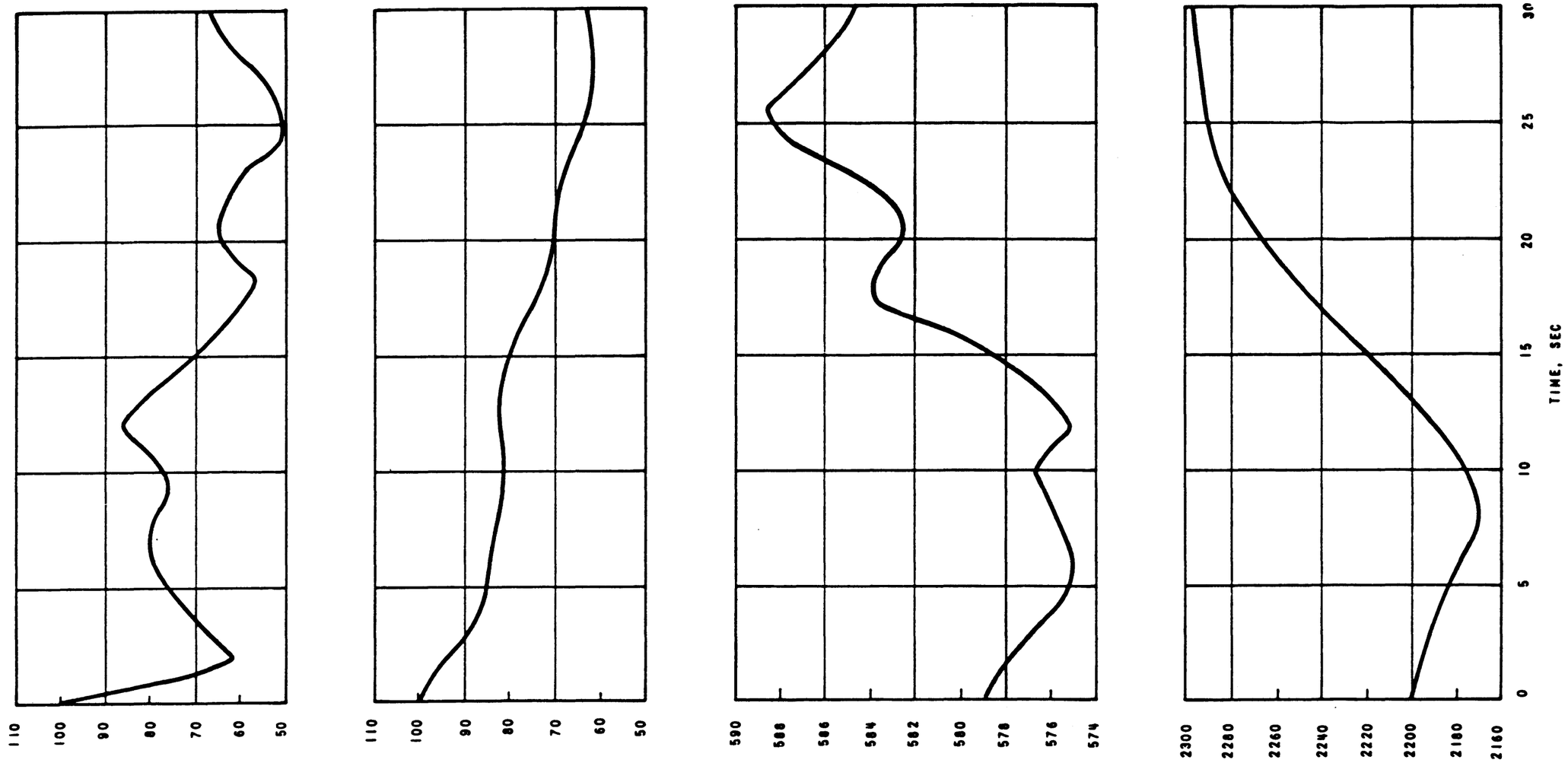
TMI Unit-1

Update -1

7/82

0.46% △K/K Rod Drop from Rated Power with Automatic Runback to 60% Demand in 12s

Fig. 14.1-20



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TMI Unit-1

0.36% △K/K Rod Drop from Rated Power with Automatic Runback to 60% Demand in 12s

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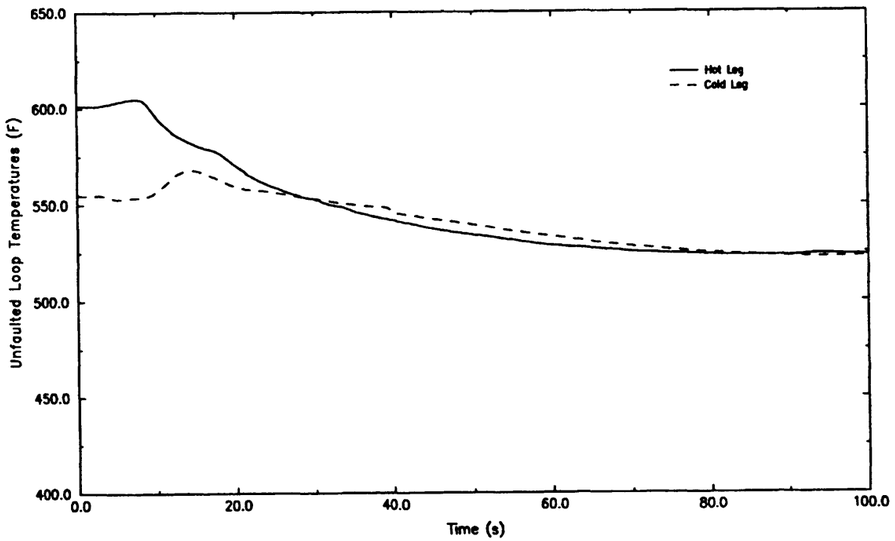
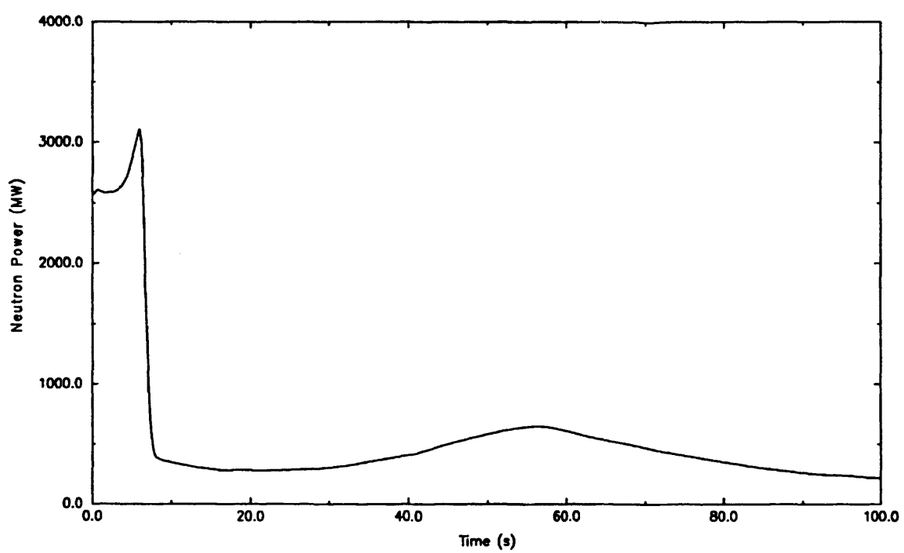
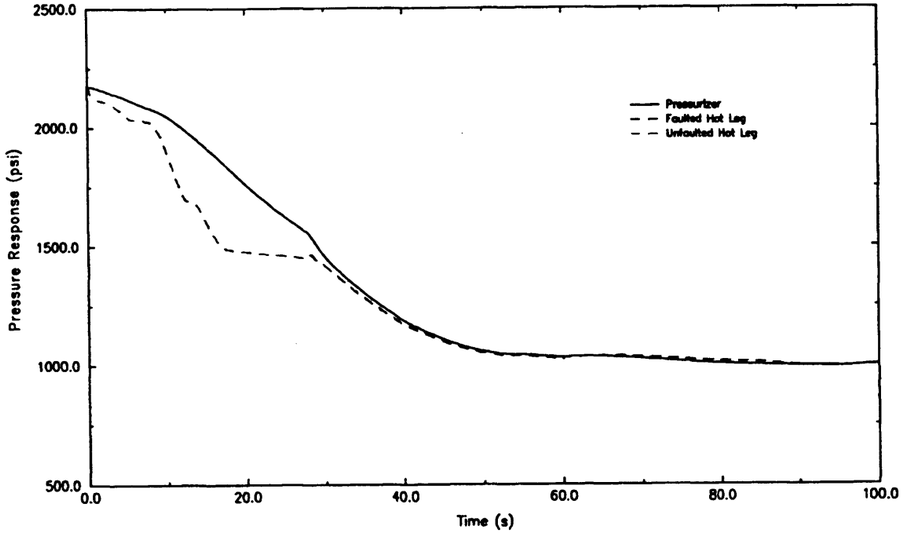
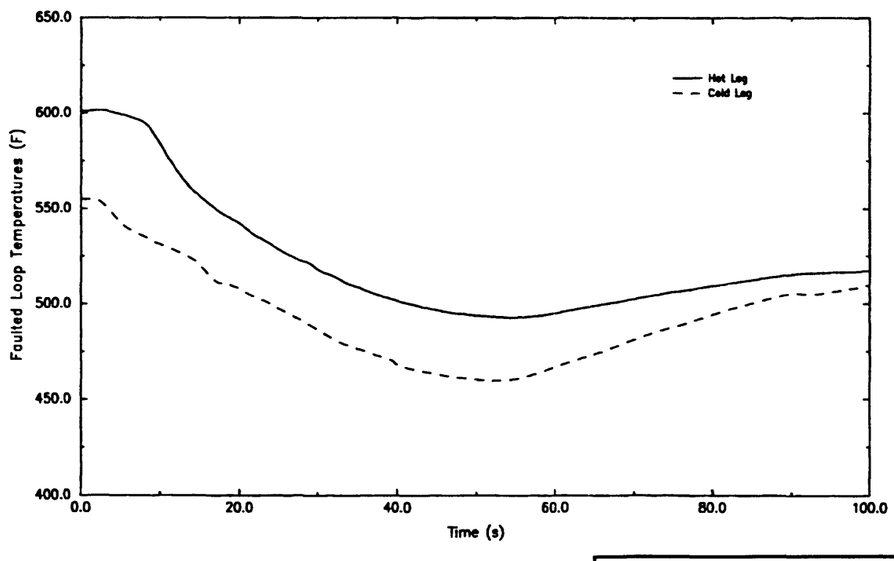
7/82

Fig. 14.1-21









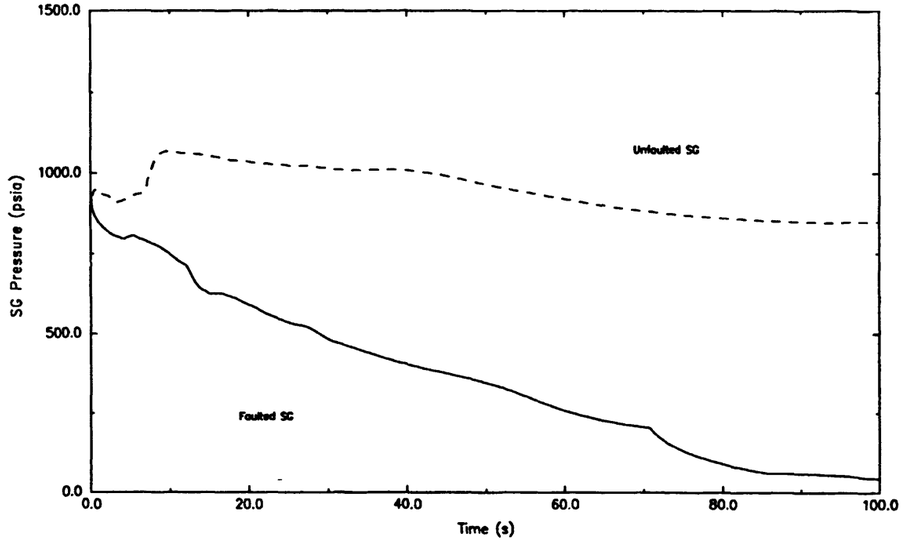
AmerGen
TMI – UNIT 1

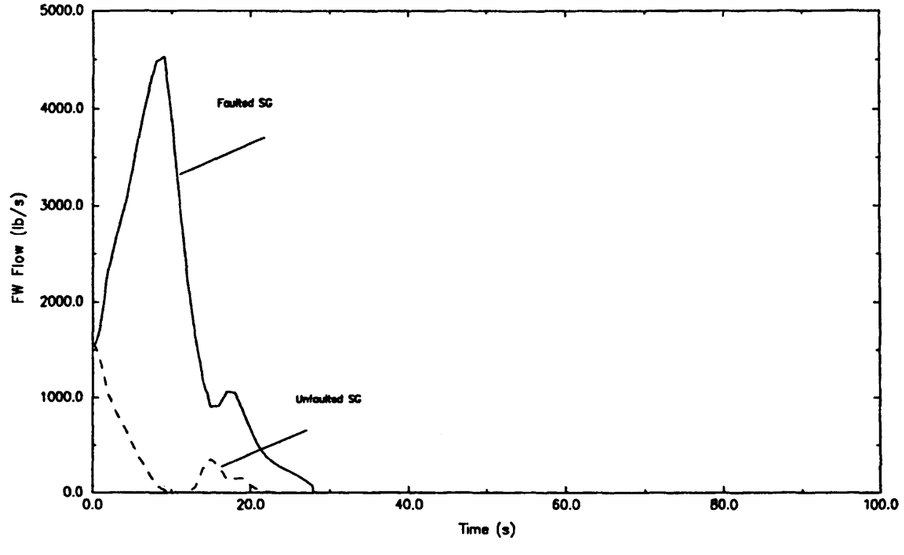
UPDATE – 15
4/00

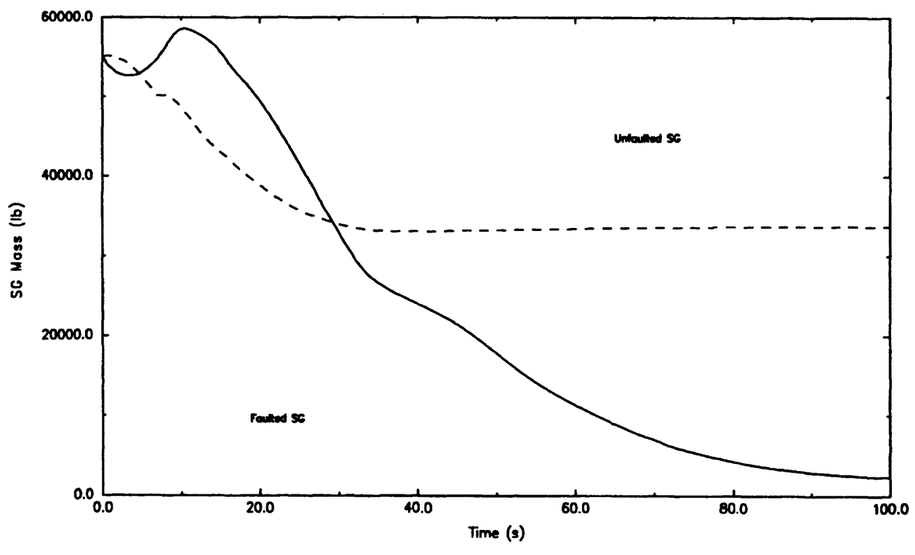
DOUBLE–ENDED RUPTURE OF 24 IN. STEAM LINE BETWEEN STEAM GENERATOR AND STEAM STOP VALVE (WITH FEED WATER ISOLATION)

CAD FILE: 6710R15.DWG

Fig. 14.1–22A







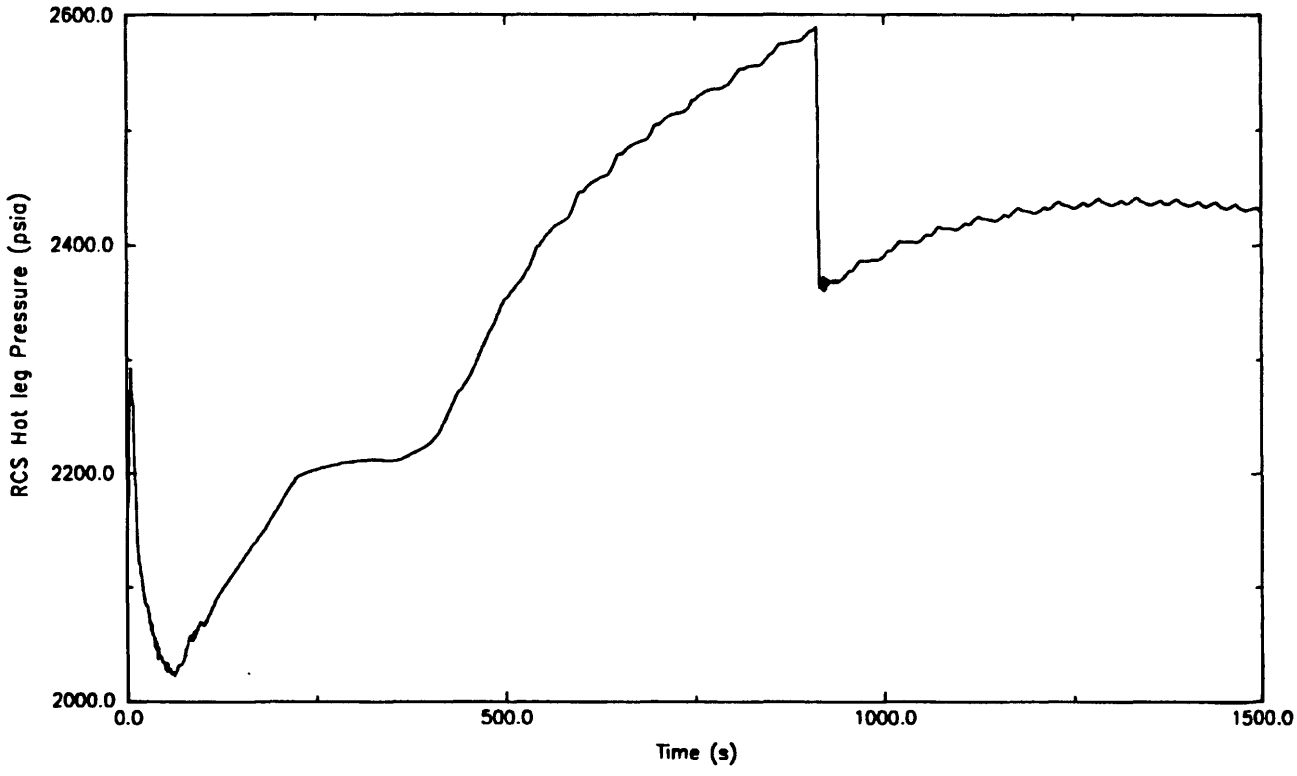
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

DOUBLE-ENDED RUPTURE OF 24 IN. STEAM LINE BETWEEN STEAM GENERATOR AND STEAM STOP VALVE (WITH FEED WATER ISOLATION)

CAD FILE: 6713R15.DWG

Fig. 14.1-22B



AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

RCS HOT LEG PRESSURE (psia) RESPONSE FOR LOSS OF ALL AC POWER (STATION BLACKOUT)

CAD FILE: 6705R15.DWG

Fig. 14.1–23

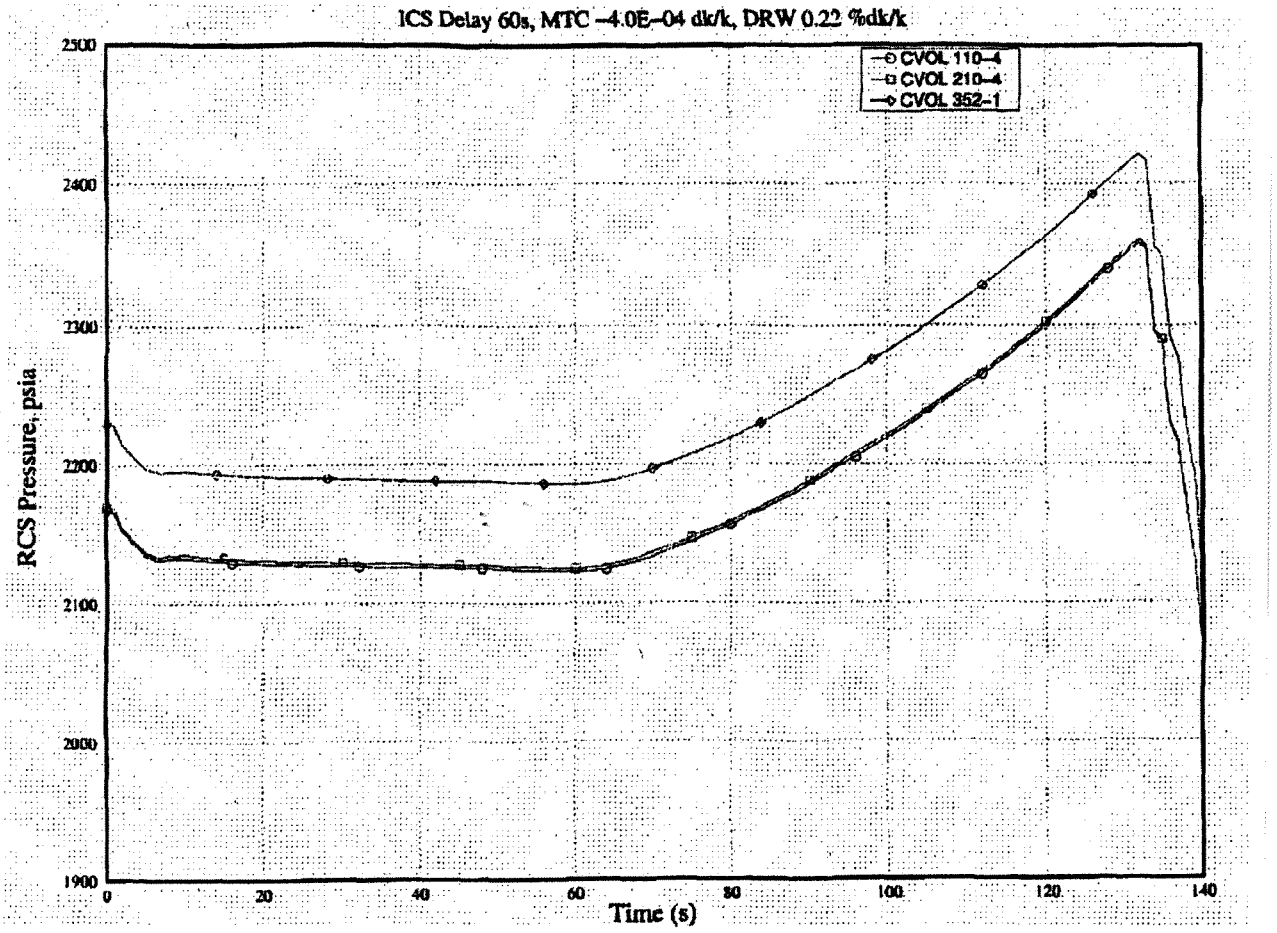


Figure 14.1-24 Dropped Rod w/ Bayonet Failure – RCS Pressure Response (NOTE 1)

NOTES:

1. CVOL 110-4 and 210-4 represent hot leg pressures.
   CVOL 352-1 represents the core exit pressure.

TMI - UNIT 1 REV. 19, APRIL 2008

Dropped Rod w/ Bayonet Failure
RCS Pressure Response

FIGURE. 14.1-24

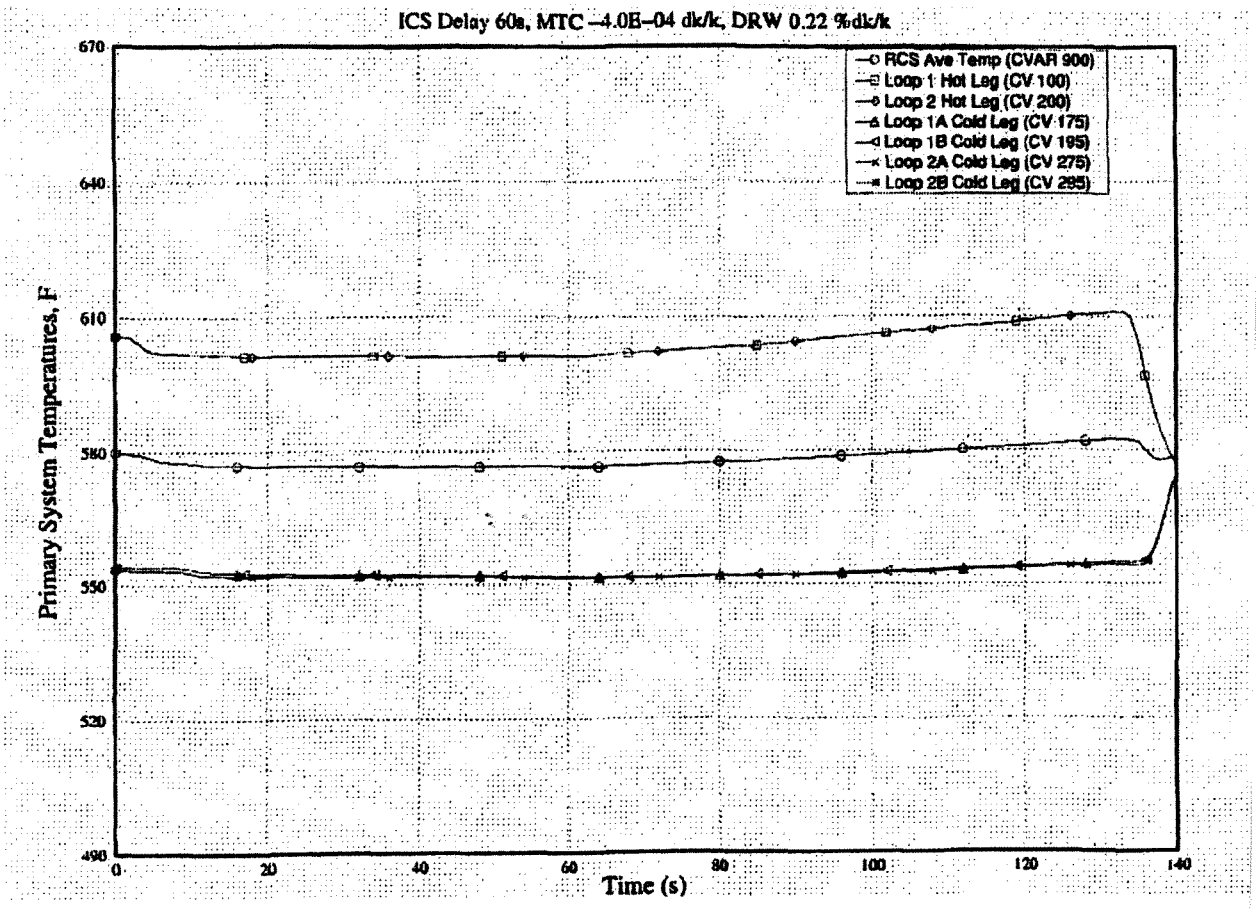


Figure 14.1-25 Dropped Rod w/ Bayonet Failure – Reactor Power Response

p.14.FIG-23B

TMI - UNIT 1	REV. 19, APRIL 2008
Dropped Rod w/ Bayonet Failure Reactor Power Response	
	FIGURE. 14.1-25

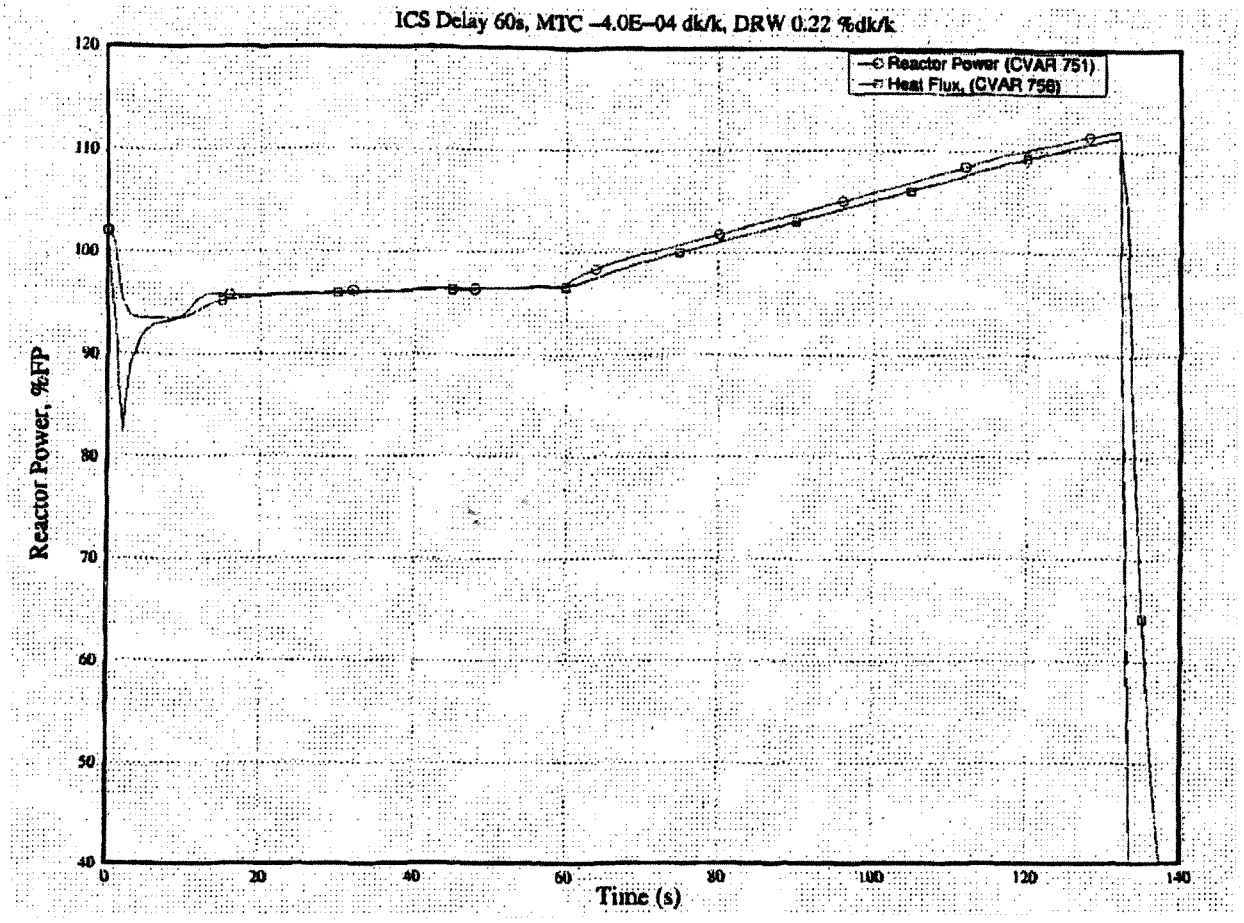
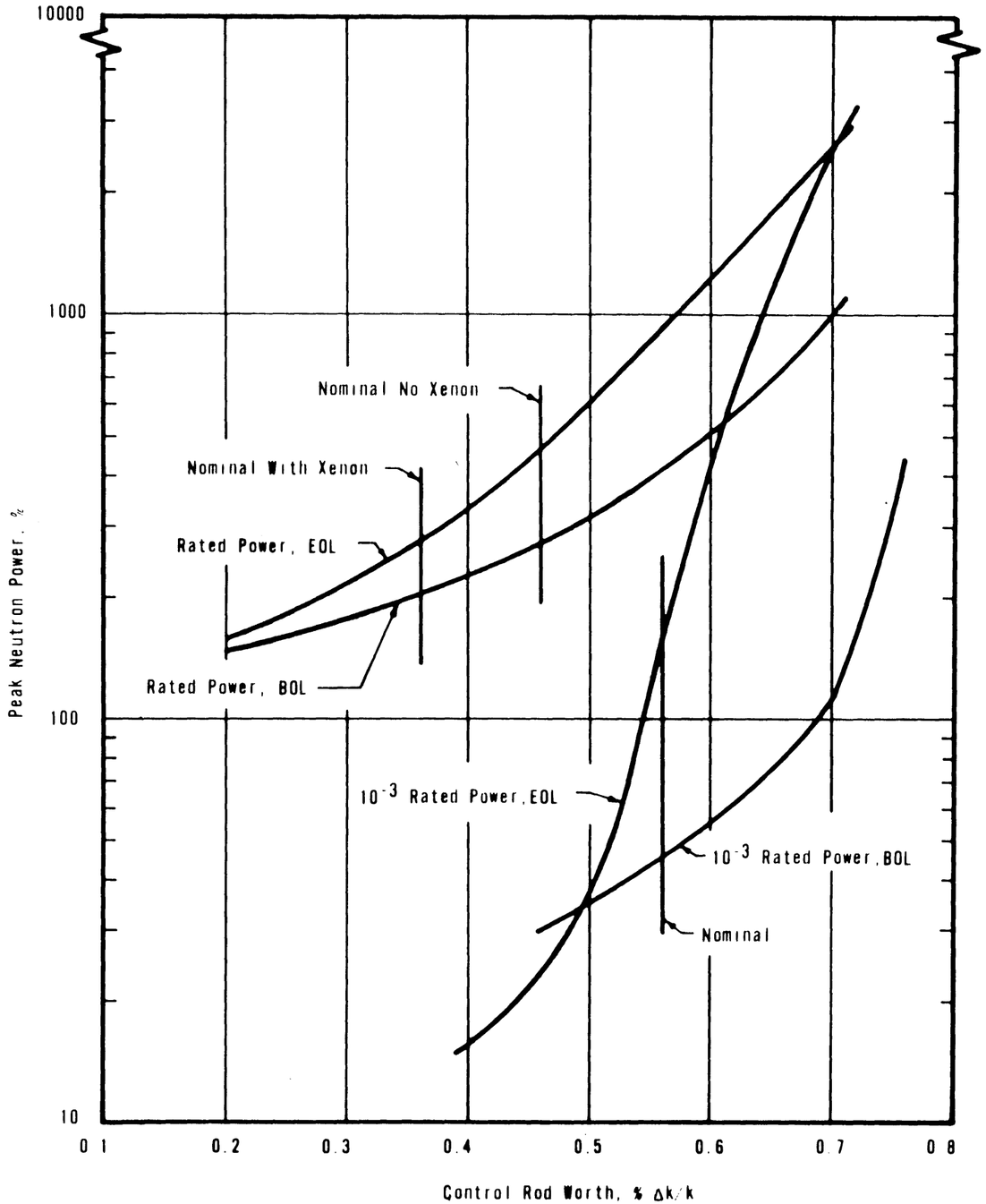


Figure 14.1-26 Dropped Rod w/ Bayonet Failure – Primary Temperature Response

TMI - UNIT 1	REV. 19, APRIL 2008
p.14.FIG-23C	Dropped Rod w/ Bayonet Failure Primary Temperature Response
	FIGURE. 14.1-26



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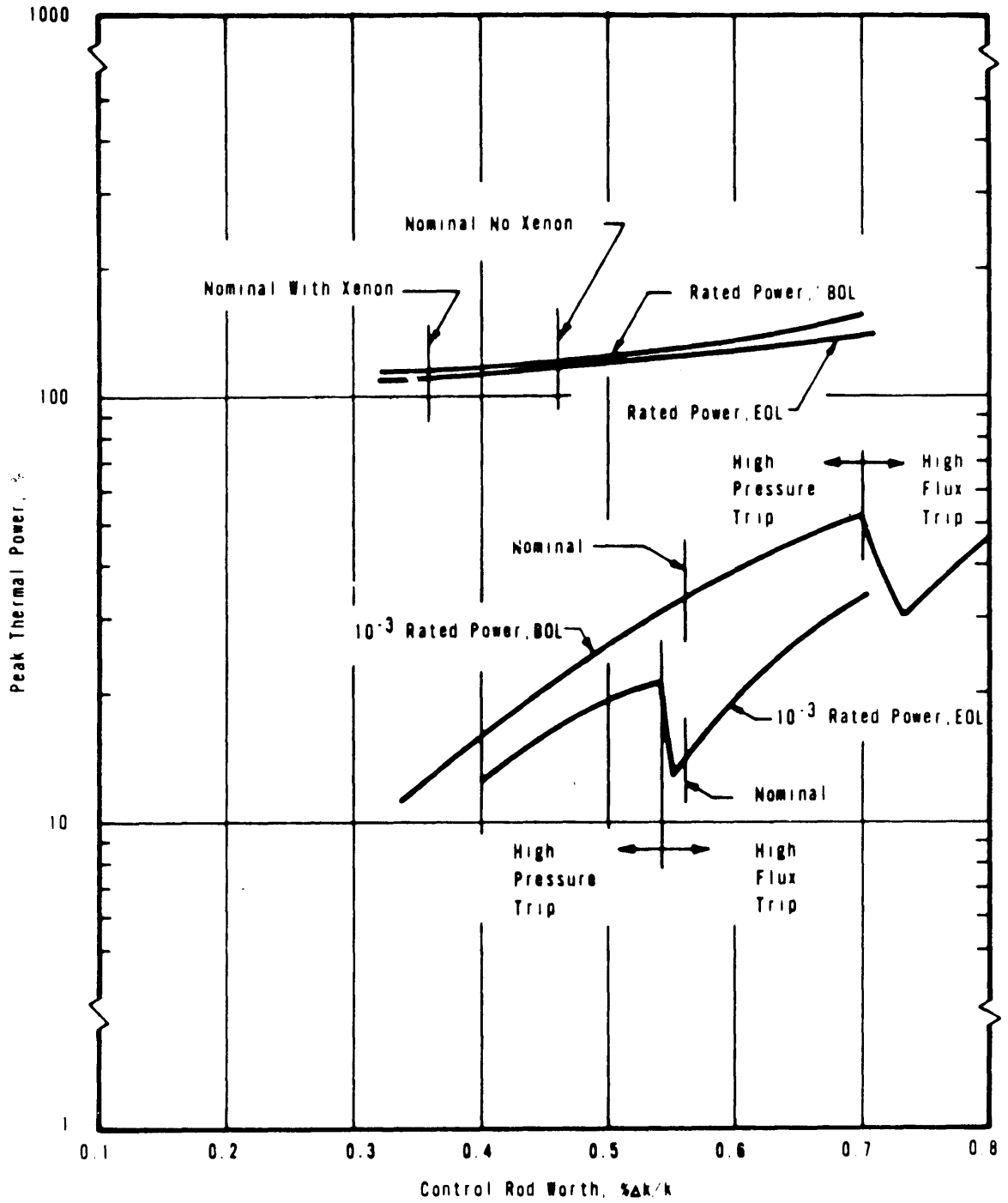
7/82

Peak Neutron Power Variation With
Ejected Control Rod Worth

Fig. 14.2-1



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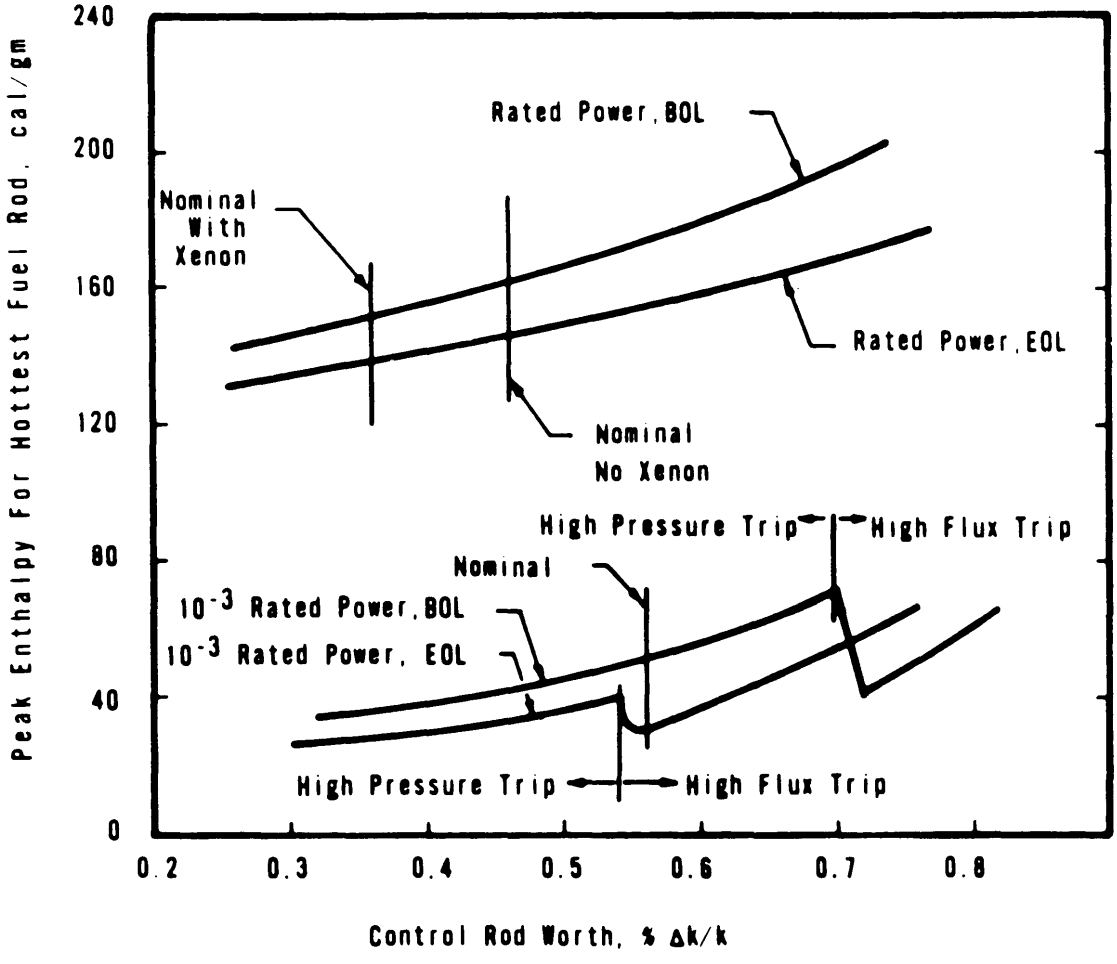
Peak Thermal Power as a Function of Ejected Control Rod Worth

Fig. 14.2-2



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TMI Unit-1

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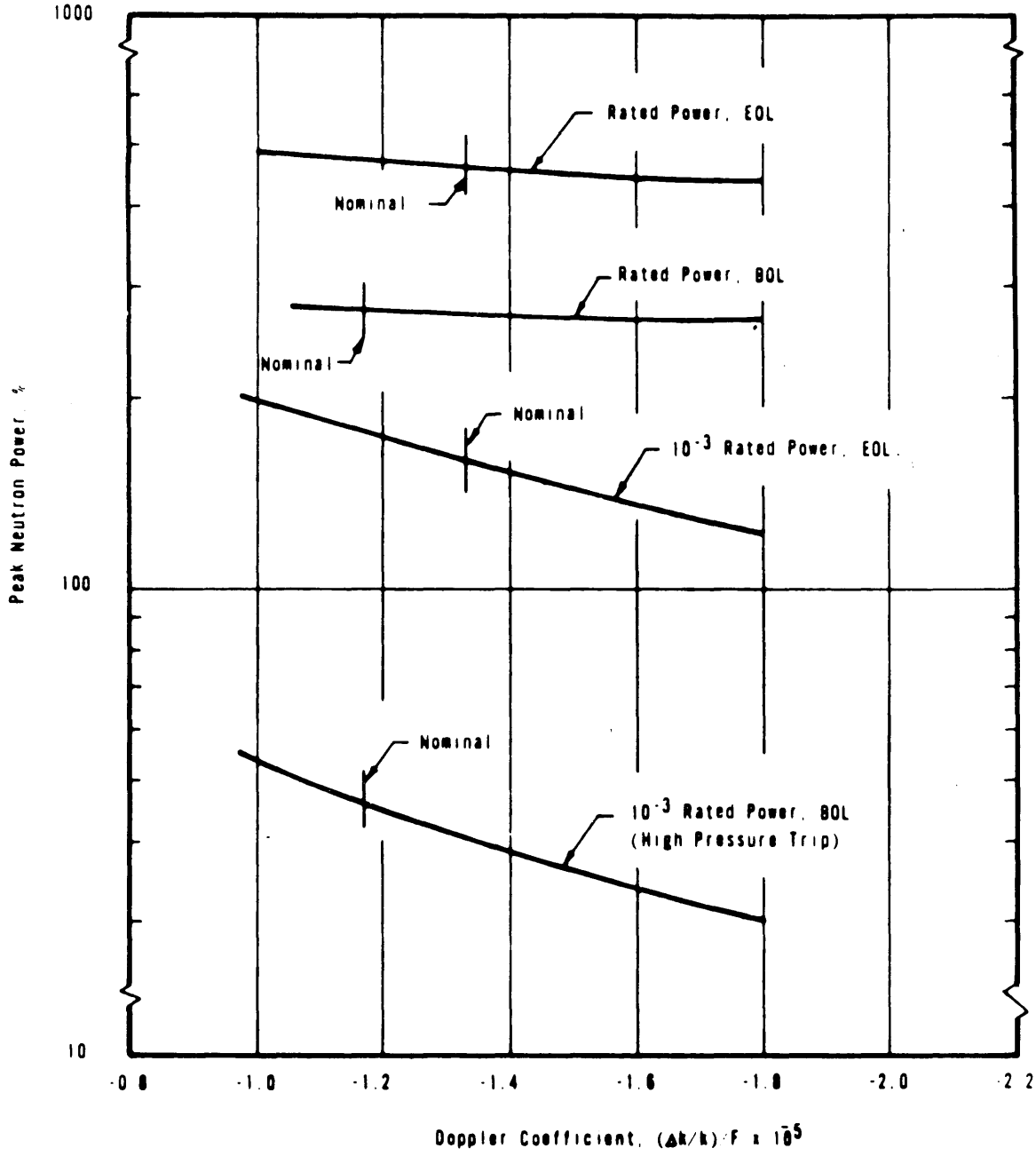
7/82

Peak Enthalpy of Hottest Fuel Rod
Versus Ejected Control Rod Worth

Fig. 14.2-3



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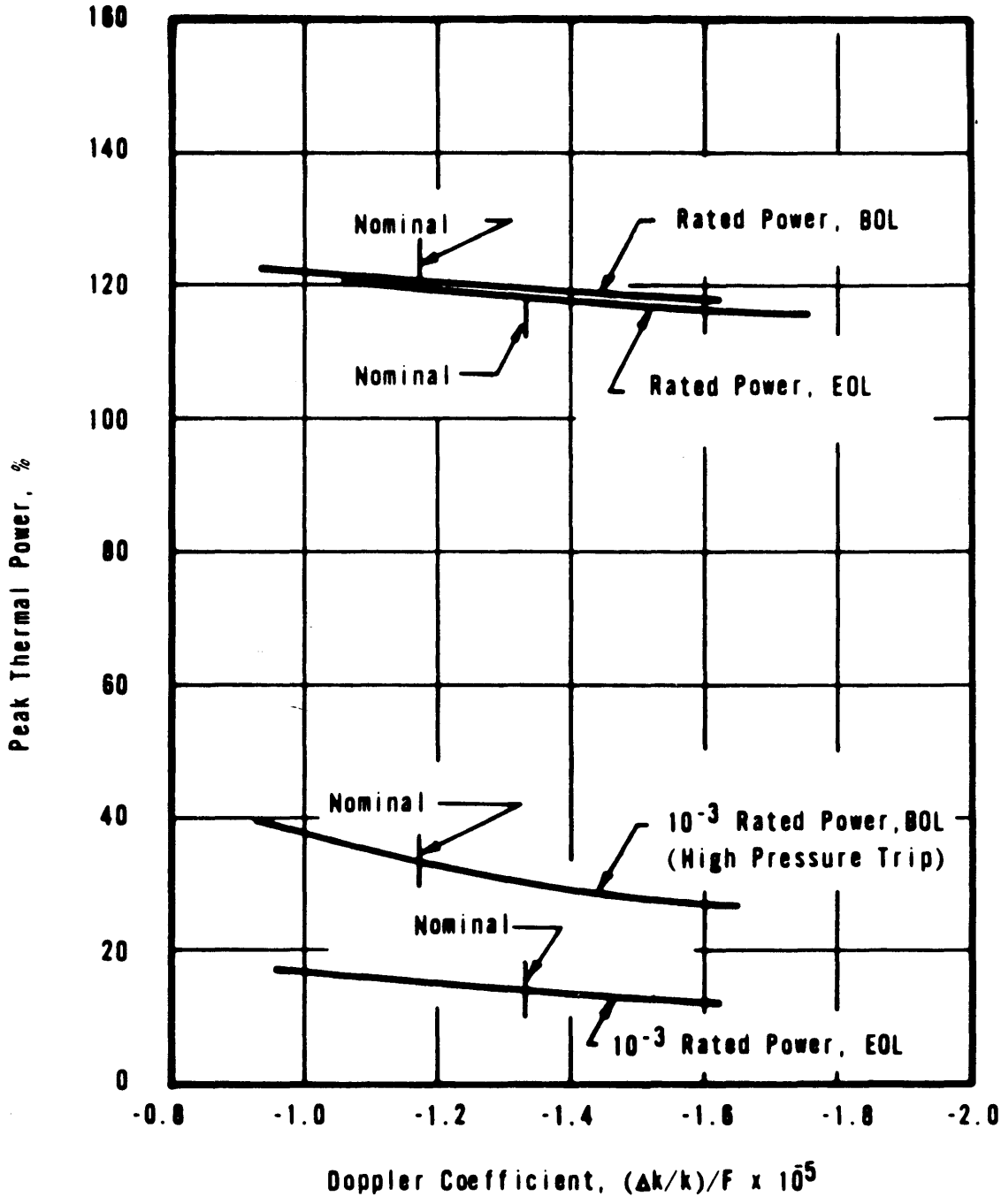
TMI Unit-1

7/82

Effect on Peak Neutron Power of Varying the Doppler Coefficient for an Ejected Rod Worth of 0.56%ΔK/K at 10^{-3} Rated Power and 0.46%ΔK/K at Rated Power

Fig. 14.2-4



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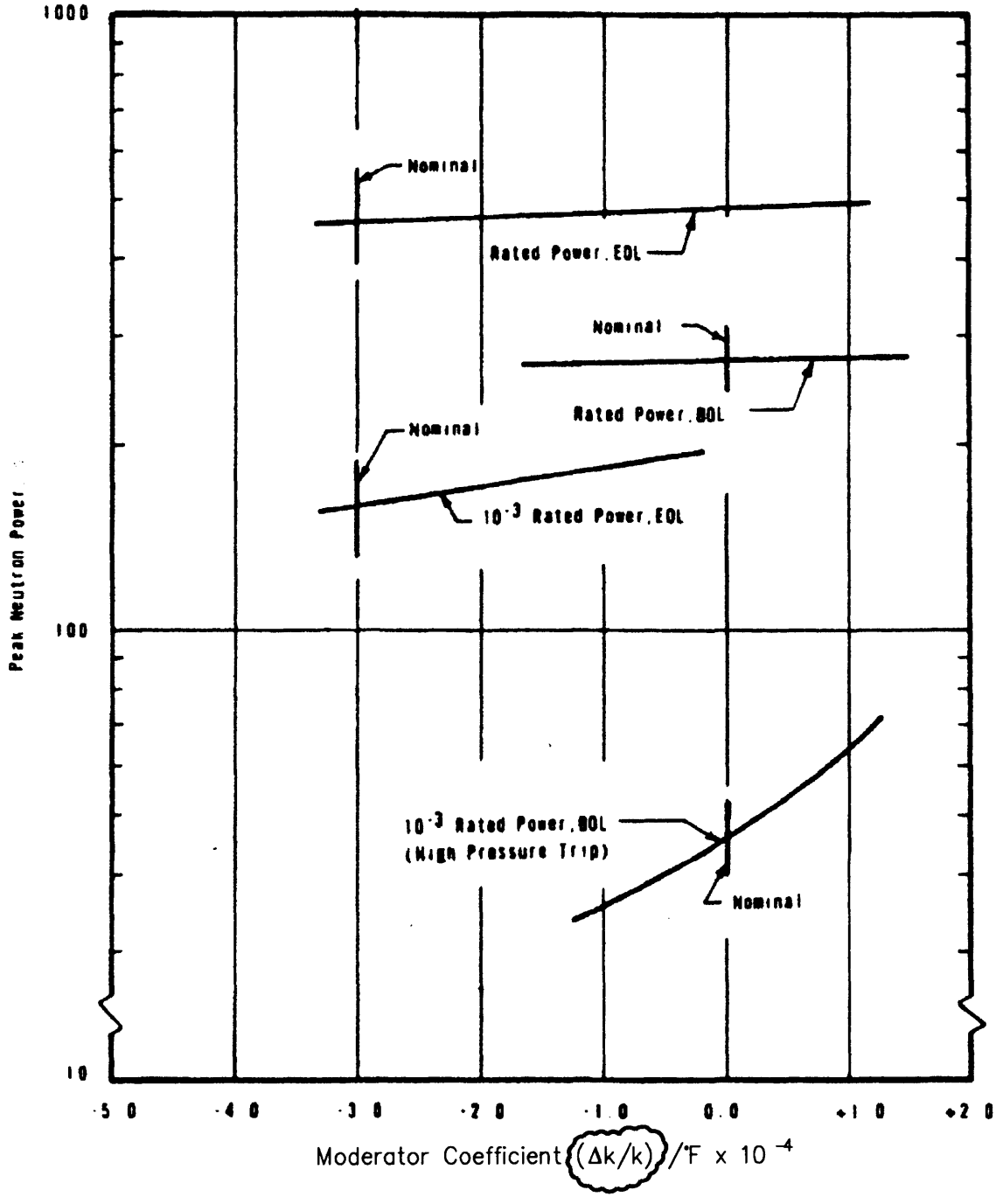
TMI Unit-1

7/82

Effect on Peak Thermal Power of Varying the Doppler Coefficient for an Ejected Rod Worth of 0.56% ΔK/K at 10^{-3} Rated Power and 0.46% ΔK/K at Rated Power

Fig. 14.2-5



TMI - UNIT 1

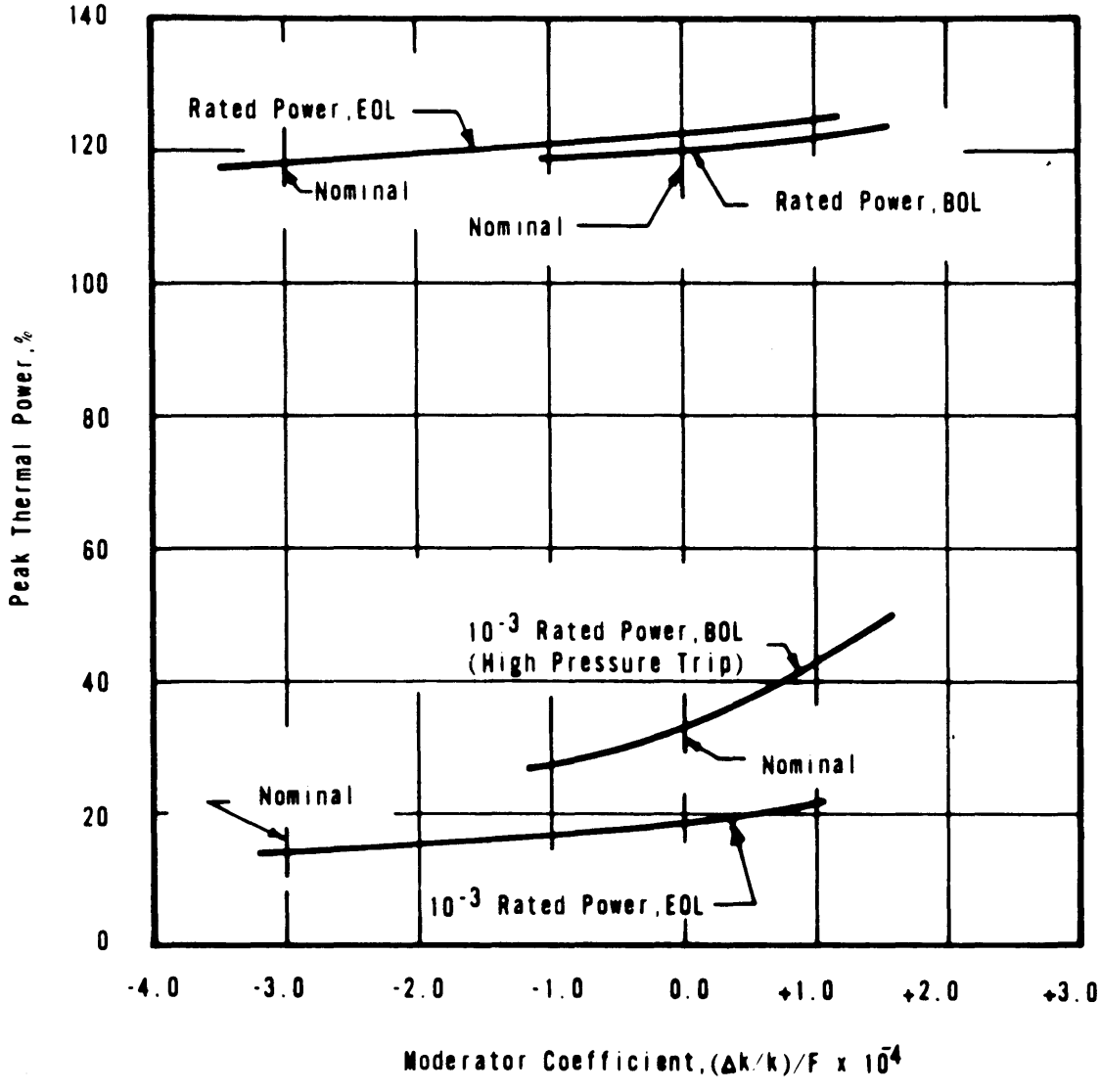
UPDATE - 14
04/98

GPU NUCLEAR

Effect on Peak Neutron Power of Varying the Moderator Coefficient for an Ejected Rod Worth of 0.56% Δk/k at 10^{-3} Rated Power and 0.46% Δk/k at Rated Power

CAD FILE: 6558RI4.DWG

FIG 14.2-6



GPU Nuclear

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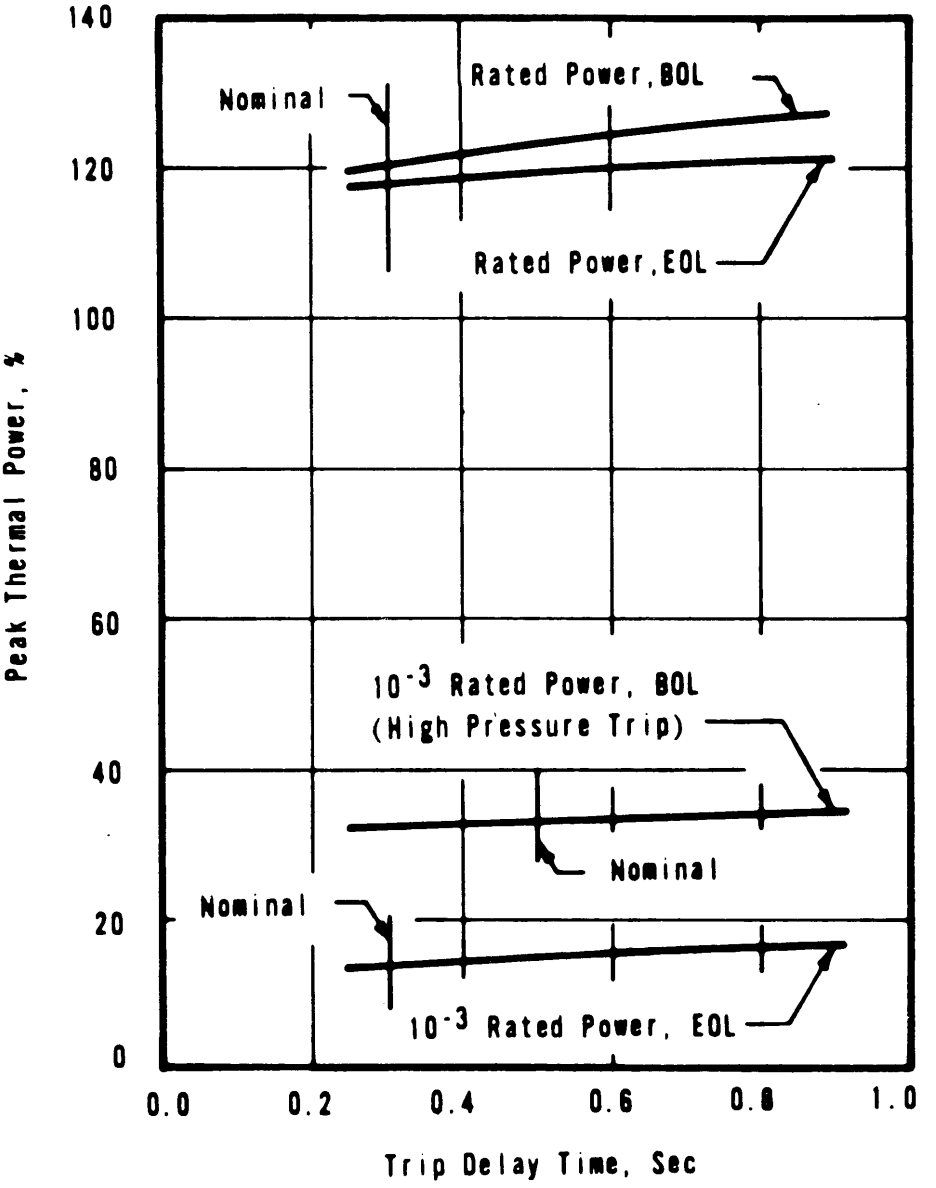
7/82

TMI Unit-1

Effect on Peak Thermal Power of Varying the Moderator Coefficient for an Ejected Rod Worth of 0.56% △K/K at 10^{-3} Rated Power and 0.46% △K/K at Rated Power

Fig. 14.2-7



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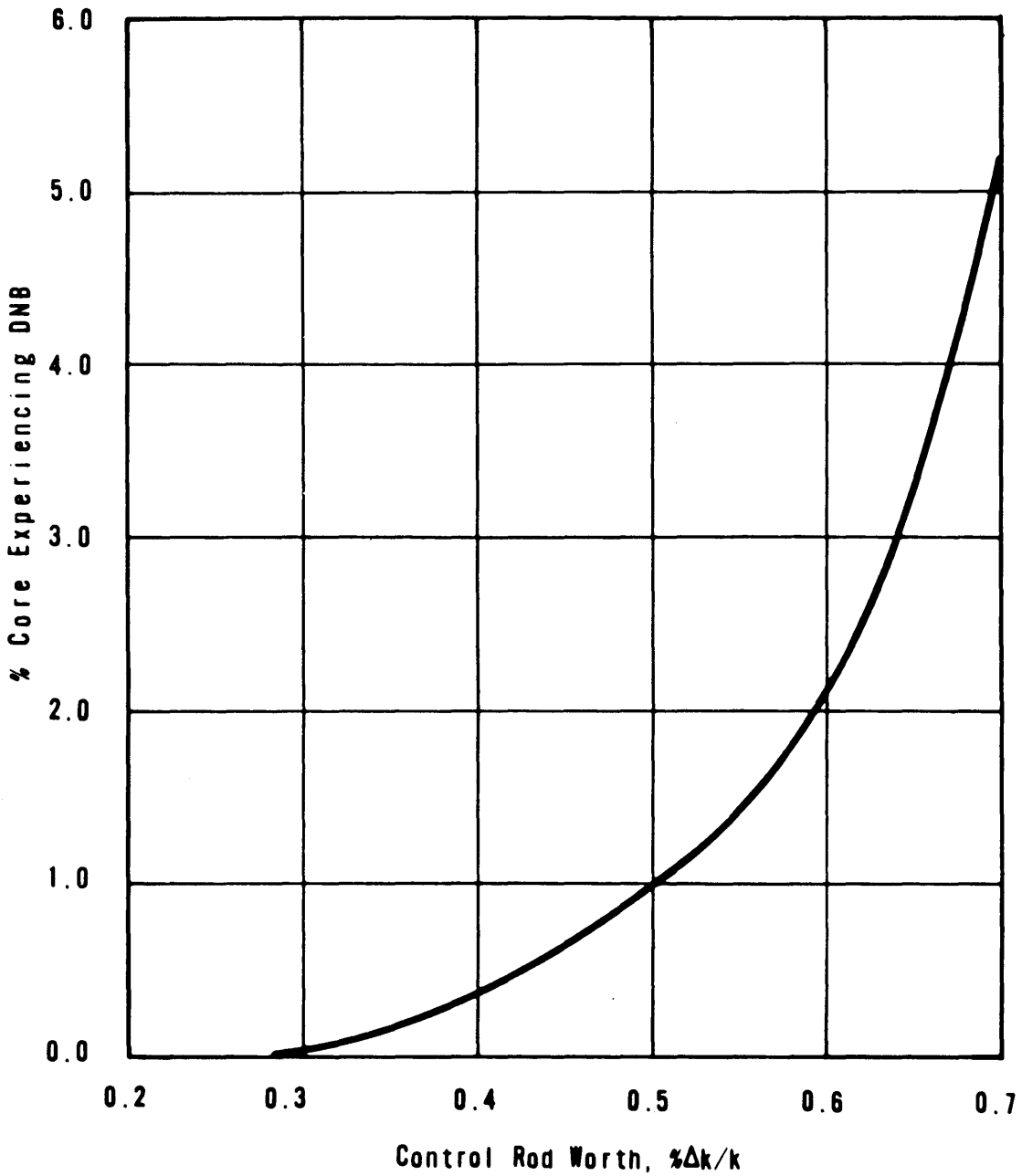
TMI Unit-1 | 7/82

Effect on Peak Thermal Power of Varying the Trip Delay Time for an Ejected Rod Worth of 0.56% $\Delta K/K$ at 10^{-3} Rated Power and 0.46% $\Delta K/K$ at Rated Power

Fig. 14.2-8



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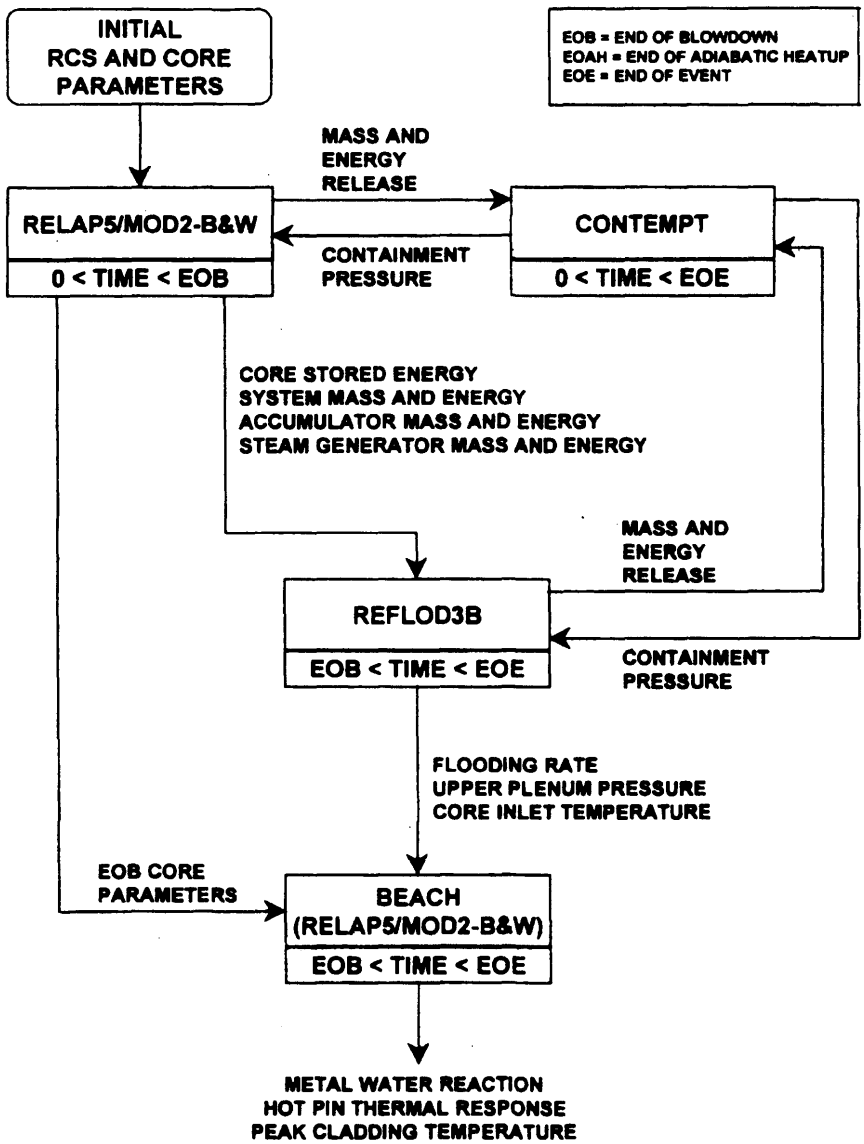
TMI Unit-1

Update -1

7/82

Percent Core Experiencing DNB as a Function of Ejected Control Rod Worth at Rated Power, BOL

Fig. 14.2-9

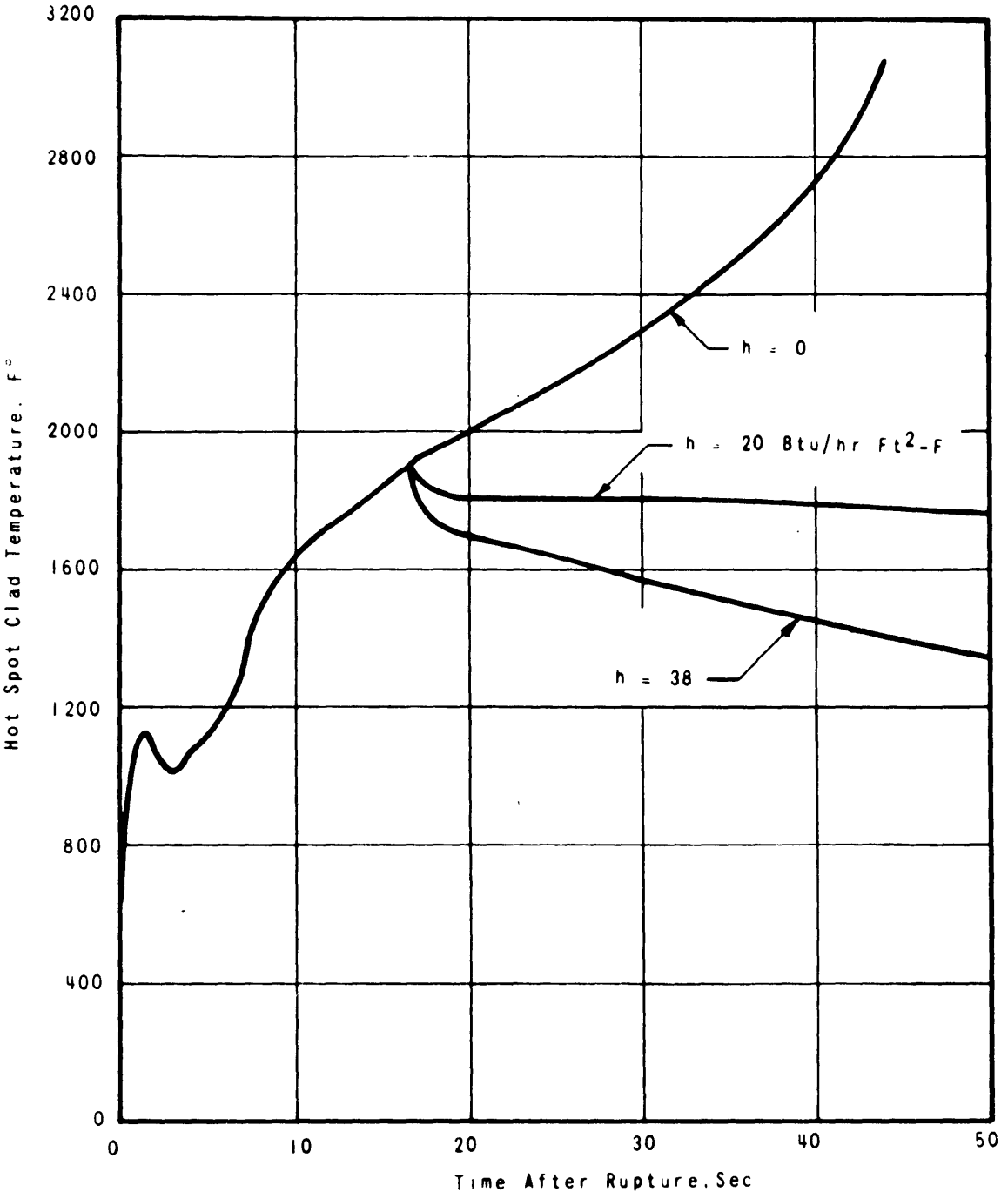


TMI - UNIT 1 UPDATE - 14
4/98

GPU NUCLEAR

LARGE BREAK ANALYSIS CODE INTERFACES

CAD FILE: 6573RI4.DWG FIG 14.2-10



GPU Nuclear

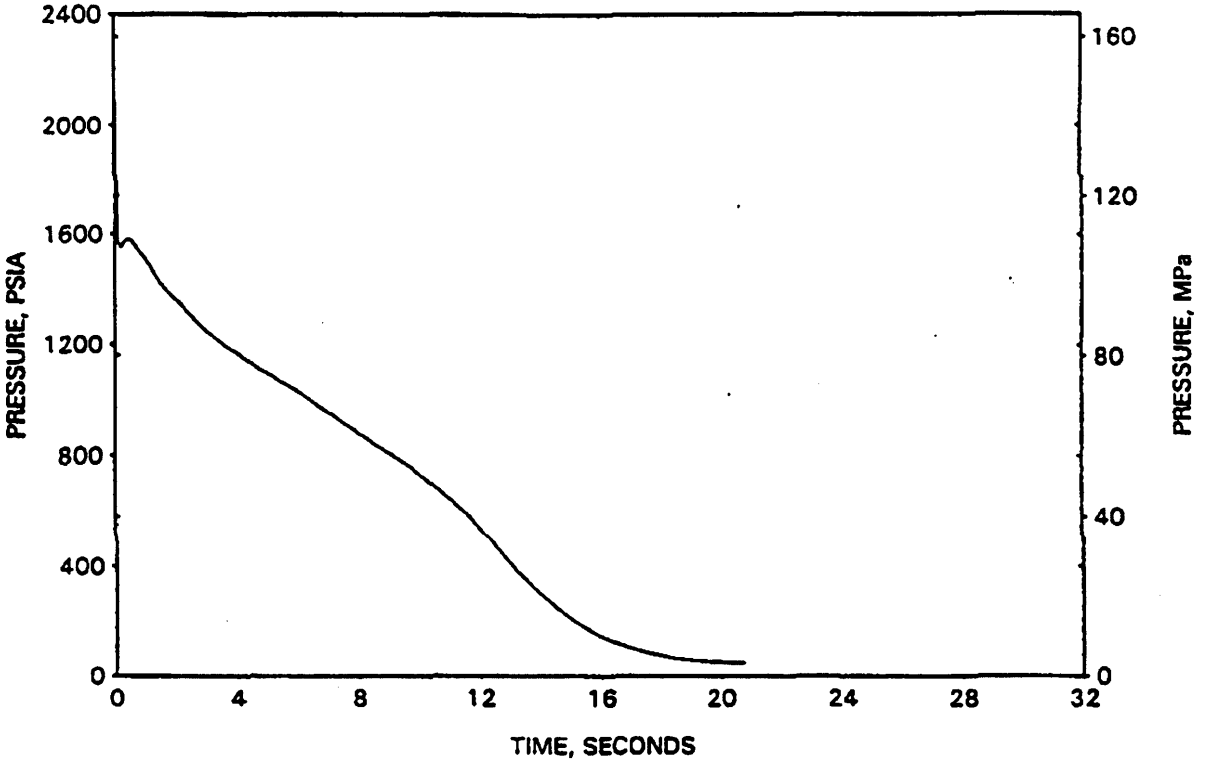
TMI Unit-1

Hot Spot Clad Temperature vs. Time for a 36 in. ID, Double-Ended, Hot Leg Pipe Rupture and Variable Quench Coefficient

Update -1

7/82

Fig. 14.2-11



AmerGen
TMI – UNIT 1

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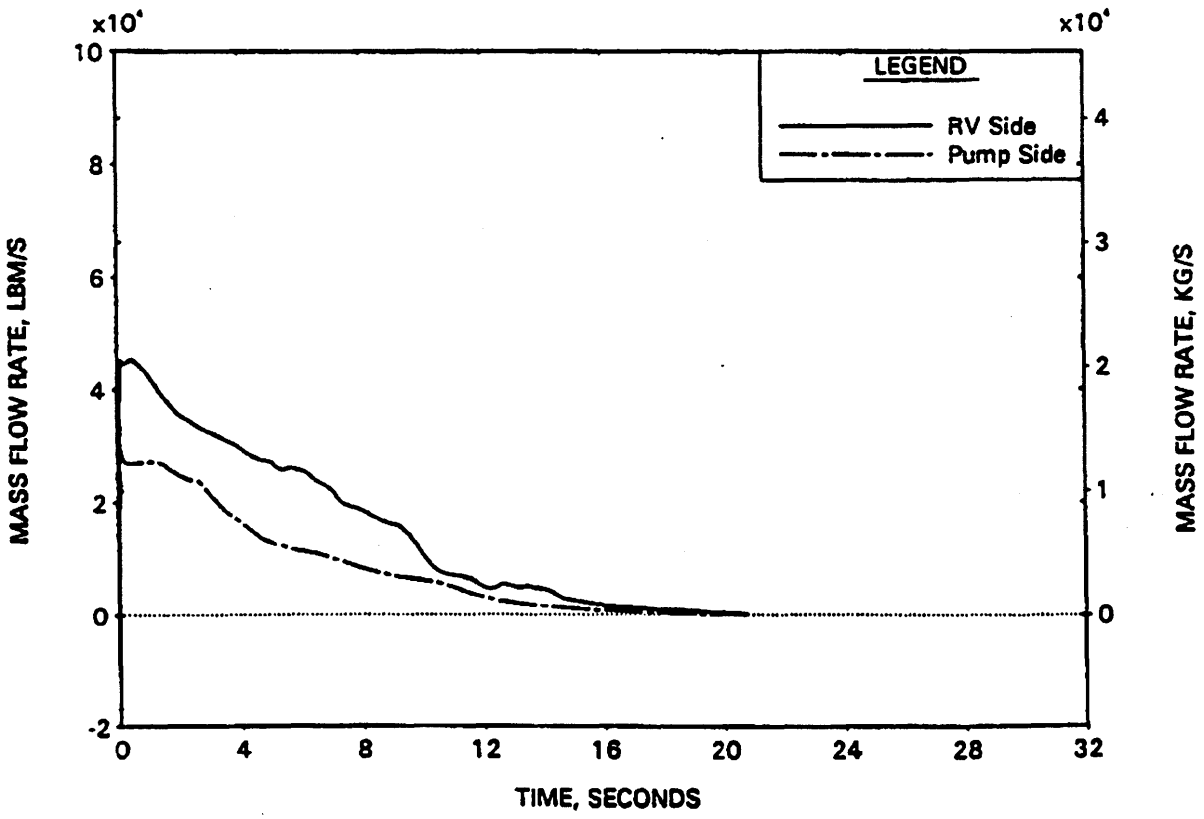
4/00

LBLOCA LIMIT CASE (BOL) – REACTOR VESSEL UPPER PLENUM PRESSURE

CAD FILE: 6574R15.DWG

FIG 14.2–12



AmerGen
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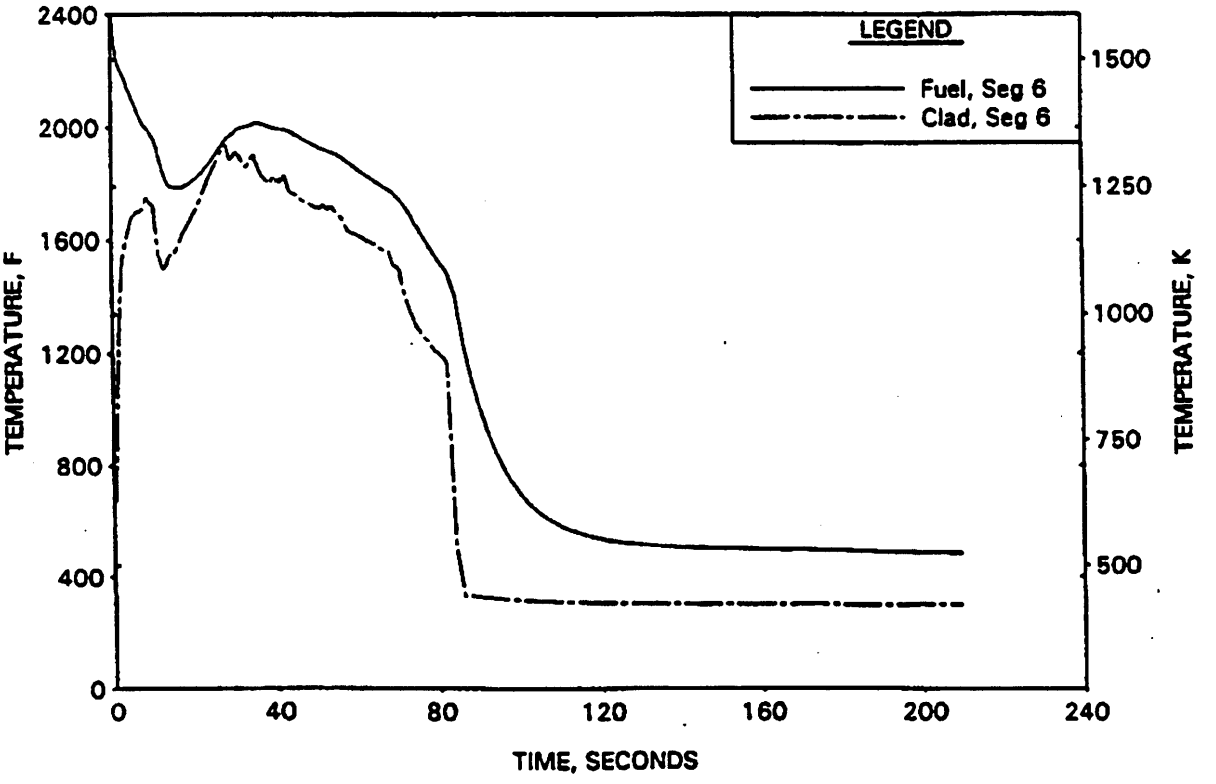
UPDATE – 15
4/00

LBLOCA LIMIT CASE (BOL) – BREAK MASS FLOW RATE

CAD FILE: 6575R15.DWG

FIG 14.2–13



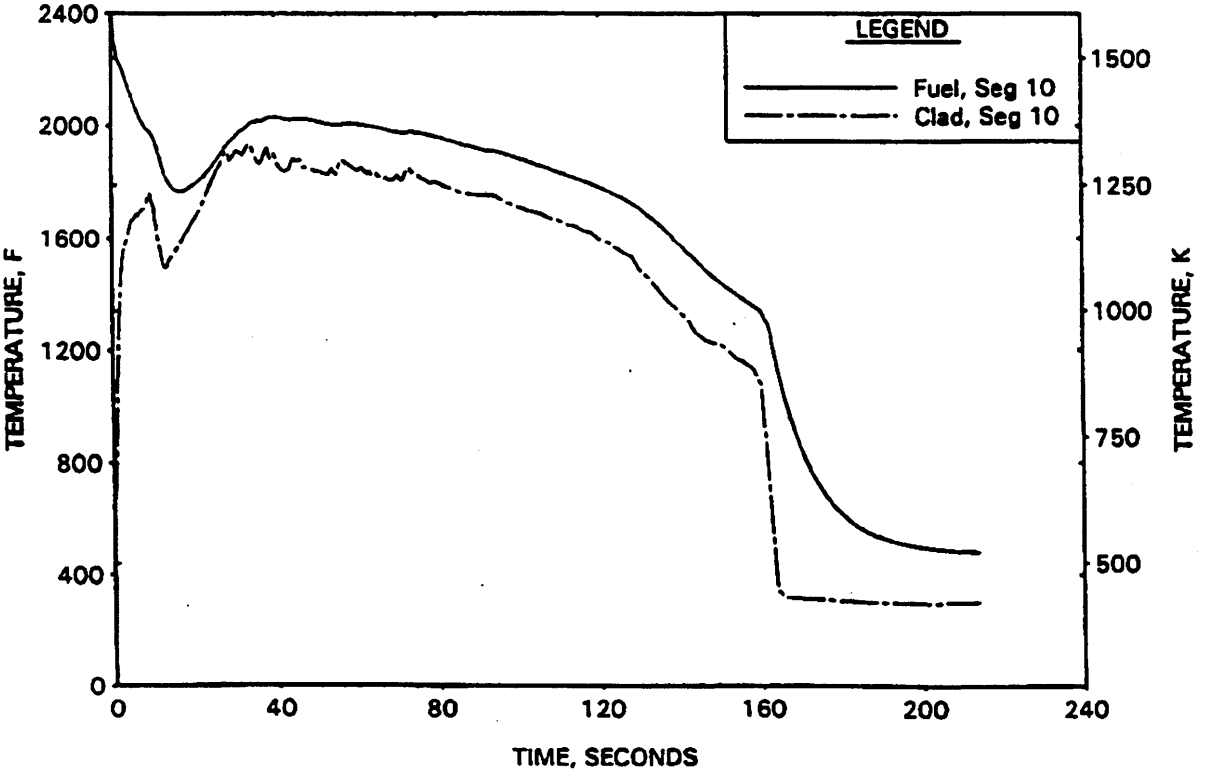
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

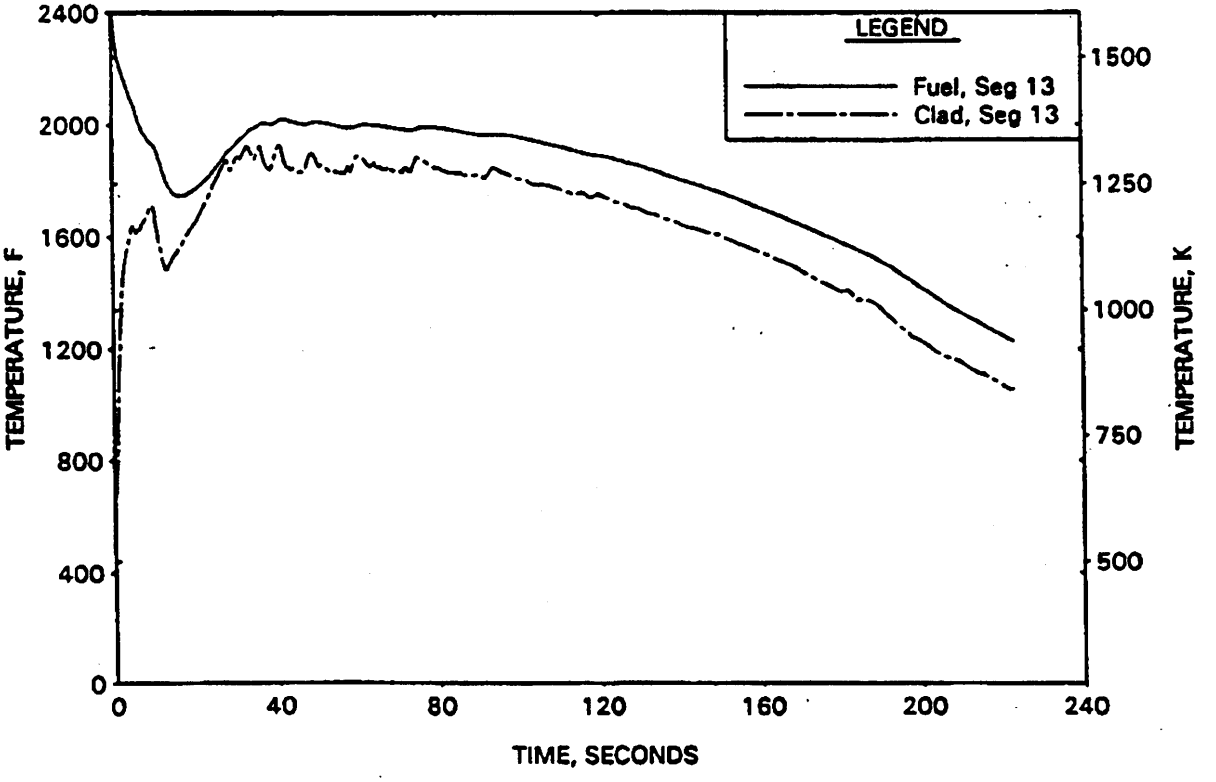
LBLOCA LIMIT CASE (BOL – 2.506–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK UNRUPTURED LOCATION

CAD FILE: 6576R15.DWG

FIG 14.2–14



AmerGen	UPDATE – 15
TMI – UNIT 1	4/00
LBLOCA LIMIT CASE (BOL – 4.264–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK UNRUPTURED LOCATION	
CAD FILE: 6577R15.DWG	FIG 14.2–15



AmerGen
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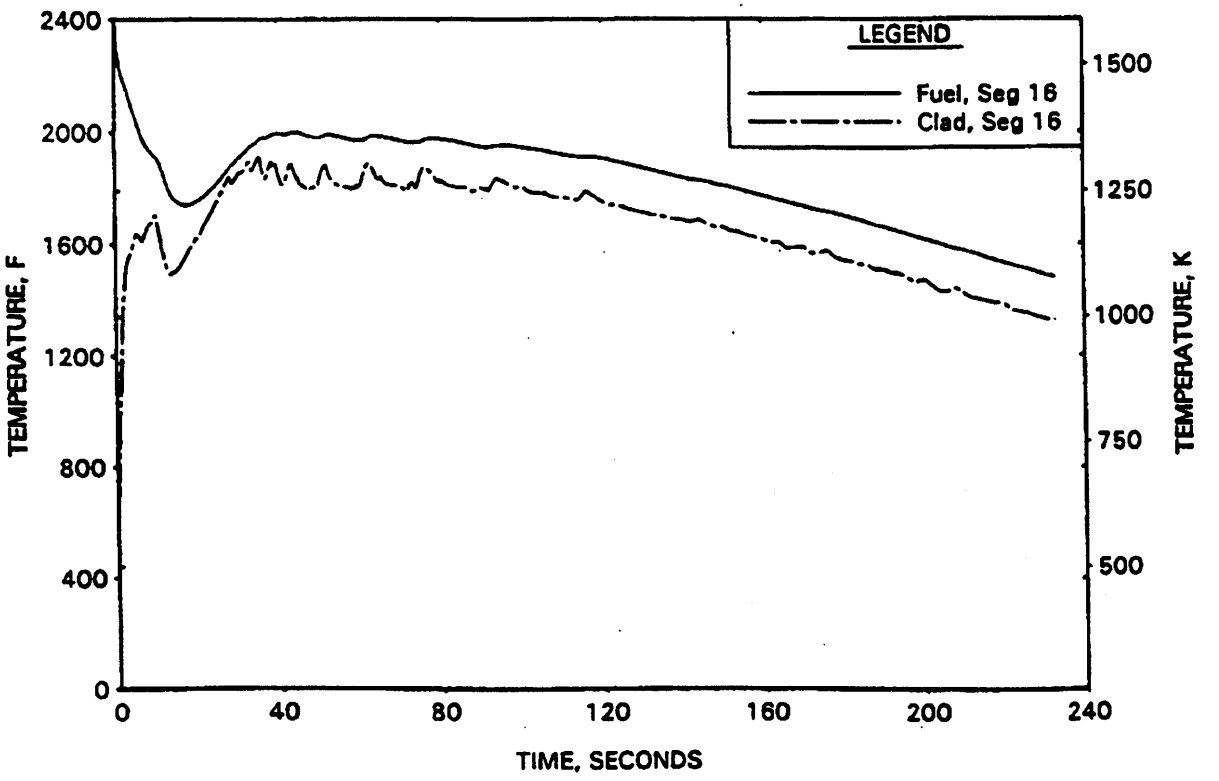
4/00

LBLOCA LIMIT CASE (BOL – 6.021–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK UNRUPTURED LOCATION

CAD FILE: 6578R15.DWG

FIG 14.2–16



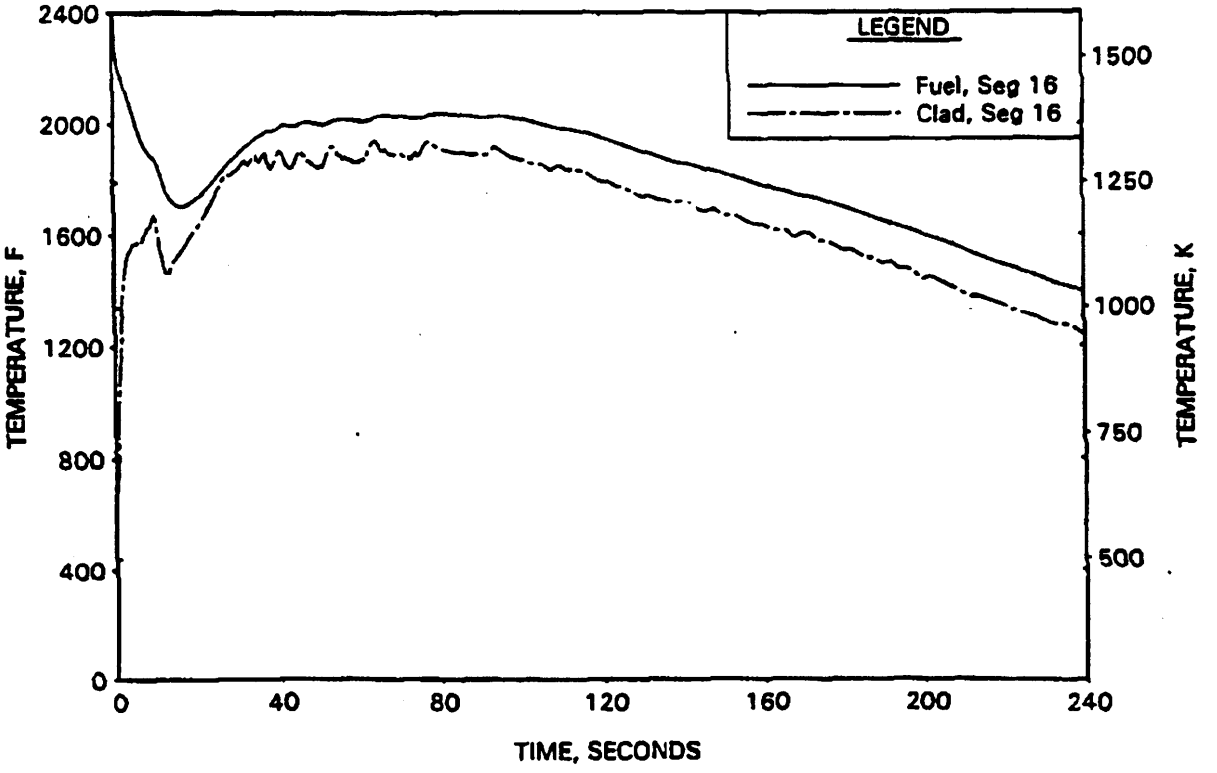
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

LBLOCA LIMIT CASE (BOL – 7.779–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK UNRUPTURED LOCATION

CAD FILE: 6579R15.DWG

FIG 14.2–17

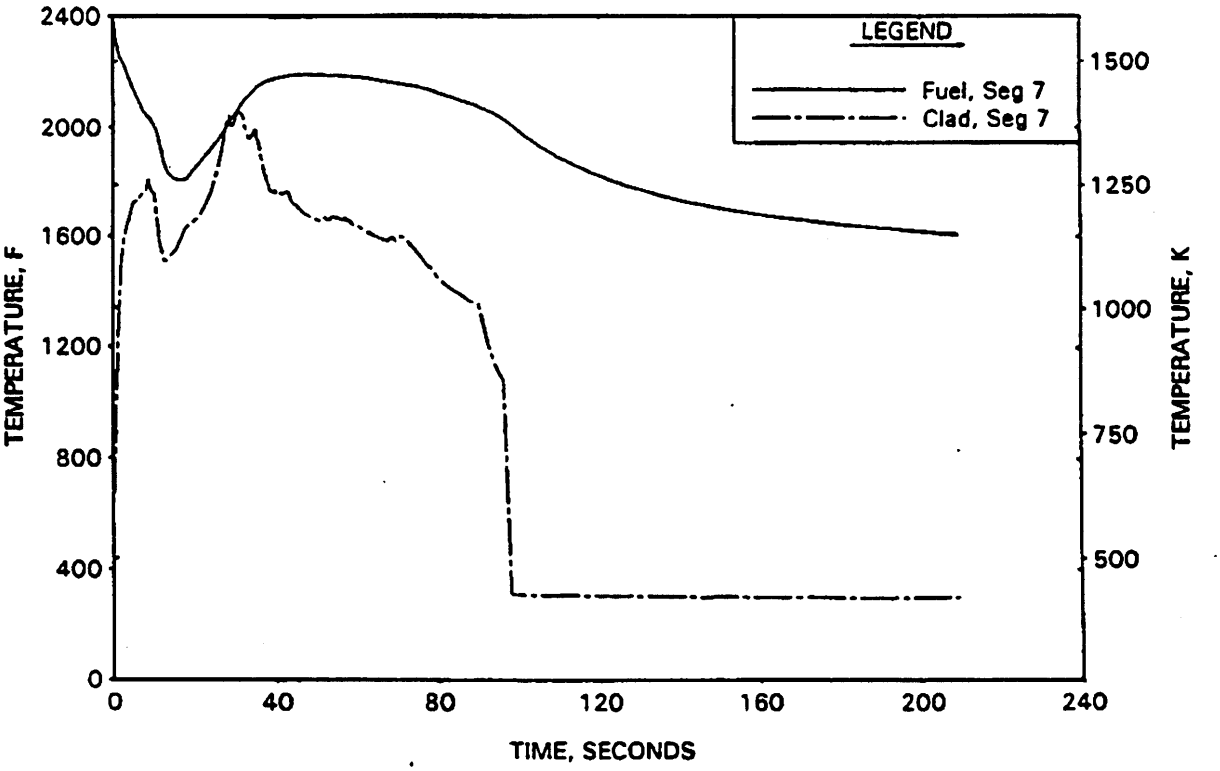


AmerGen
TMI – UNIT 1

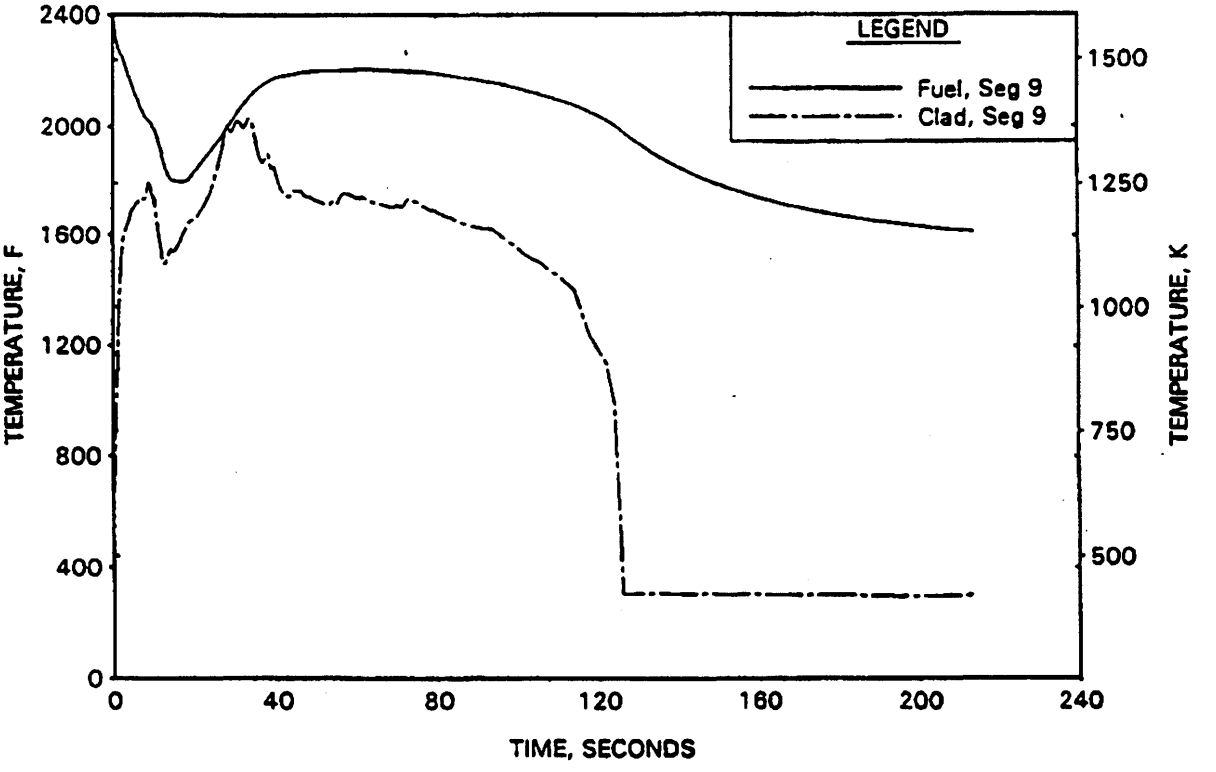
UPDATE – 15
4/00

LBLOCA LIMIT CASE (BOL – 9.536–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK UNRUPTURED LOCATION

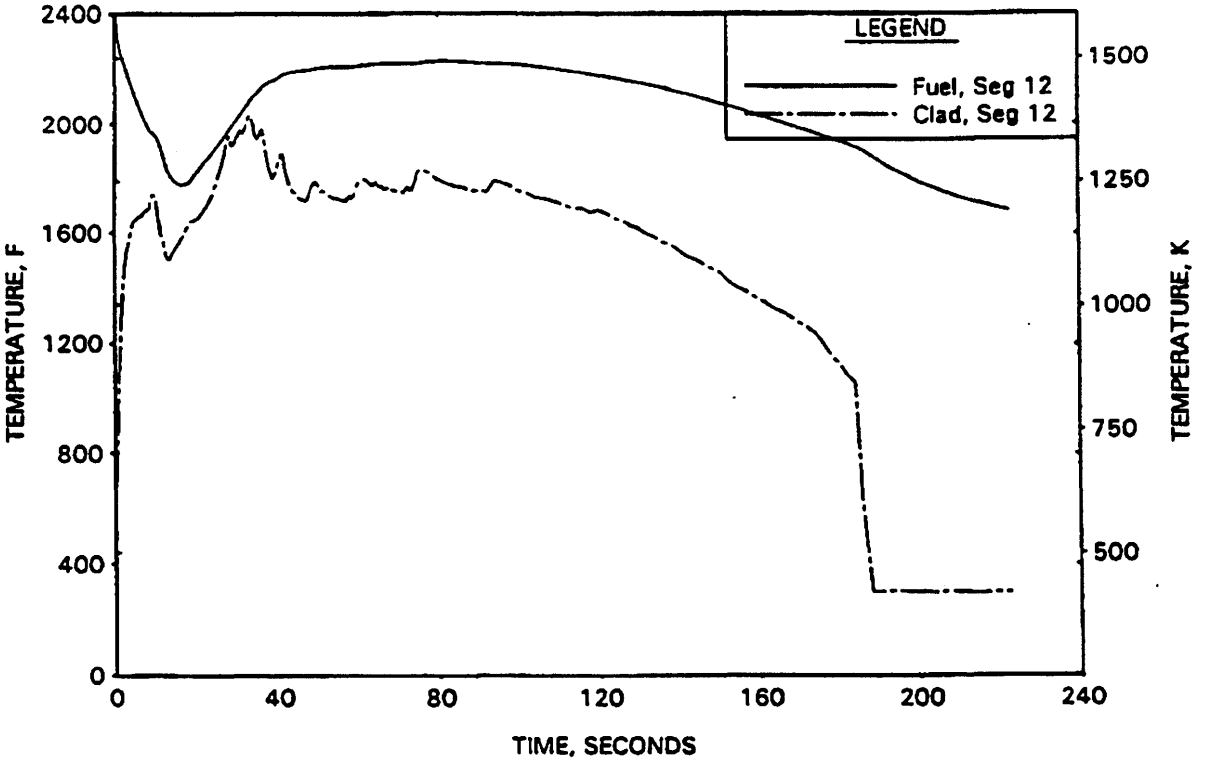
CAD FILE: 6580R15.DWG FIG 14.2–18



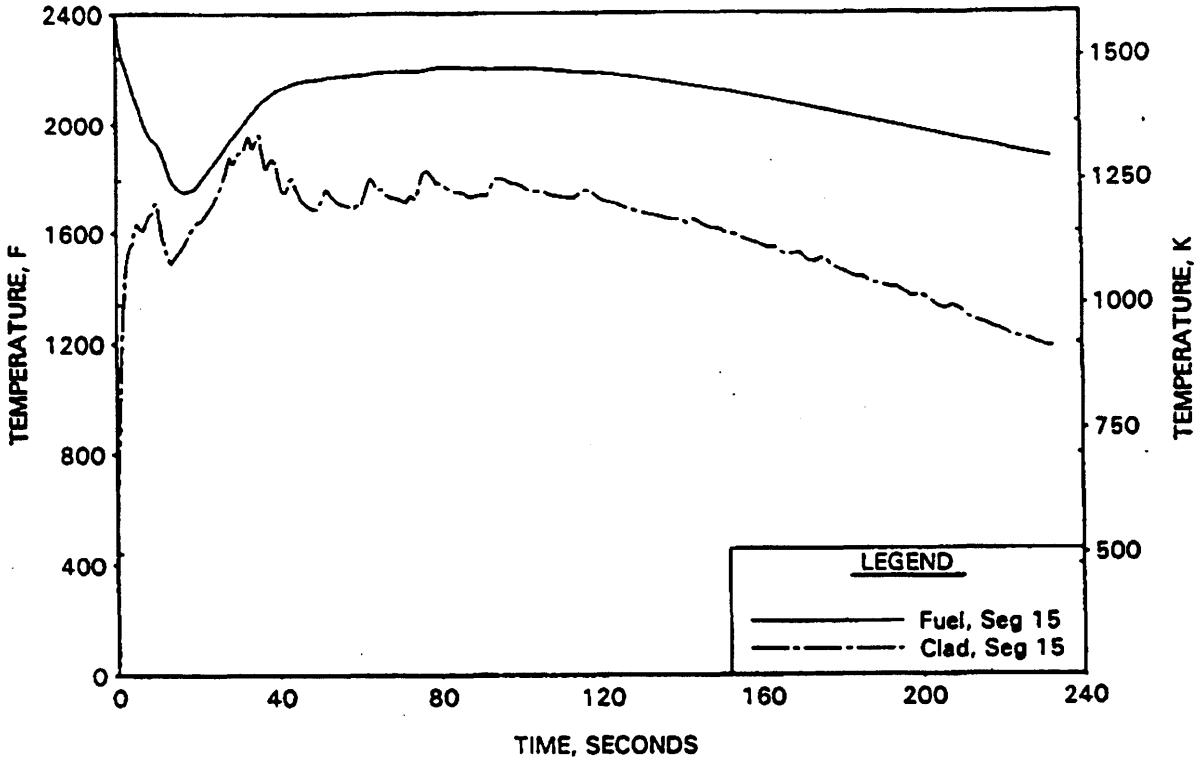
AmerGen TMI – UNIT 1	UPDATE – 15 4/00
LBLOCA LIMIT CASE (BOL – 2.506–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK UNRUPTURED LOCATION	
CAD FILE: 6581R15.DWG	FIG 14.2–19



AmerGen TMI – UNIT 1	UPDATE – 15 4/00
LBLOCA LIMIT CASE (BOL – 4.264–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK RUPTURED LOCATION	
CAD FILE: 6582R15.DWG	FIG 14.2–20



AmerGen TMI – UNIT 1	UPDATE – 15 4/00
LBLOCA LIMIT CASE (BOL – 6.021–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK RUPTURED LOCATION	
CAD FILE: 6583R15.DWG	FIG 14.2–21



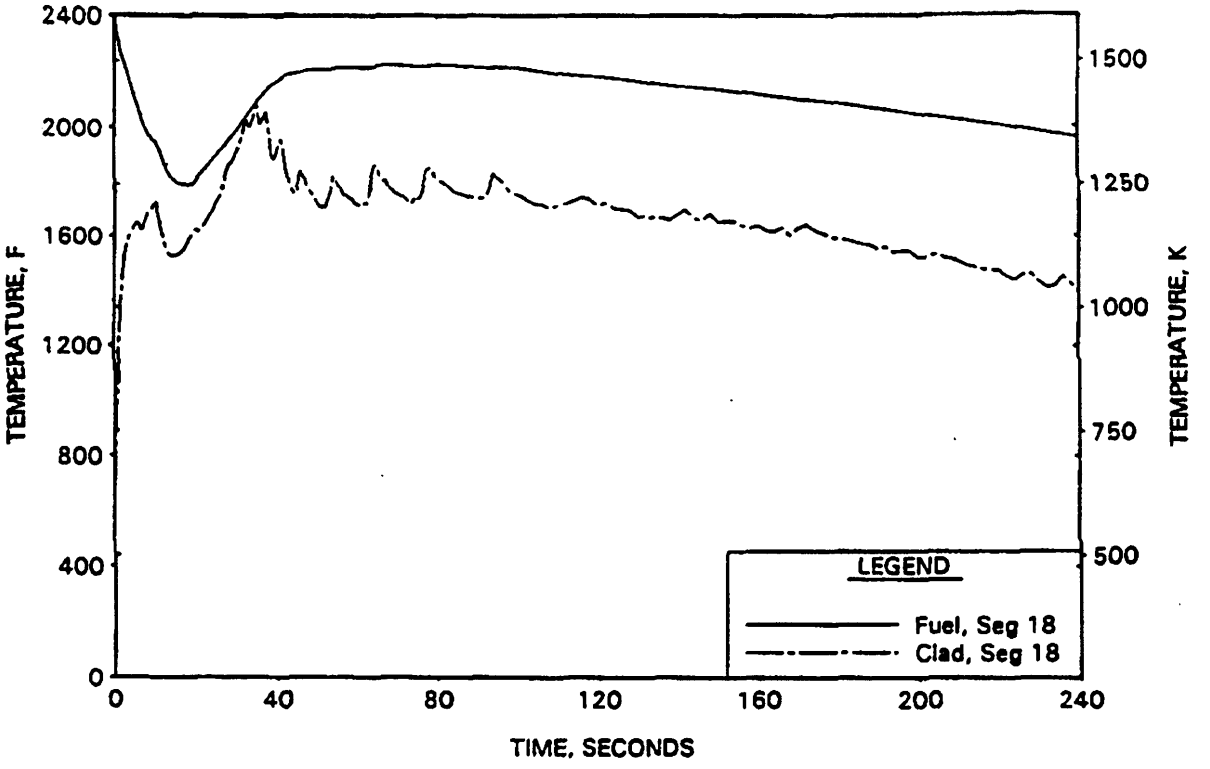
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

LBLOCA LIMIT CASE (BOL – 7.779–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK RUPTURED LOCATION

CAD FILE: 6584R15.DWG

FIG 14.2–22

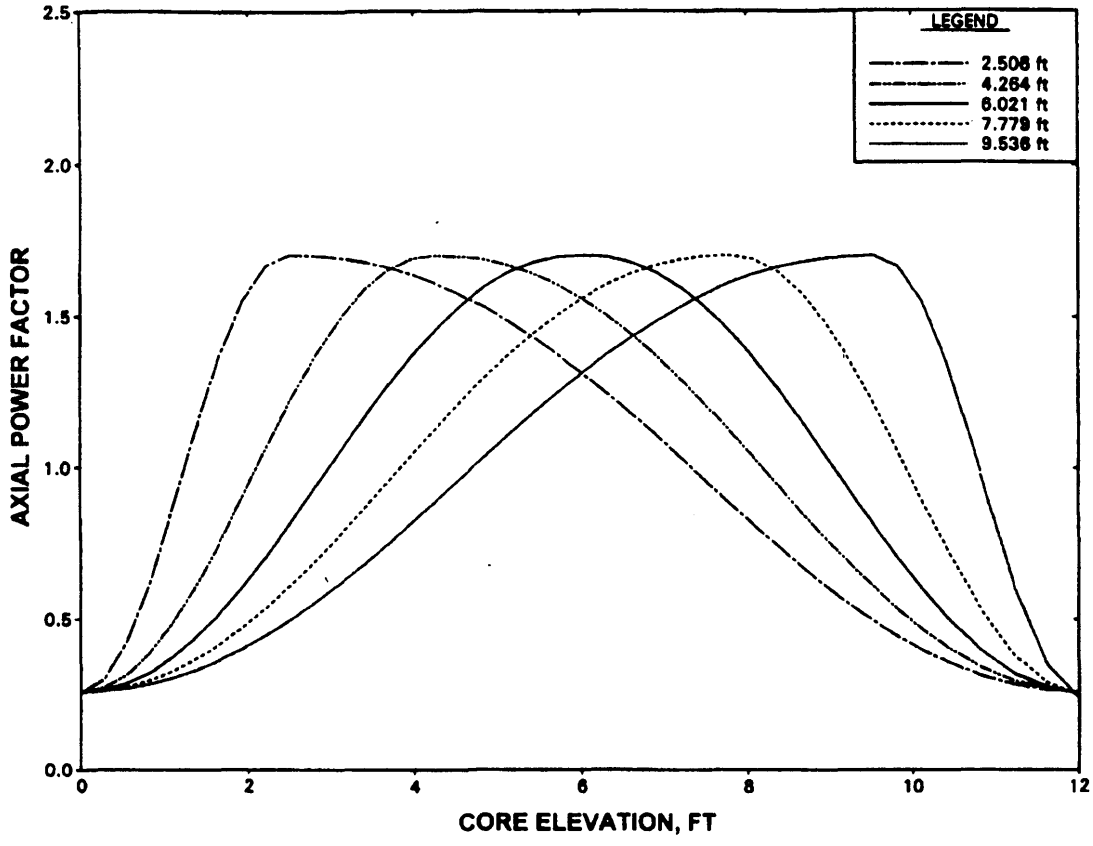


AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

LBLOCA LIMIT CASE (BOL – 9.536–FT) – HOT CHANNEL FUEL AND CLAD TEMPERATURE AT PEAK RUPTURED LOCATION

CAD FILE: 6585R15.DWG FIG 14.2–23



TMI - UNIT 1

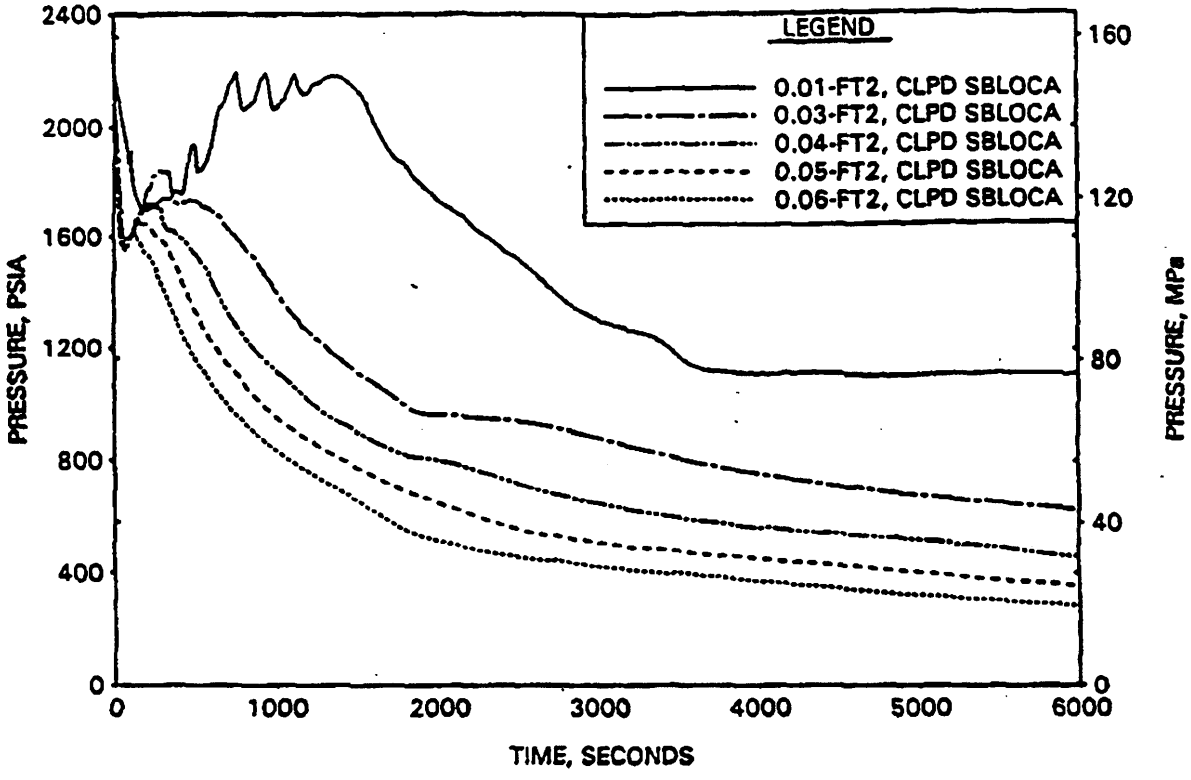
UPDATE - 14
4/98

GPU NUCLEAR

LOCA LIMIT AXIAL POWER SHAPES

CAD FILE: 6586RI4.DWG

FIG 14.2-24



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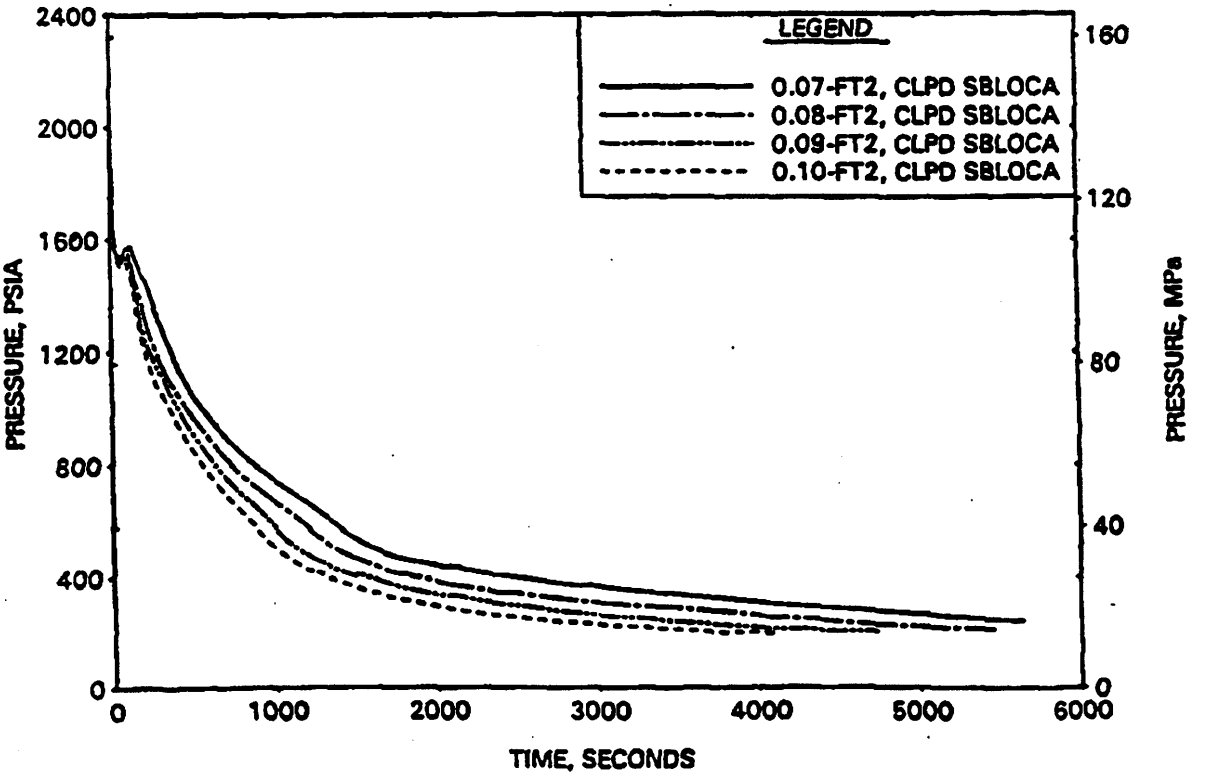
UPDATE – 15
4/00

SBLOCA COMPARISON OF RCS PRESSURE
(0.01 – 0.06 FT^2 BREAKS)

CAD FILE: 6587R15.DWG

FIG 14.2–25



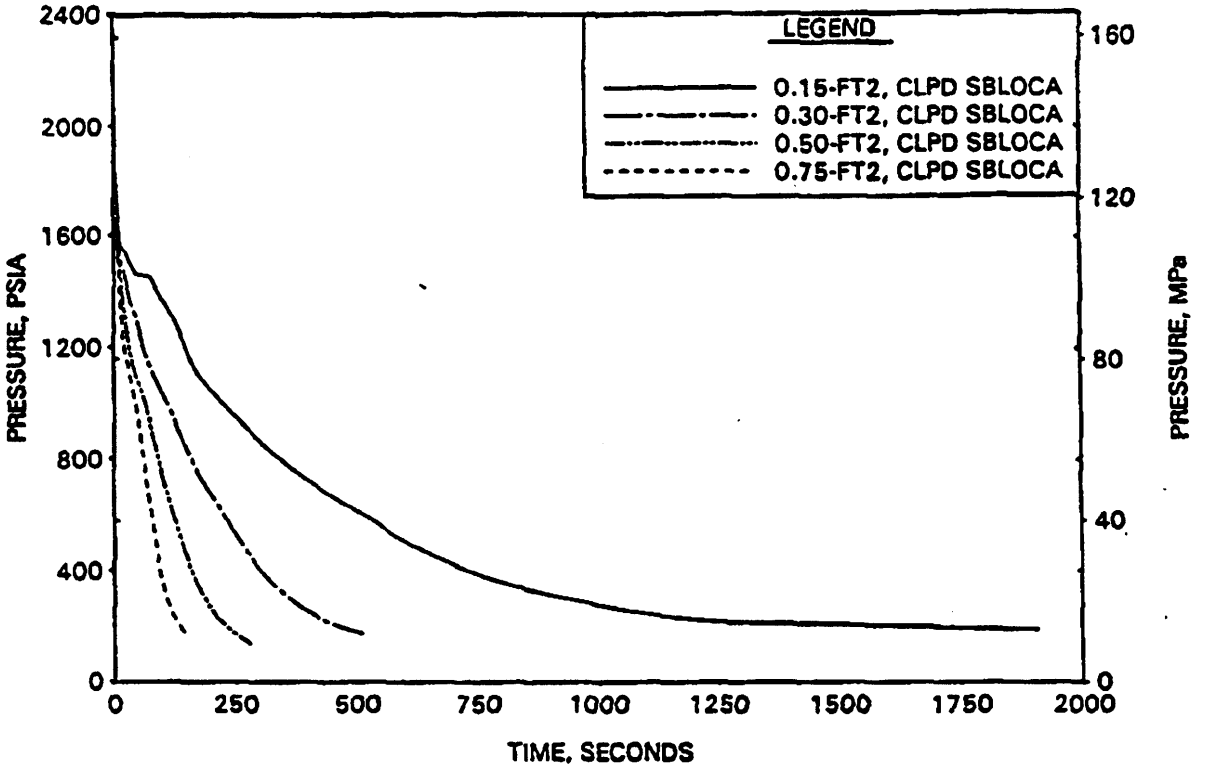
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

SBLOCA COMPARISON OF RCS PRESSURE
(0.07 – 0.10 FT^2 BREAKS)

CAD FILE: 6588R15.DWG

FIG 14.2–26



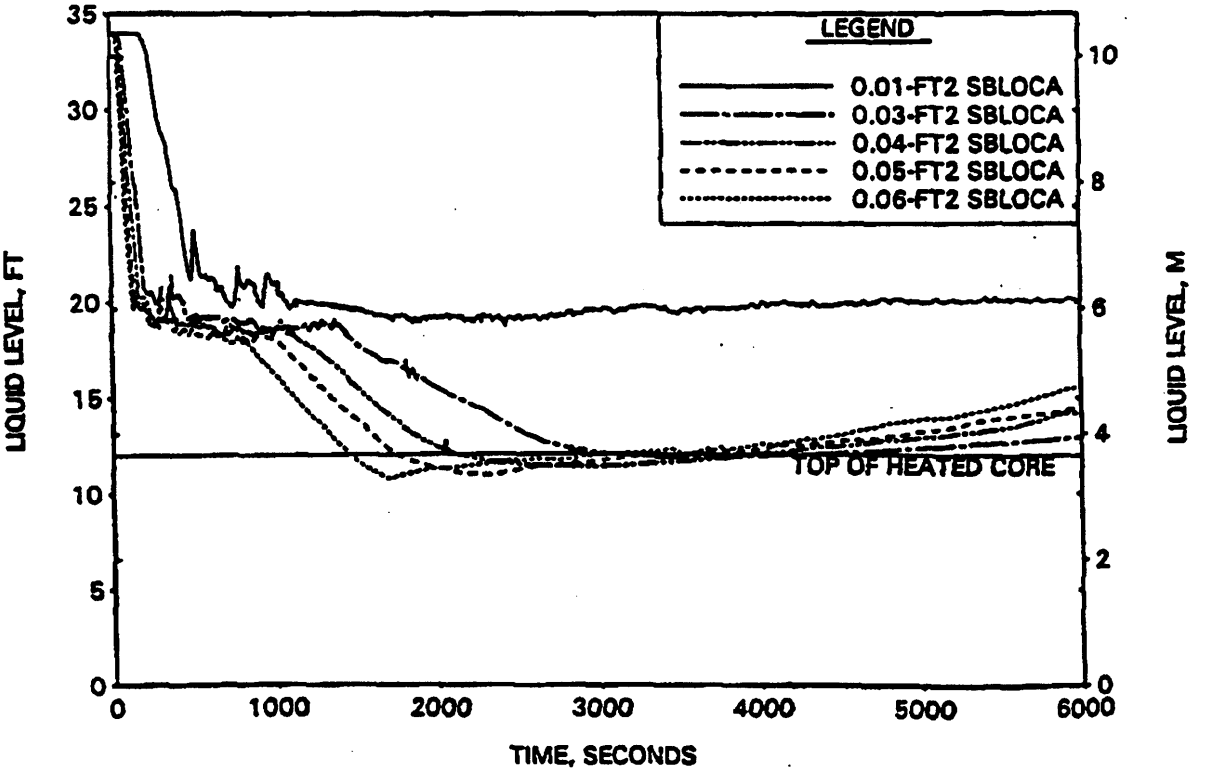
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

SBLOCA COMPARISON OF RCS PRESSURE
(0.15 – 0.75 FT² BREAKS)

CAD FILE: 6589R15.DWG

FIG 14.2–27



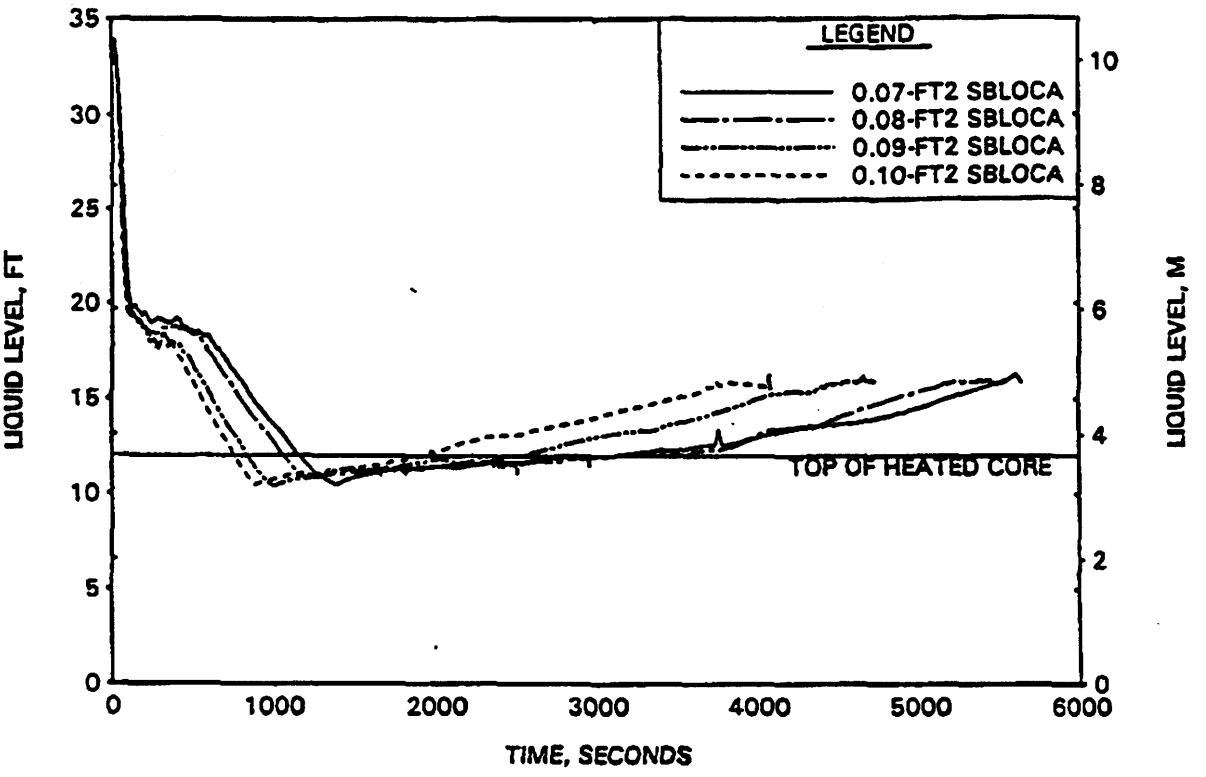
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

SBLOCA COMPARISON OF REACTOR VESSEL COLLAPSED LIQUID LEVEL (0.01 – 0.06 FT^2 BREAKS)

CAD FILE: 6605R15.DWG

FIG 14.2-28



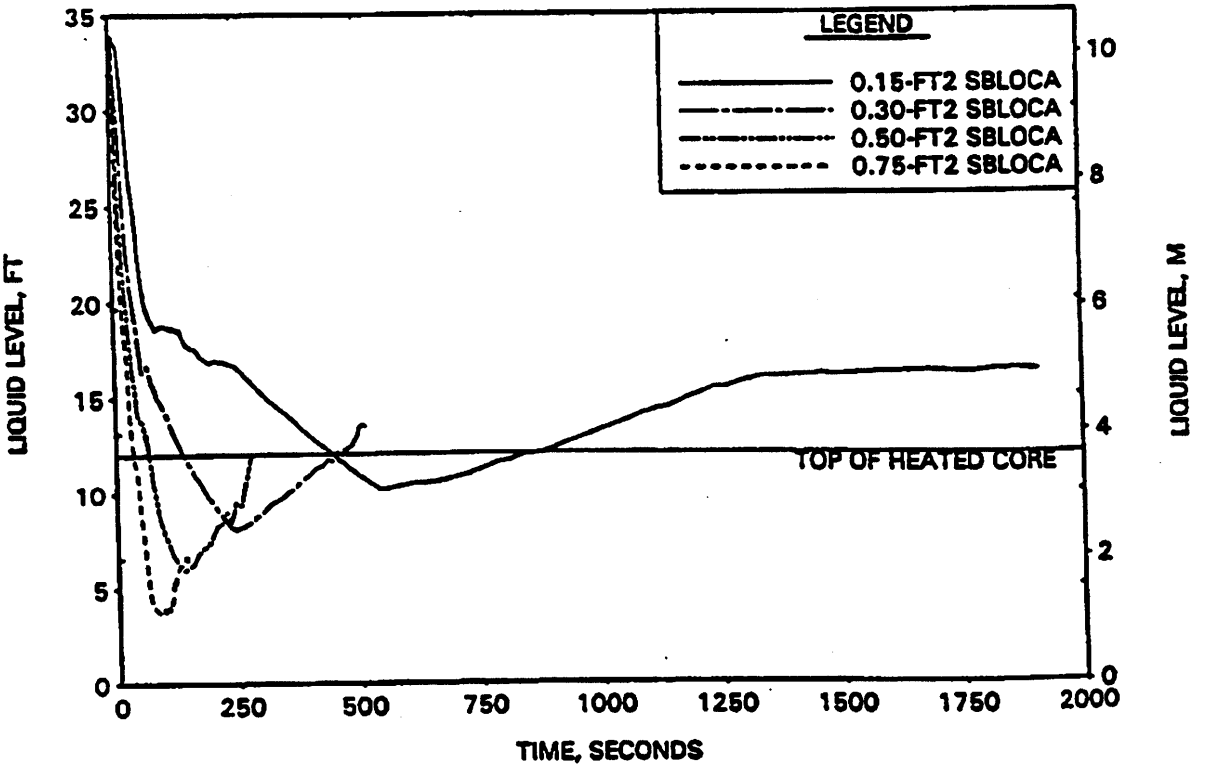
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

SBLOCA COMPARISON OF REACTOR VESSEL COLLAPSED LIQUID LEVEL (0.07 – 0.10 FT^2 BREAKS)

CAD FILE: 6606R15.DWG

FIG 14.2–29



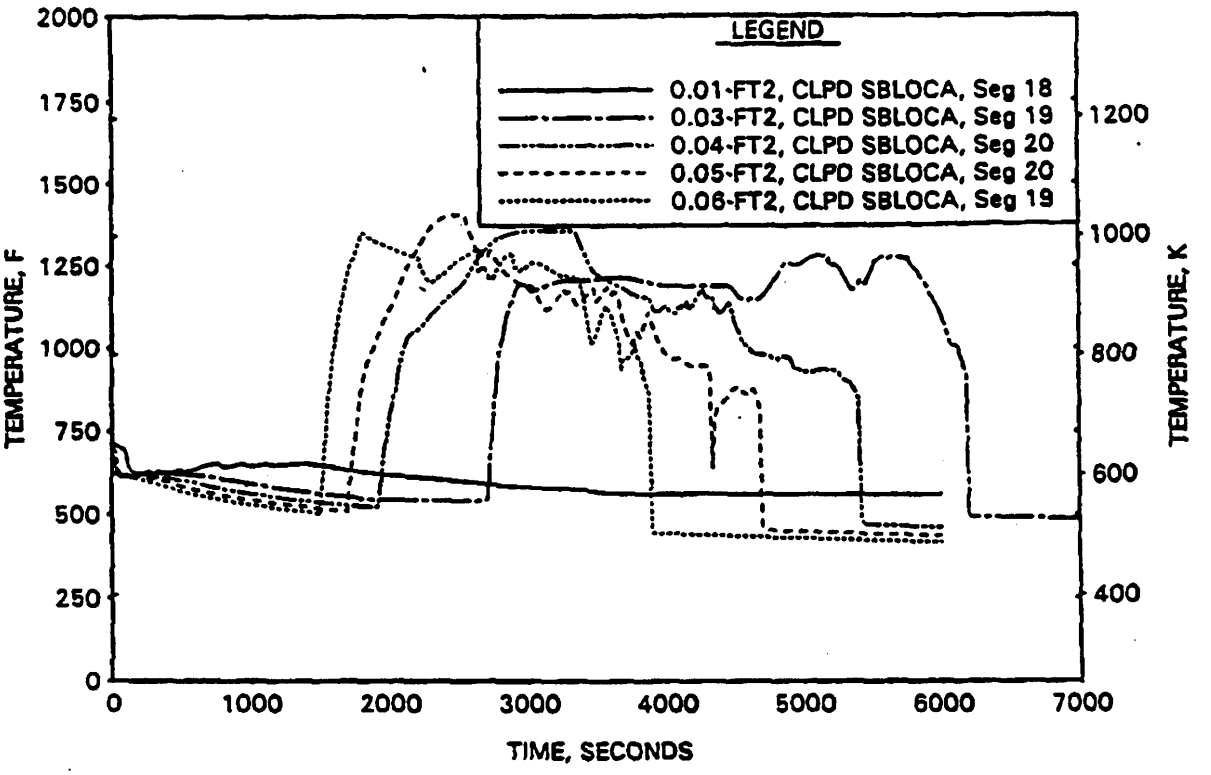
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

SBLOCA COMPARISON OF REACTOR VESSEL COLLAPSED LIQUID LEVEL (0.15 – 0.75 FT^2 BREAKS)

CAD FILE: 6607R15.DWG

FIG 14.2-30



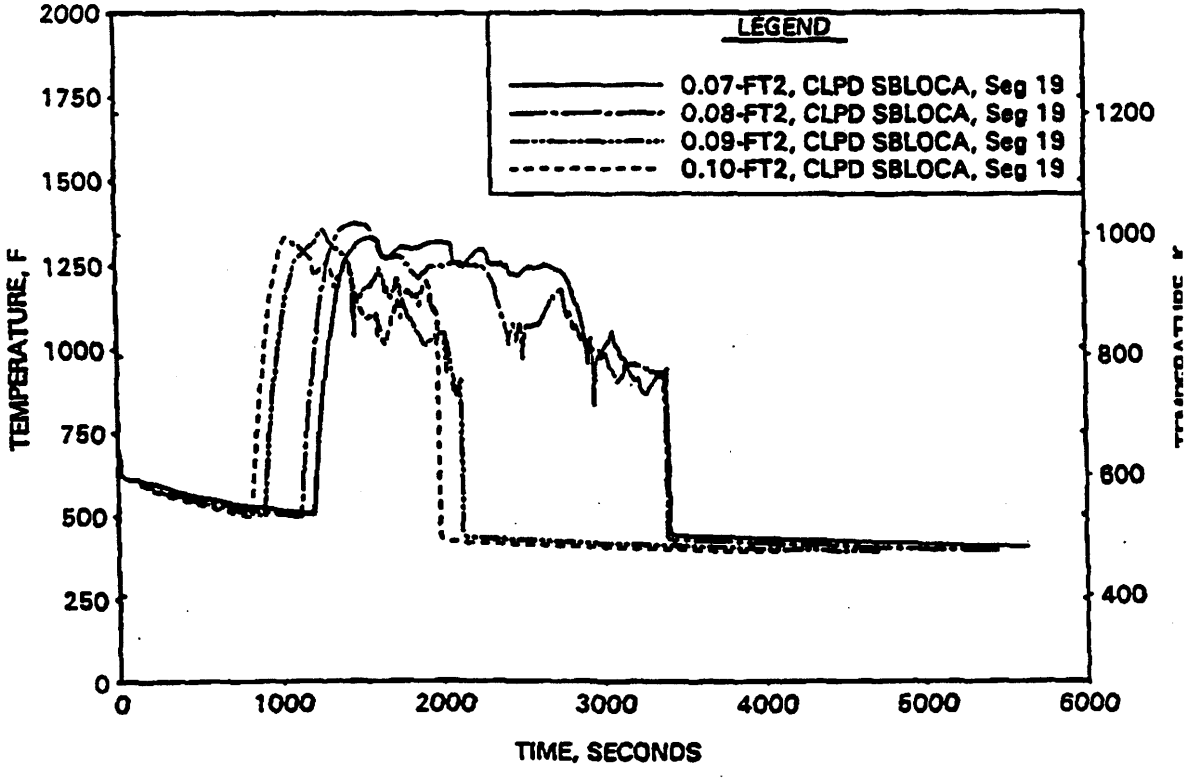
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

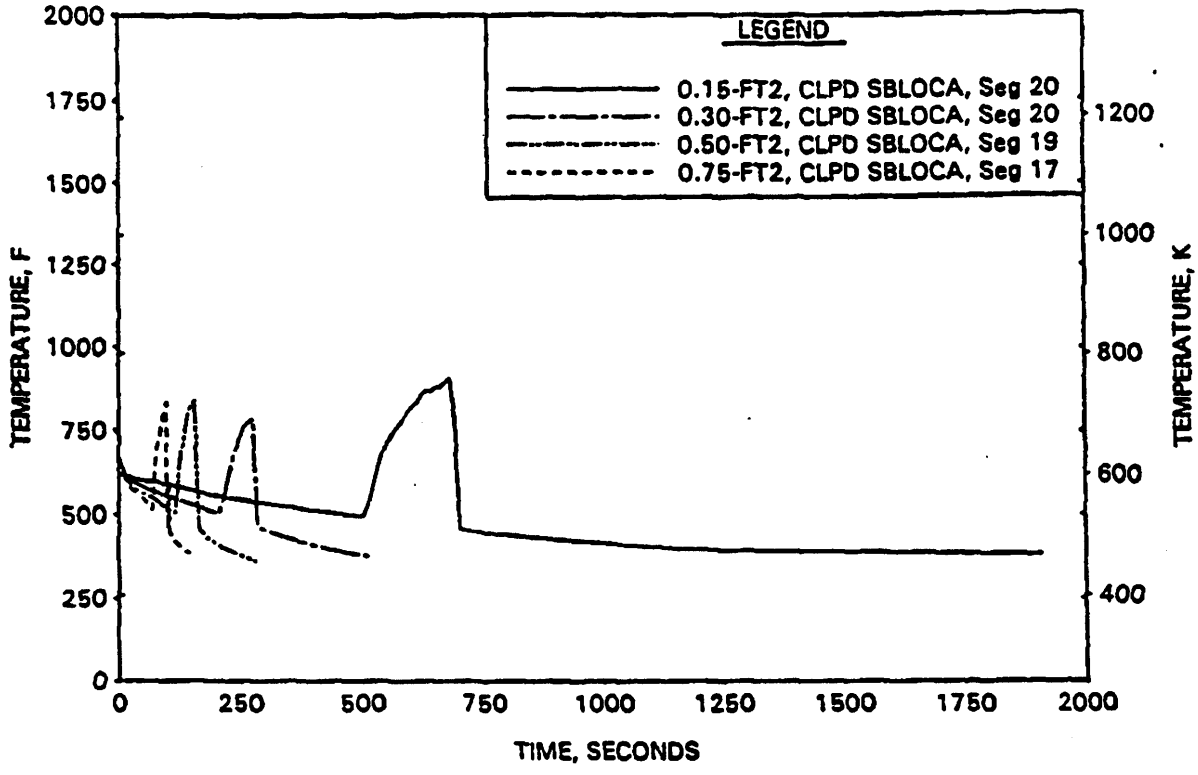
SBLOCA COMPARISON OF PEAK CLAD TEMPERATURES (0.01 – 0.06 FT^2 BREAKS)

CAD FILE: 6608R15.DWG

FIG 14.2–31



AmerGen TMI – UNIT 1	UPDATE – 15 4/00
SBLOCA COMPARISON OF PEAK CLAD TEMPERATURES (0.07 – 0.10 FT^2 BREAKS)	
CAD FILE: 6590R15.DWG	FIG 14.2–32



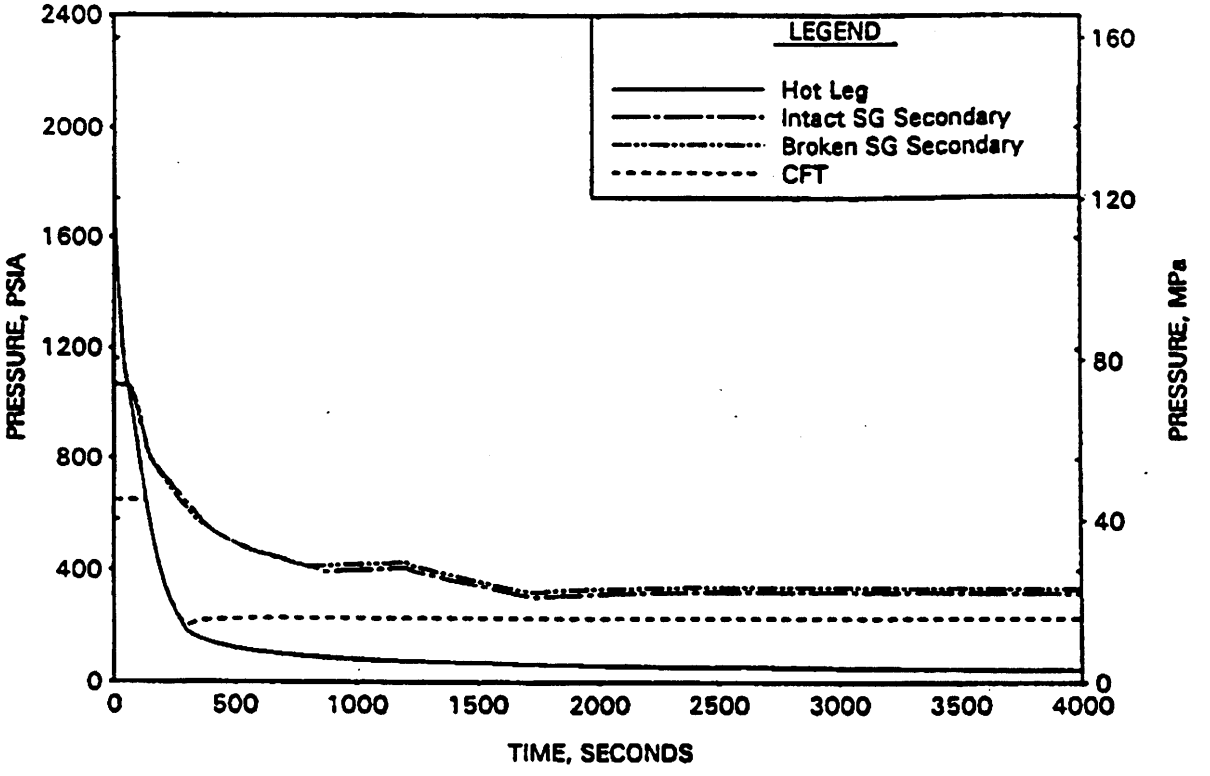
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

SBLOCA COMPARISON OF PEAK CLAD TEMPERATURES (0.15 – 0.75 FT^2 BREAKS)

CAD FILE: 6591R15.DWG

FIG 14.2–33



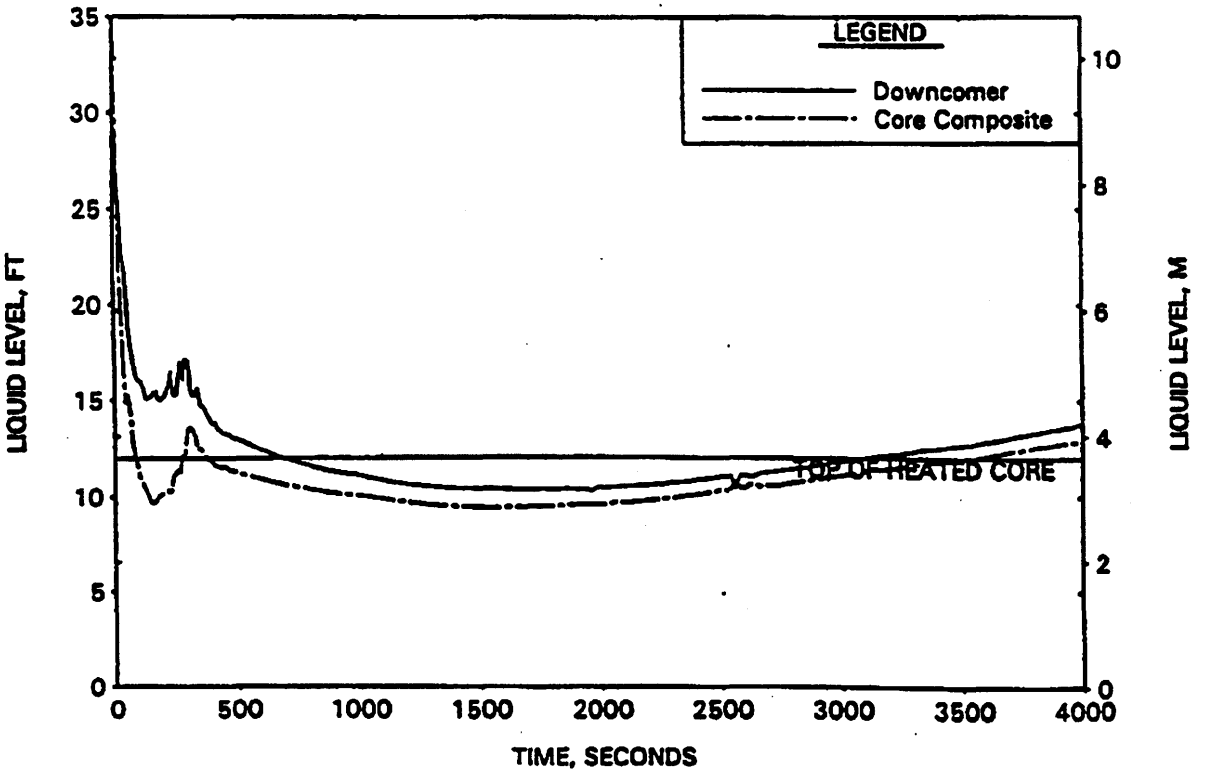
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

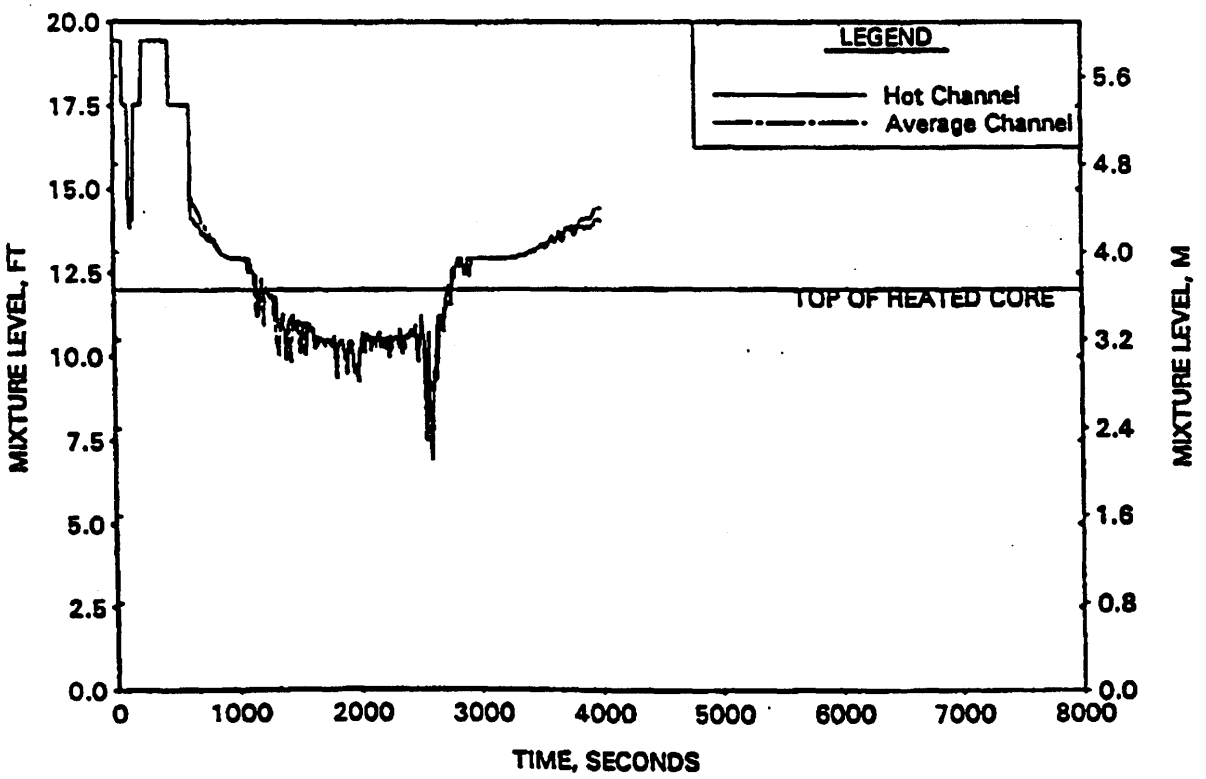
CFT LINE BREAK SYSTEM PRESSURE

CAD FILE: 6592R15.DWG

FIG 14.2–34



AmerGen TMI – UNIT 1	UPDATE – 15 4/00
CFT LINE BREAK COLLAPSED LIQUID LEVELS	
CAD FILE: 6593R15.DWG	FIG 14.2–35



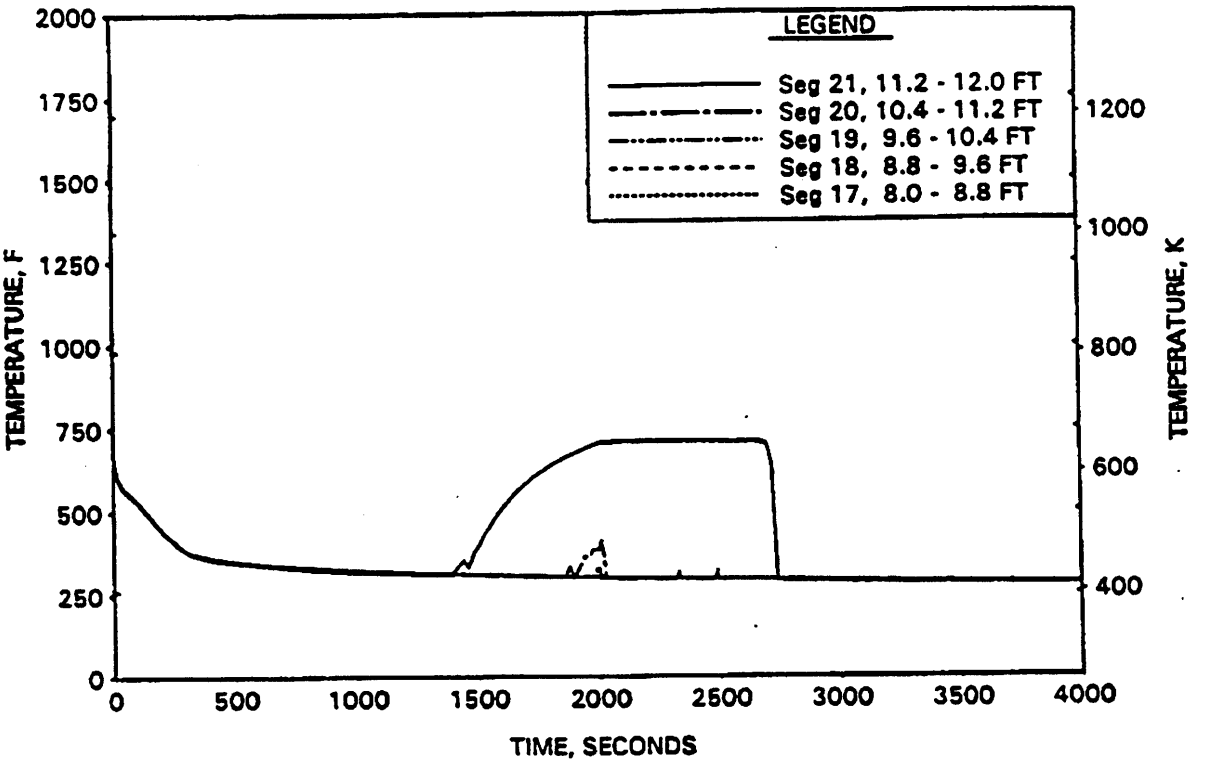
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

CFT LINE BREAK MIXTURE LEVELS

CAD FILE: 6594R15.DWG

FIG 14.2–36



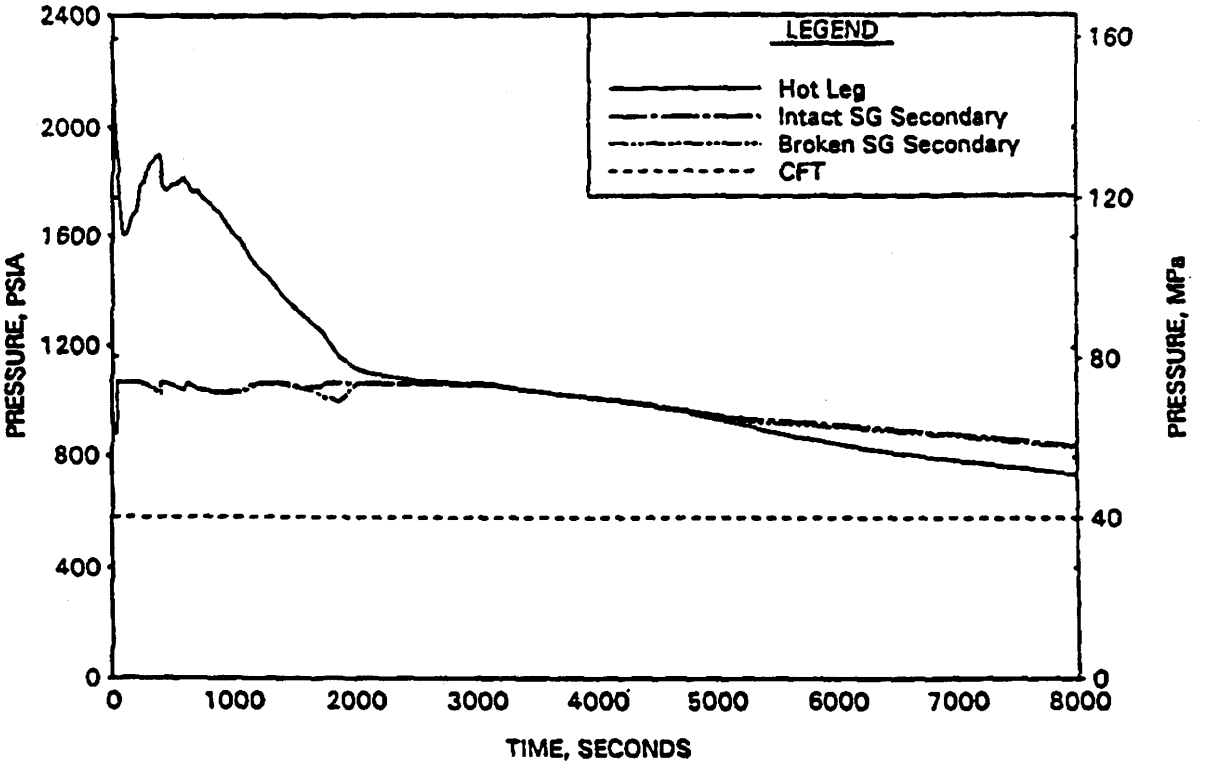
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

CFT LINE BREAK HOT CHANNEL CLAD TEMPERATURES

CAD FILE: 6595R15.DWG

FIG 14.2–37



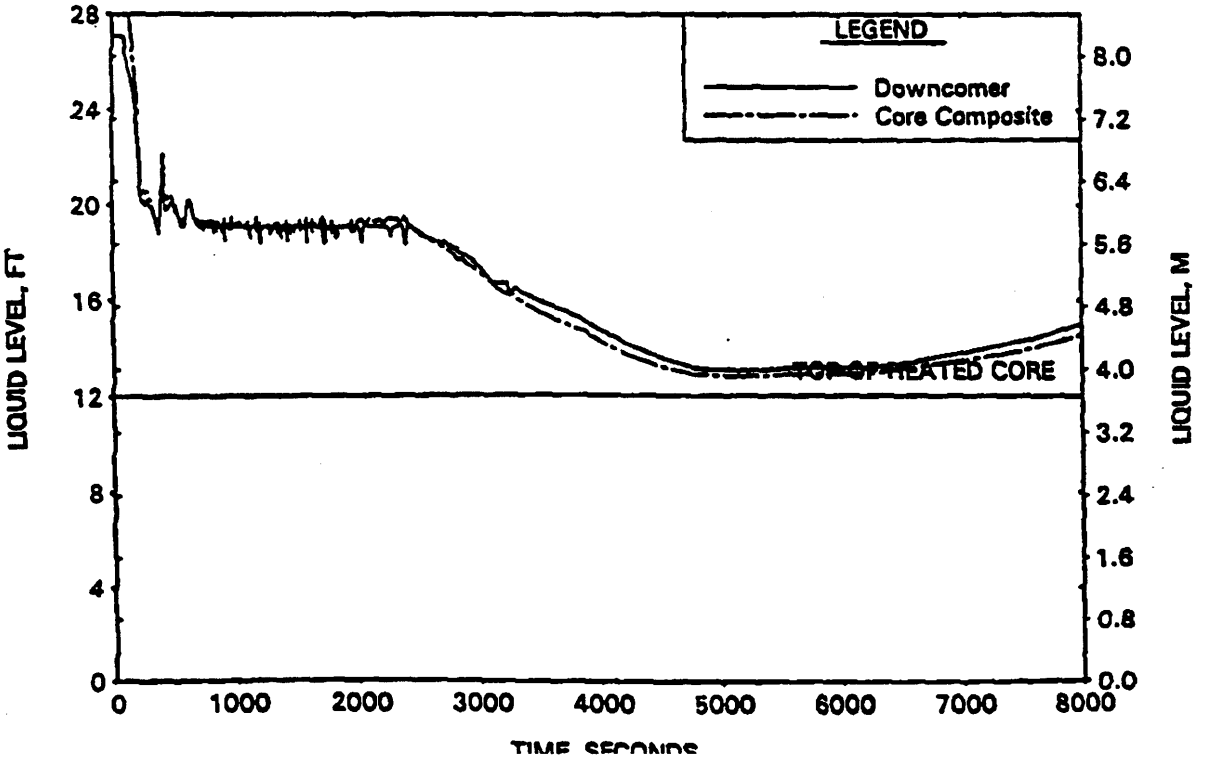
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

HPI LINE BREAK SYSTEM PRESSURE
(OPERATOR ACTION)

CAD FILE: 6596R15.DWG

FIG 14.2–38



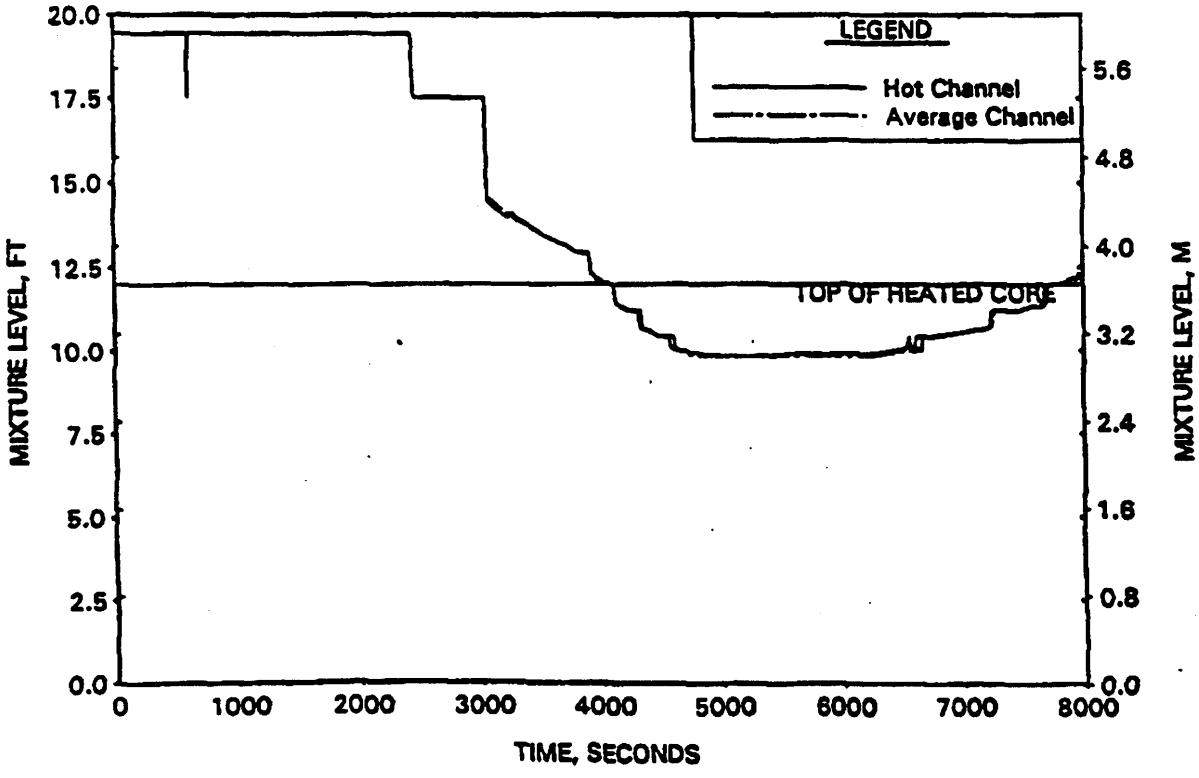
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

HPI LINE BREAK COLLAPSED LIQUID LEVELS (OPERATOR ACTION)

CAD FILE: 6597R15.DWG

FIG 14.2–39



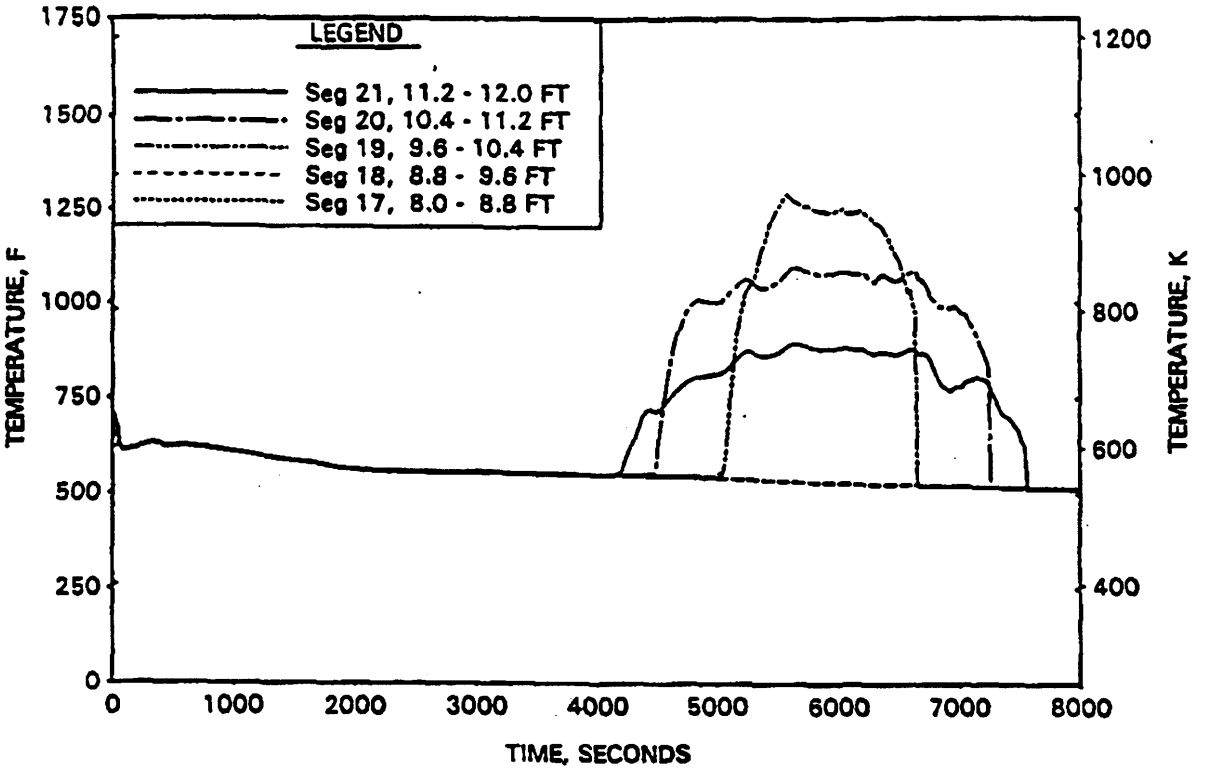
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

HPI LINE BREAK MIXTURE LEVELS
(OPERATOR ACTION)

CAD FILE: 6598R15.DWG

FIG 14.2–40



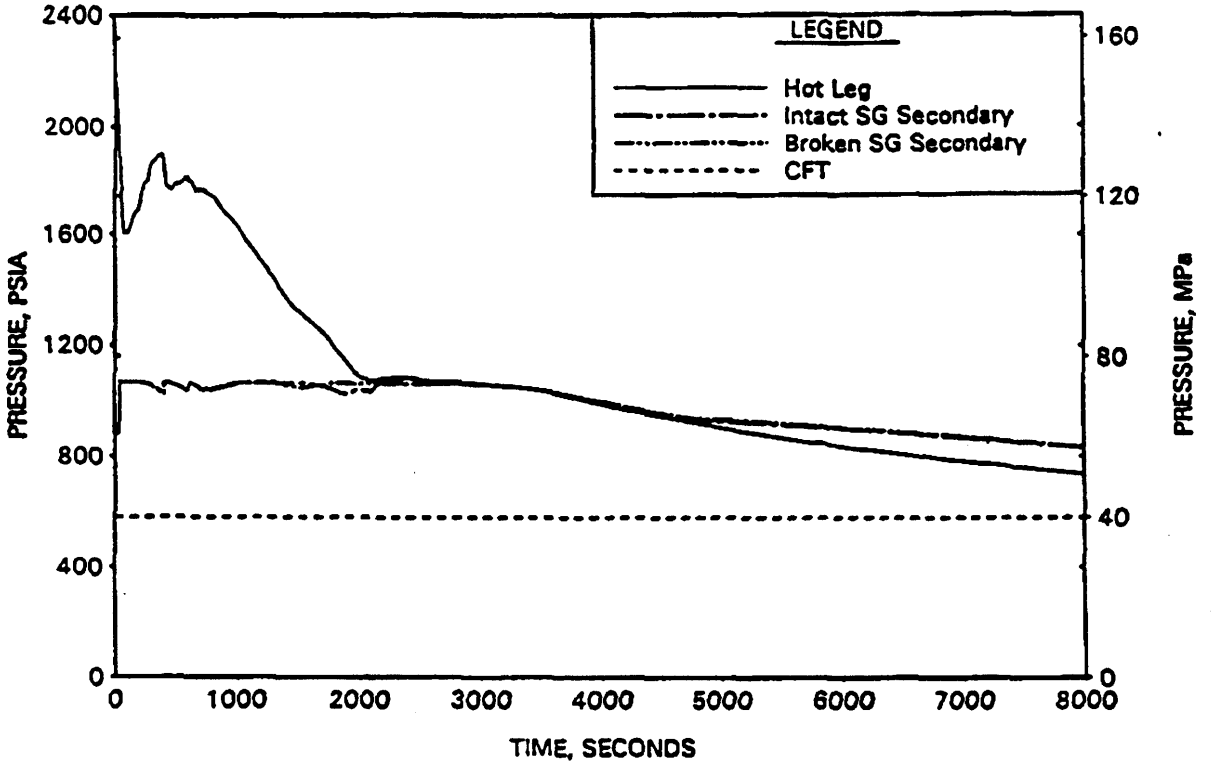
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

HPI LINE BREAK HOT CHANNEL CLAD TEMPERATURES (OPERATOR ACTION)

CAD FILE: 6599R15.DWG

FIG 14.2–41



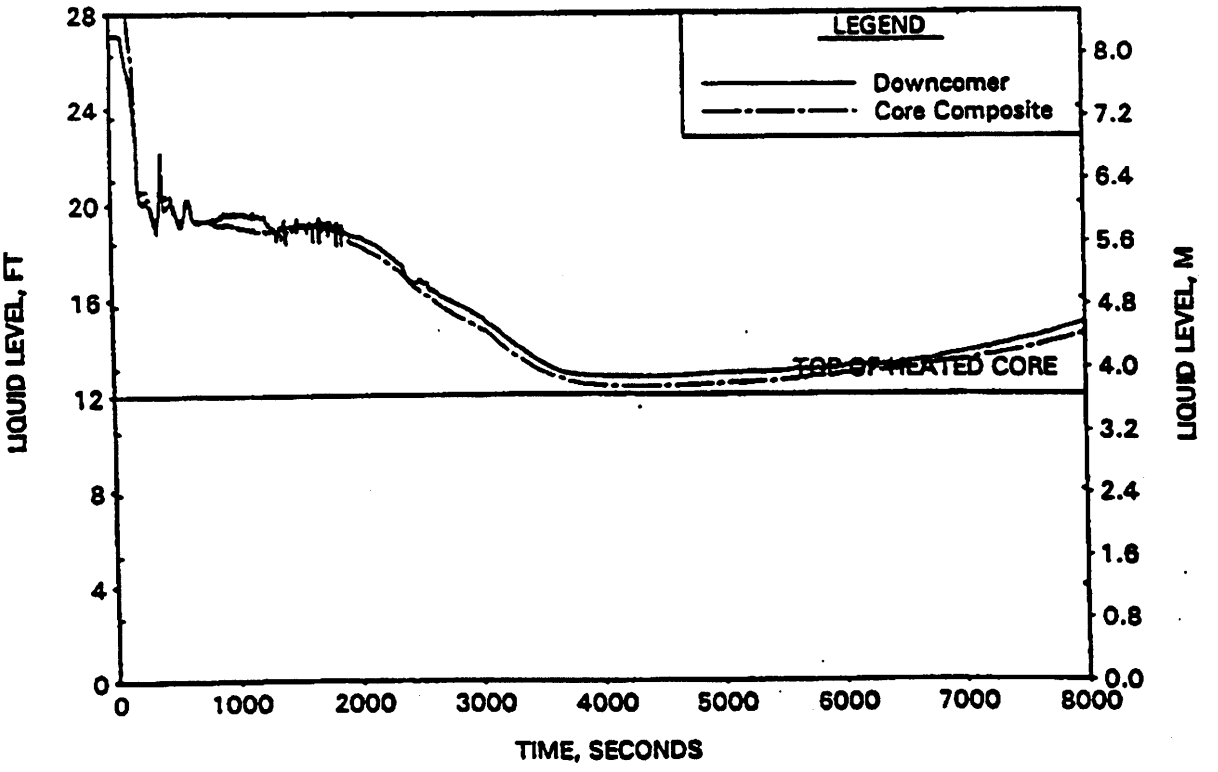
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

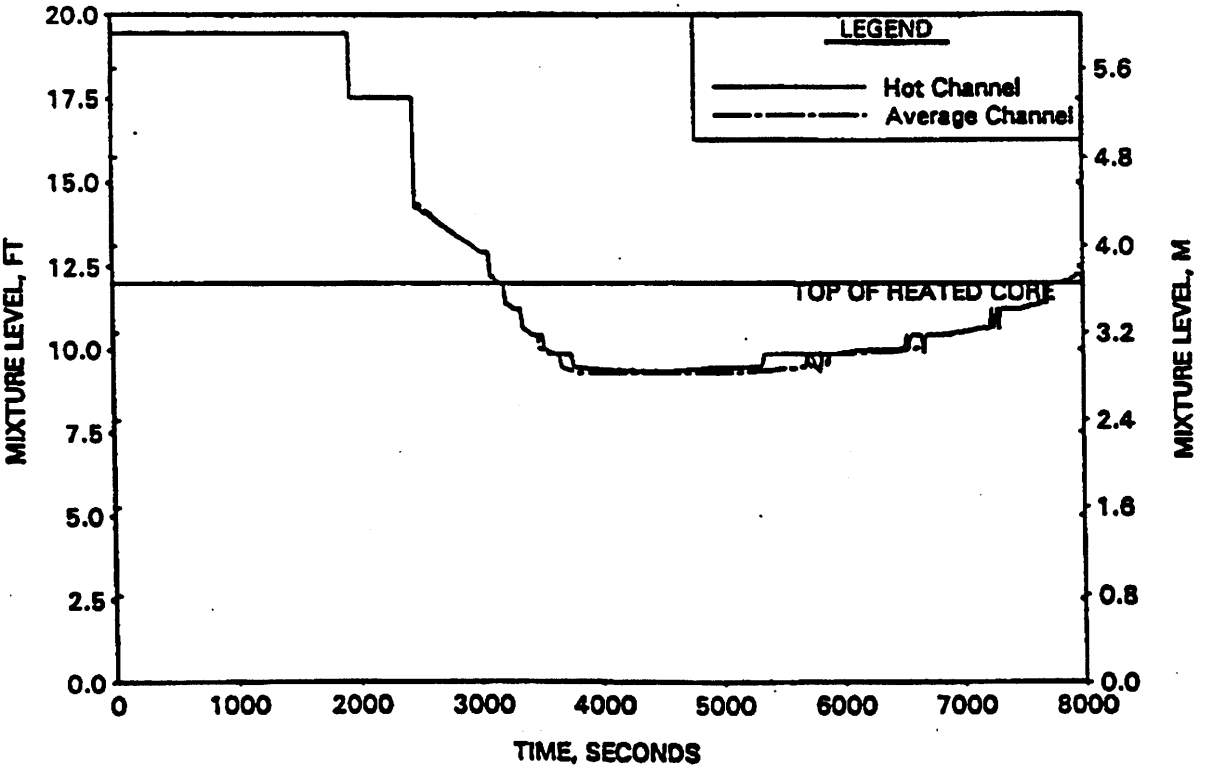
HPI LINE BREAK SYSTEM PRESSURE (NO OPERATOR ACTION)

CAD FILE: 6600R15.DWG

FIG 14.2–42



AmerGen TMI – UNIT 1	UPDATE – 15 4/00
HPI LINE BREAK COLLAPSED LIQUID LEVELS (NO OPERATOR ACTION)	
CAD FILE: 6609R15.DWG	FIG 14.2–43



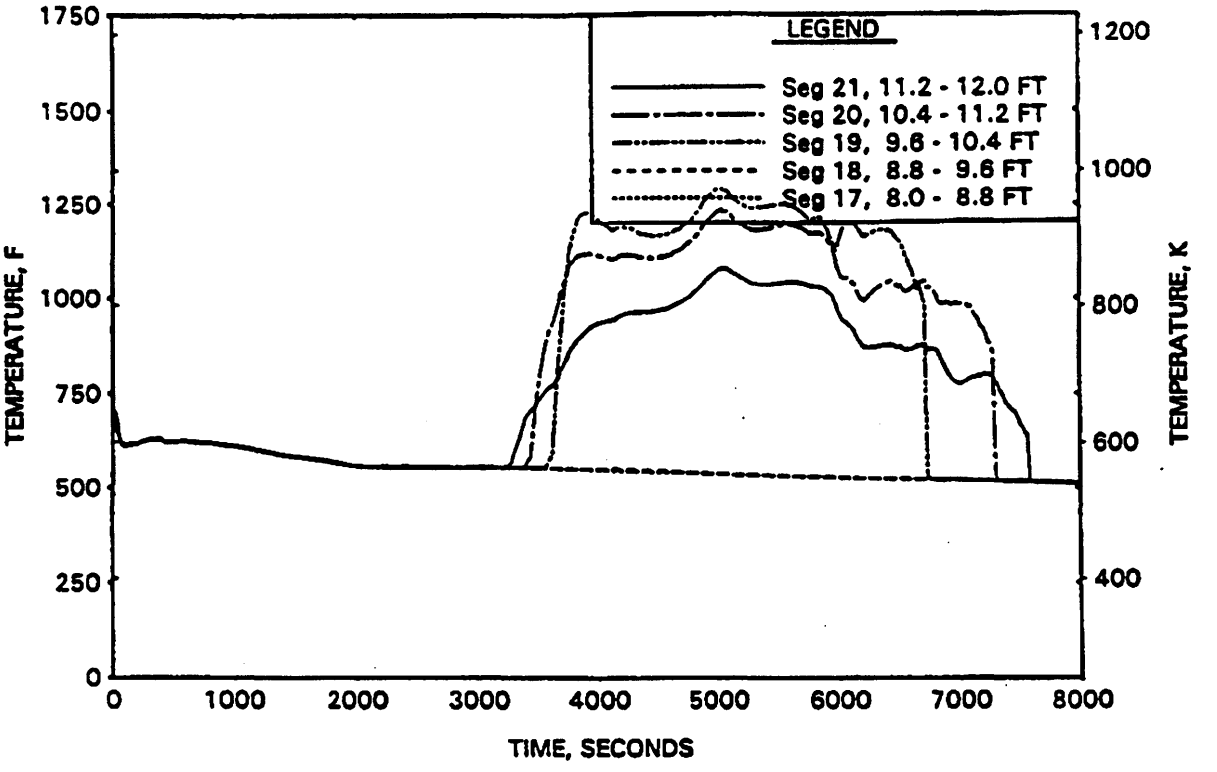
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

HPI LINE BREAK MIXTURE LEVELS (NO OPERATOR ACTION)

CAD FILE: 6610R15.DWG

FIG 14.2–44



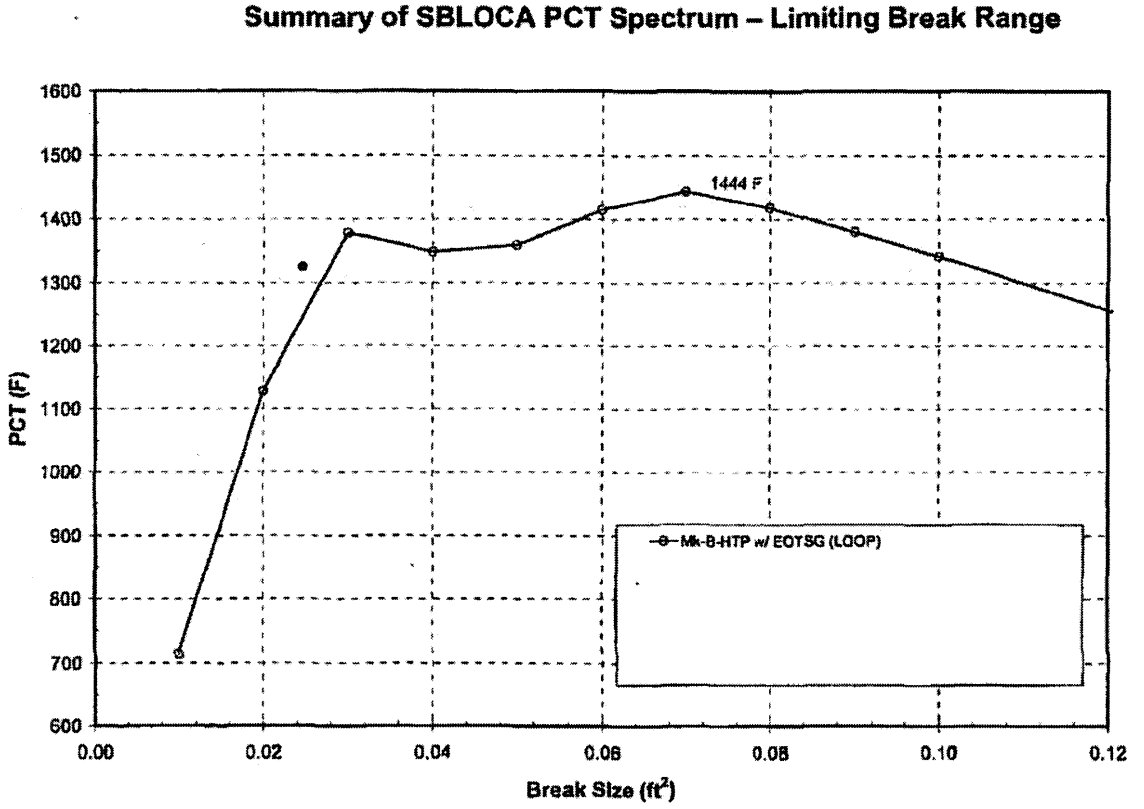
AmerGen
TMI – UNIT 1

UPDATE – 15
4/00

HPI LINE BREAK HOT CHANNEL CLAD TEMPERATURES (NO OPERATOR ACTION)

CAD FILE: 6611R15.DWG

FIG 14.2–45

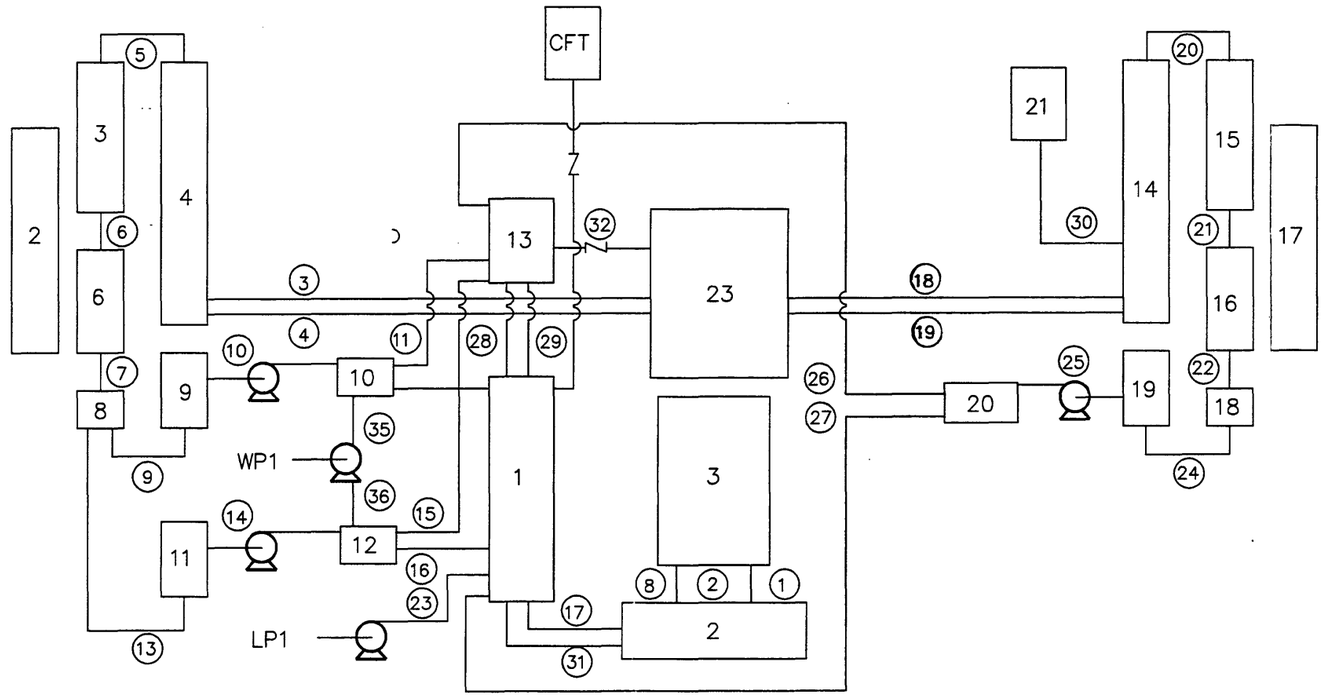


TMI-1
Updated Final Safety Analysis Report

SBLOCA SPECTRUM - PCT VERSUS BREAK SIZE

Figure 14.2-68 **Rev. 20, 04/10**

p. 14.FIG-68



Node No.	Identification
1	Downcomer
2	Lower Plenum
3	Core
4,14	Hot Leg Piping
5,15	SG & Upper Head
6,16	Steam Generator Tubes
7,17	Secondary,SG
8,18	SG Lower Head
9,11,19	Cold Leg Piping
10,12,20	Cold Leg Piping
13	Upper Downcomer
21	Pressurizer
22	Containment
23	Upper Plenum

Path No.	Identification
1,2	Core
3,4,18,19	Hot Leg Piping
5,20	Hot Leg,Upper
6,21	SG Tubes
7,22	SG Lower Head
8	Core Bypass
9,13,24	Cold Leg Piping
10,14,25	Pumps
11,12,15,16,26,27	Cold Leg Piping
17,31	Downcomer
23	LPI
28,29	Upper Downcomer
30	Pressurizer
32	Vent Valve
33,34	Leak & Return Path
35,36	HPI
37	Containment Sprays

Note: Additional Data Not Shown on Diagram are:

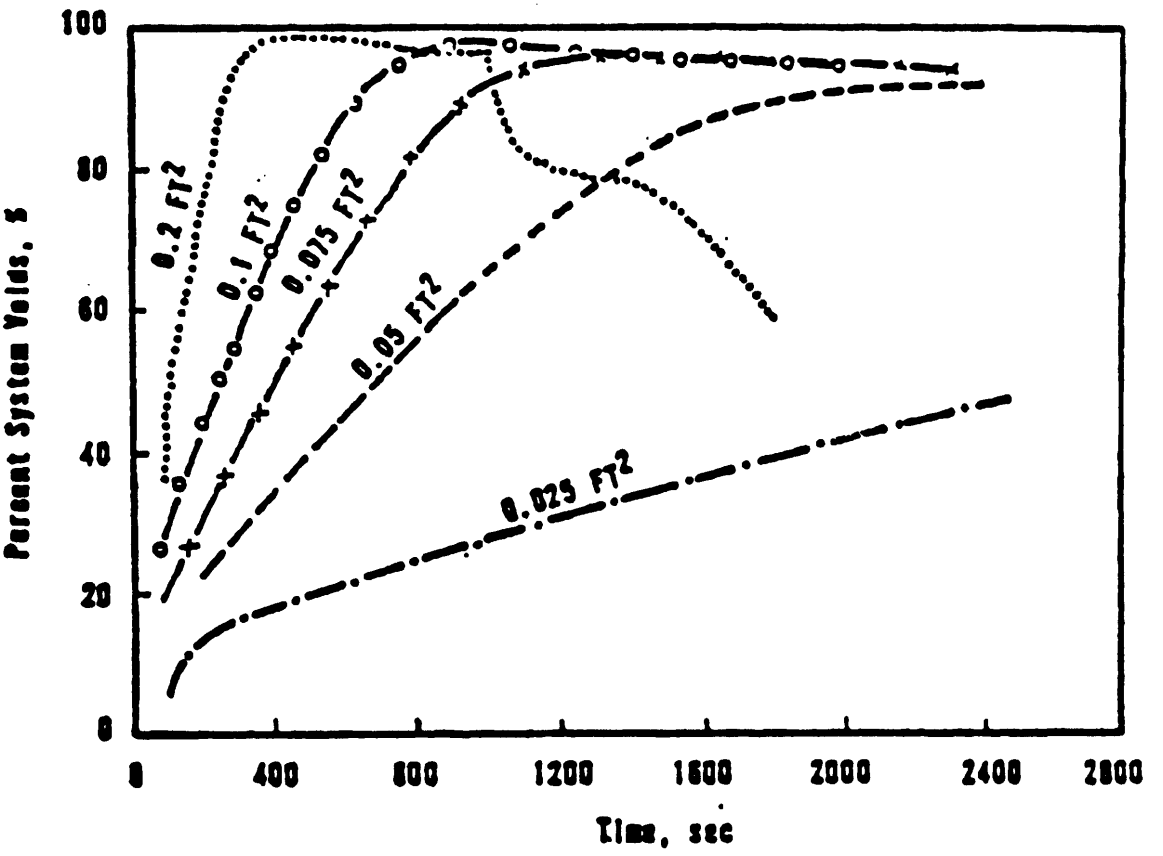
Node 22 Is Containment Node
Path 33 Is Leak Path From Break Node to Containment
Path 34 Is Return Leak Path From Containment to Break Node
Path 37 Represents Containment Spray System

TMI Unit 1 Update–14
4/98

GPU NUCLEAR

Craft 2 Noding Diagram for Small Break

CADD #S1B,SKM,00,0341,000–,1400 Fig.14.2–47



TMI - UNIT 1

UPDATE - 14
4/98

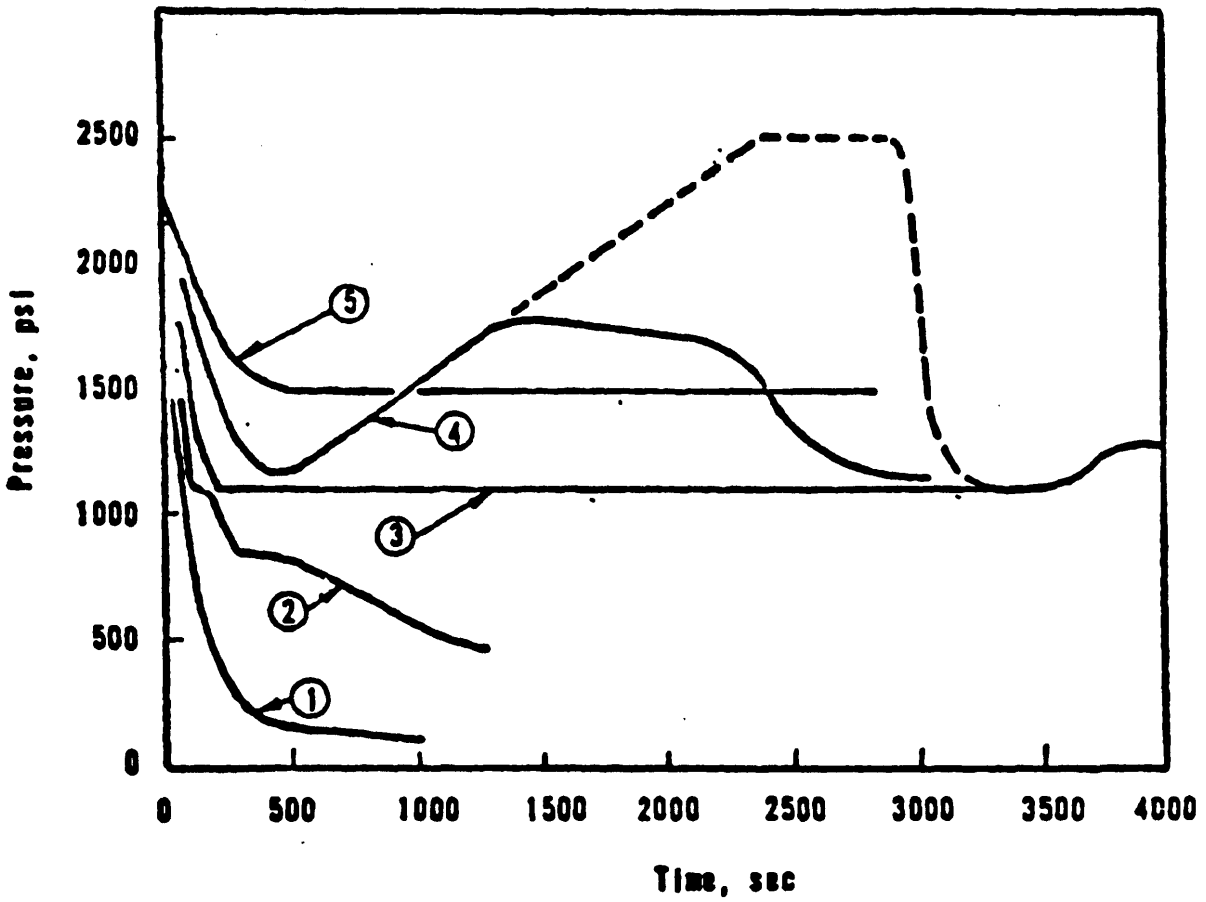
GPU NUCLEAR

BREAK SPECTRUM - AVERAGE SYSTEM VOID FRACTION WITH THE RC PUMPS OPERATIVE AND 2 HPI PUMPS

CAD FILE: 6602RI4.DWG

FIG 14.2-48

PRESSURE VS TIME-SMALL BREAKS WITH AUXILIARY FEEDWATER



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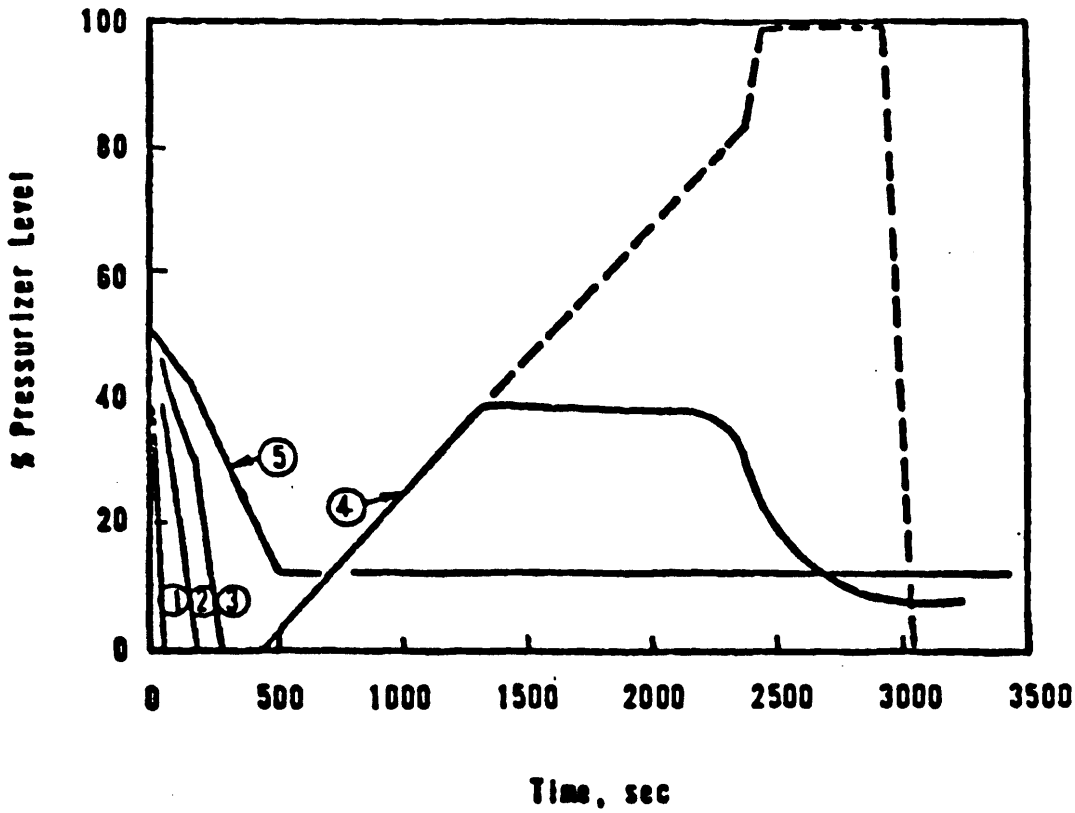
GPU NUCLEAR

PRESSURE VS TIME - SMALL BREAKS
WITH EMERGENCY FEED WATER

CAD FILE: 6603RI4.DWG

FIG 14.2-49

PRESSURIZER LEVEL VS TIME-SMALL BREAKS WITH AUXILIARY FEEDWATER



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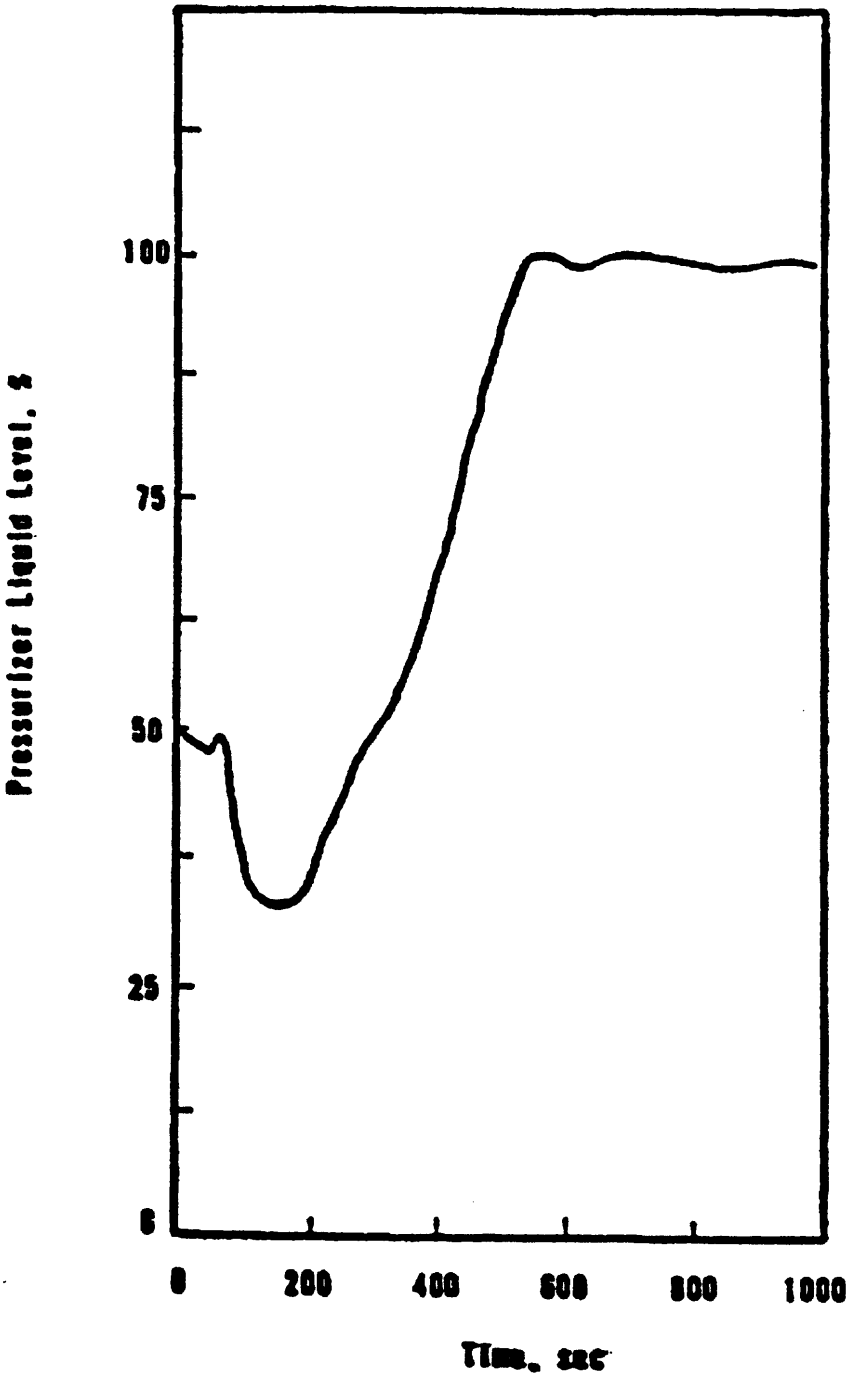
GPU NUCLEAR	
PRESSURE LEVEL VS TIME - SMALL BREAKS WITH EMERGENCY FEED WATER	
CAD FILE: 6604RI4.DWG	FIG 14.2-50

PRESSURIZER LEVEL VS TIME FOR SMALL BREAK IN PRESSURIZER



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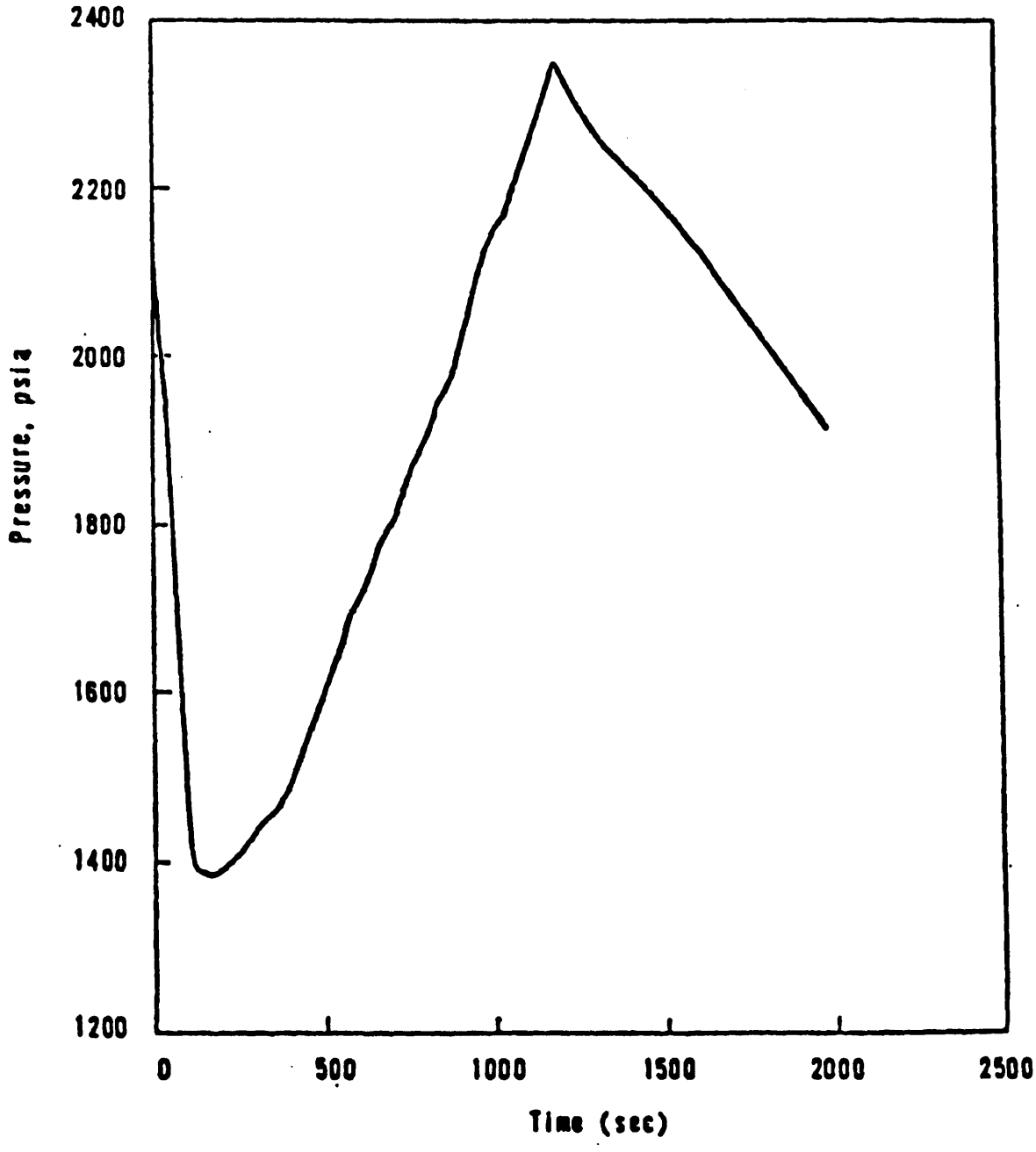
PRESSURIZER LEVEL VS. TIME - SMALL BREAK IN PRESSURIZER

CAD FILE: 6626RI4.DWG

FIG 14.2-51



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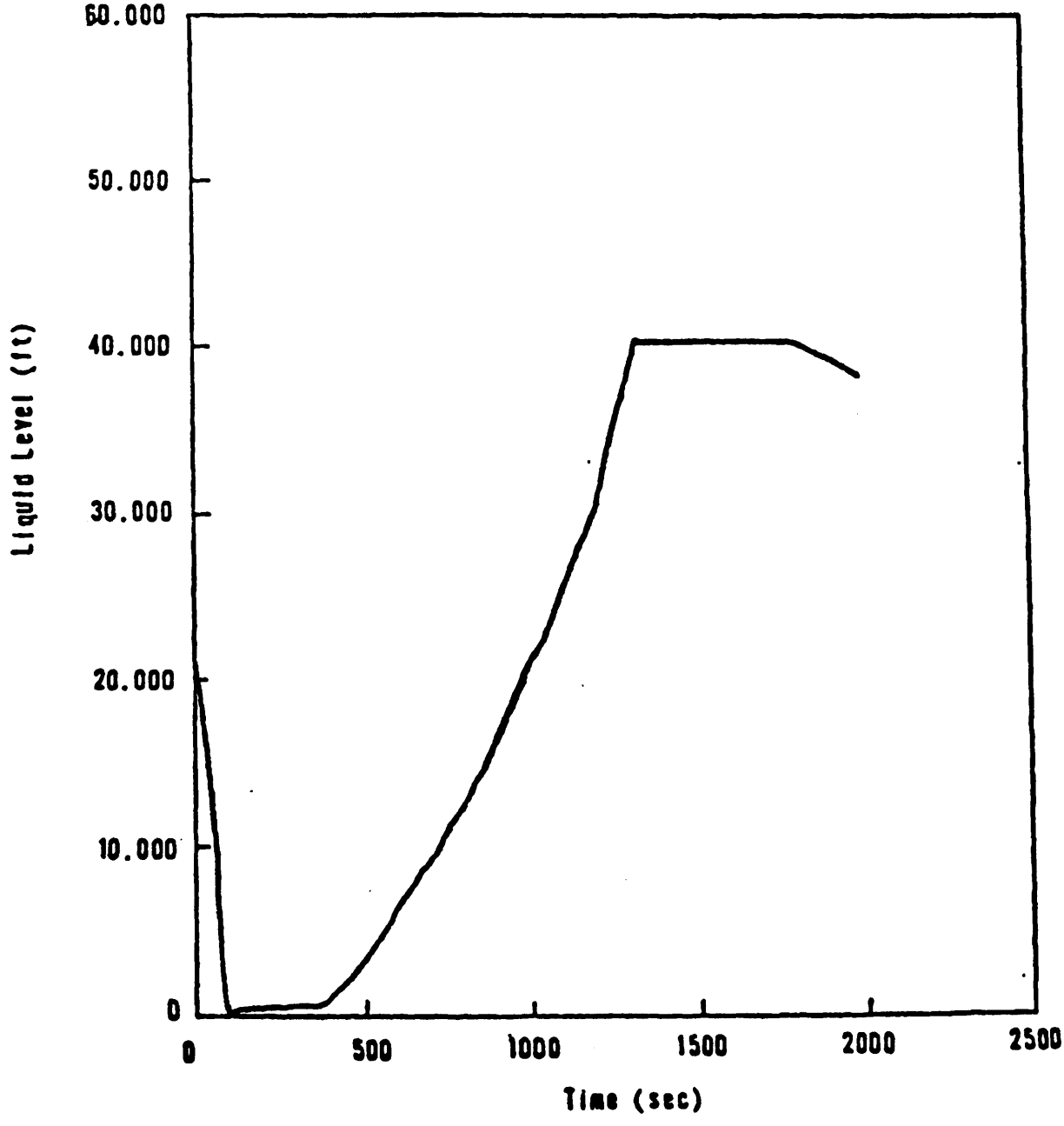
GPU NUCLEAR

0.01 FT^2 COLD LEG BREAK WITH NO EFW
2 HPI's & STUCK PORV AT 20 MIN-
NODE 14 PRESSURE VS. TIME

CAD FILE: 6627RI4.DWG

FIG 14.2-52



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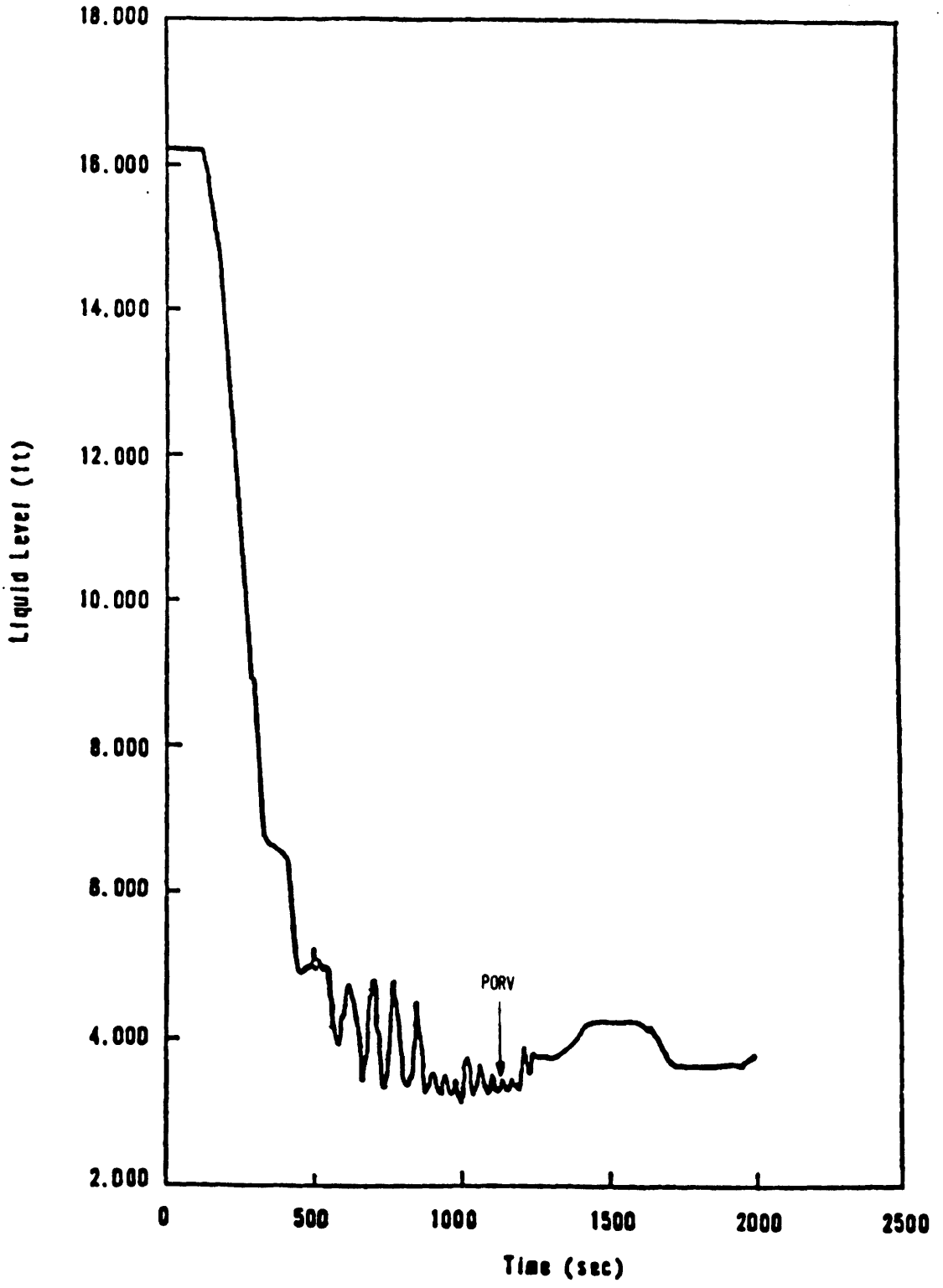
GPU NUCLEAR

0.01 FT^2 COLD LEG BREAK WITH NO EFW
2 HPI's & STUCK PORV AT 20 MIN-
PRESSURIZER LIQUID LEVEL

CAD FILE: 6628RI4.DWG

FIG 14.2-53



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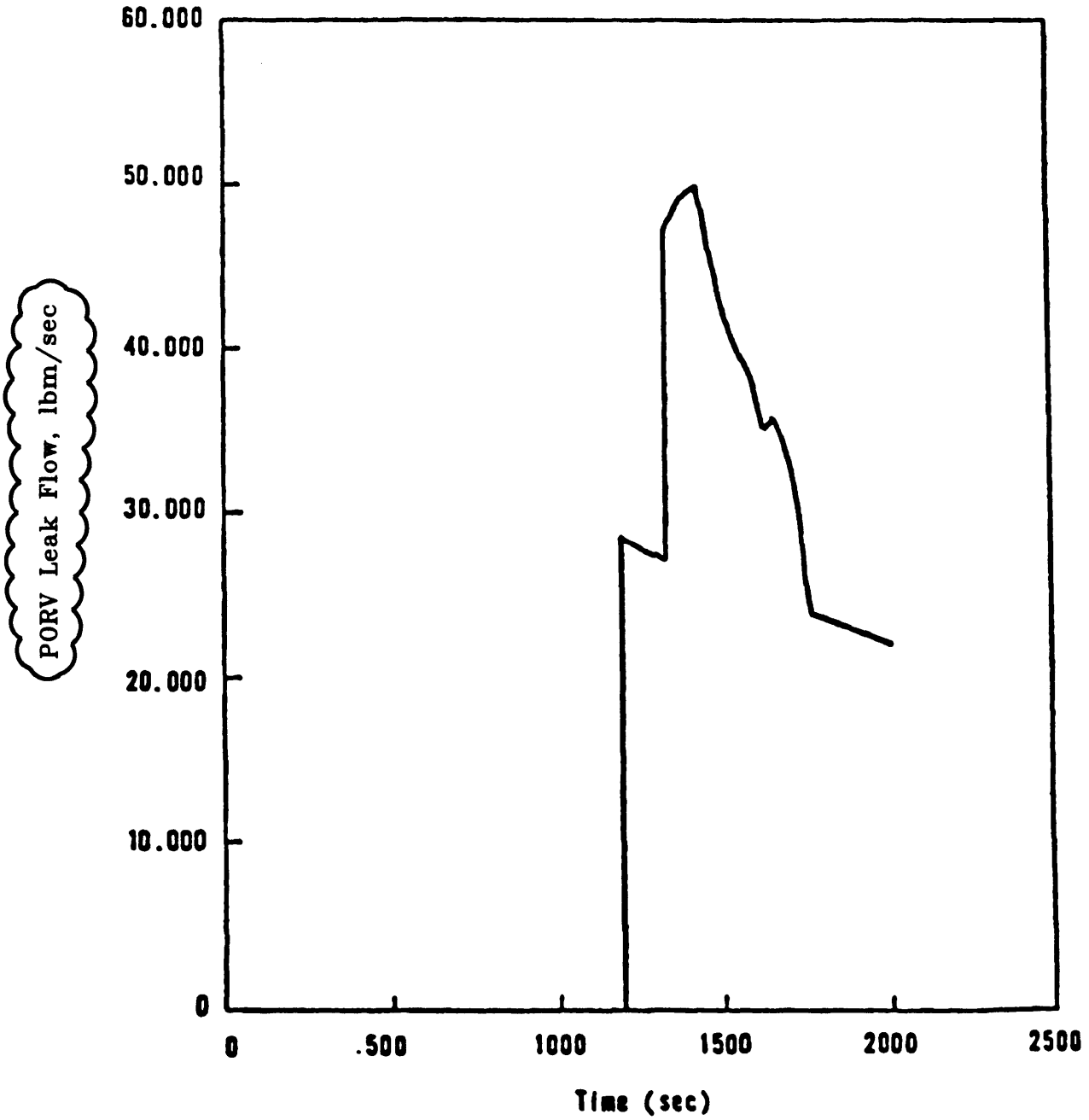
GPU NUCLEAR

0.01 FT^2 COLD LEG BREAK WITH NO EFW
2 HPI's & STUCK PORV AT 20 MIN-
UPPER PLENUM LIQUID LEVEL

CAD FILE: 6629RI4.DWG

FIG 14.2-54



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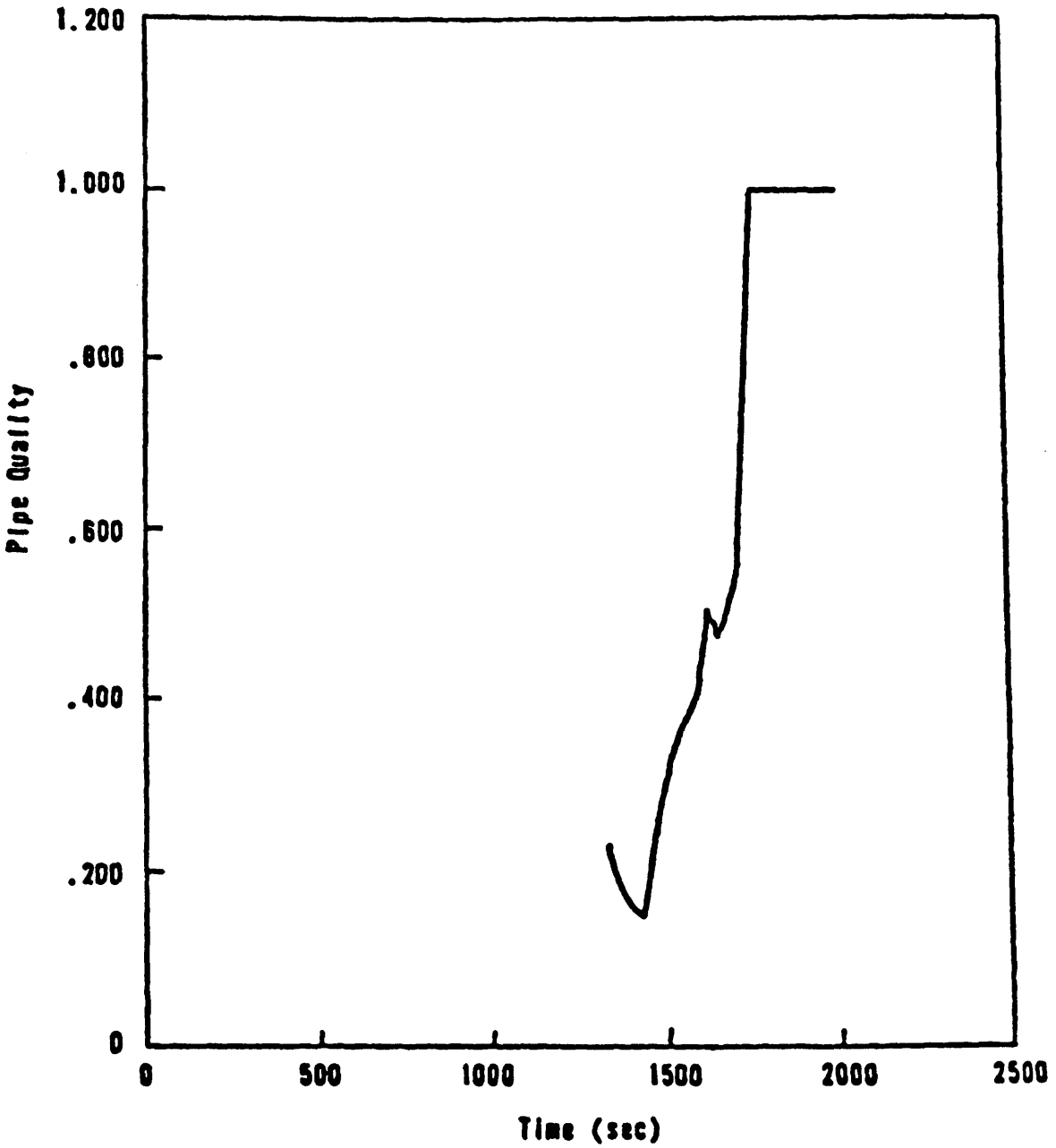
GPU NUCLEAR

0.01 FT^2 COLD LEG BREAK WITH NO EFW
2 HPI'S & STUCK PORV AT 20 MIN-
PORV LEAK FLOW

CAD FILE: 6613RI4.DWG

FIG 14.2-55



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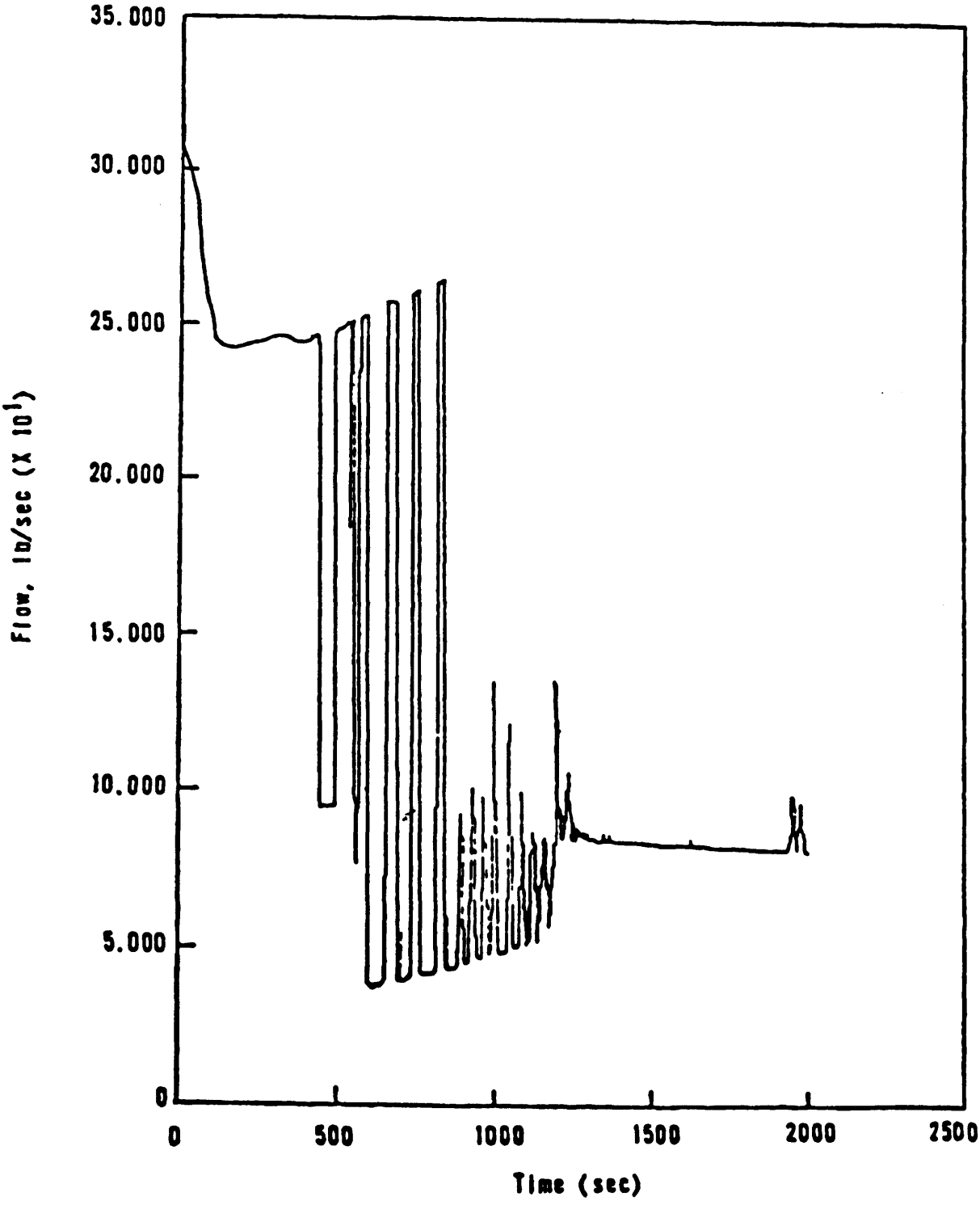
GPU NUCLEAR

0.01 FT^2 COLD LEG BREAK WITH NO EFW
2 HPI's & STUCK PORV AT 20 MIN-
PORV LEAK FLOW QUALITY

CAD FILE: 6614RI4.DWG

FIG 14.2-56



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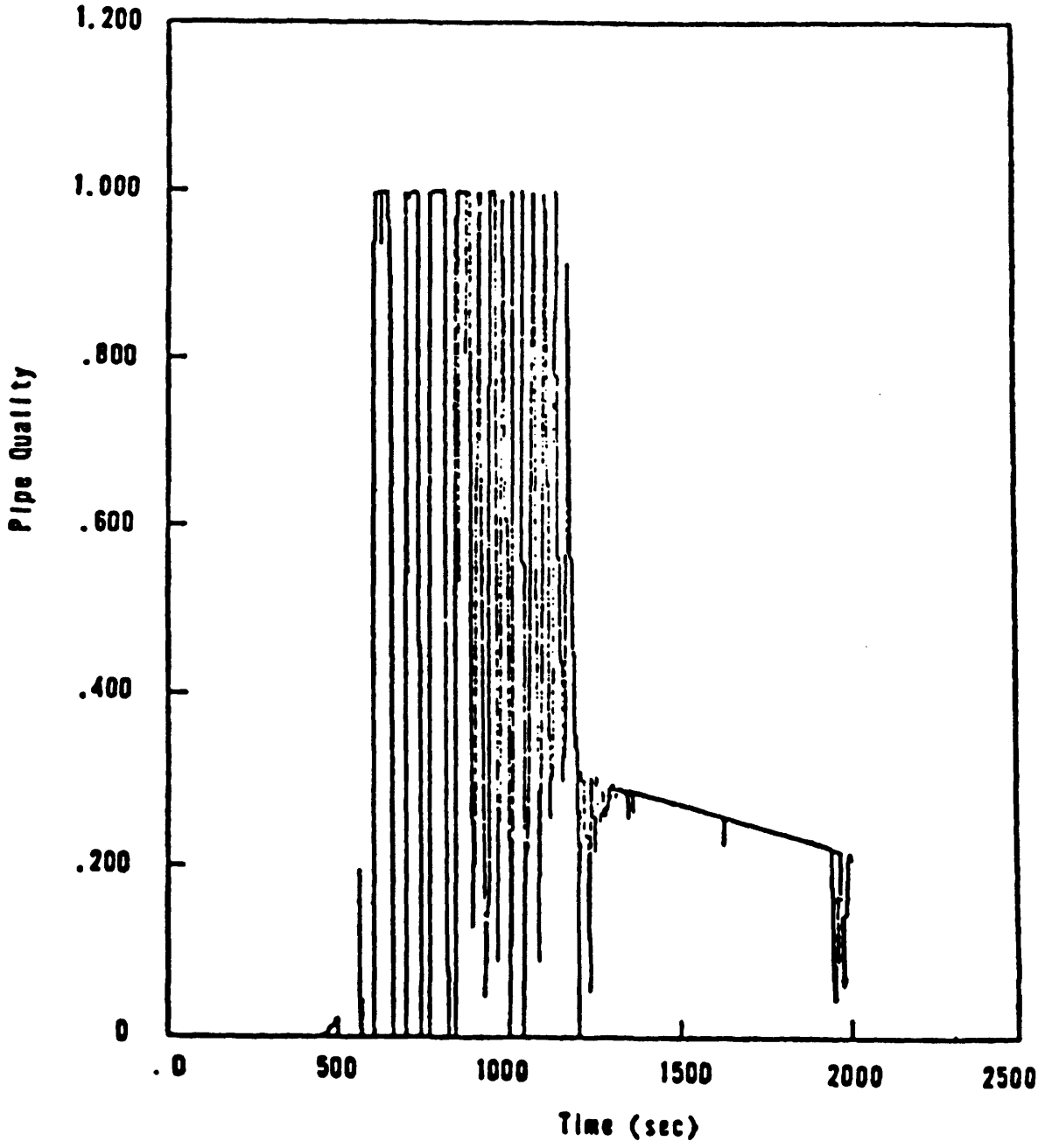
GPU NUCLEAR

0.01 FT^2 COLD LEG BREAK WITH NO EFW
2 HPI's & STUCK PORV AT 20 MIN-
COLD LEG BREAK FLOW

CAD FILE: 6615R14.DWG

FIG 14.2-57



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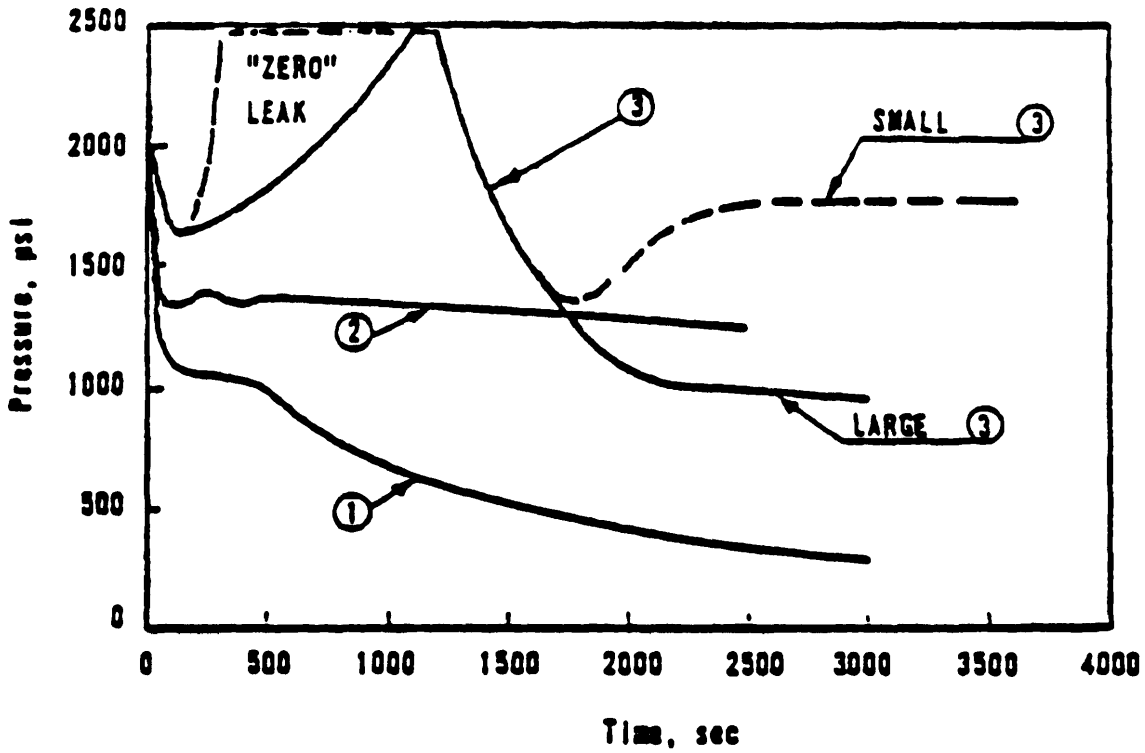
GPU NUCLEAR

0.01 FT^2 COLD LEG BREAK WITH NO EFW
2 HPI's & STUCK PORV AT 20 MIN-
COLD LEG BREAK LEAK FLOW QUALITY

CAD FILE: 6616RI4.DWG

FIG 14.2-58



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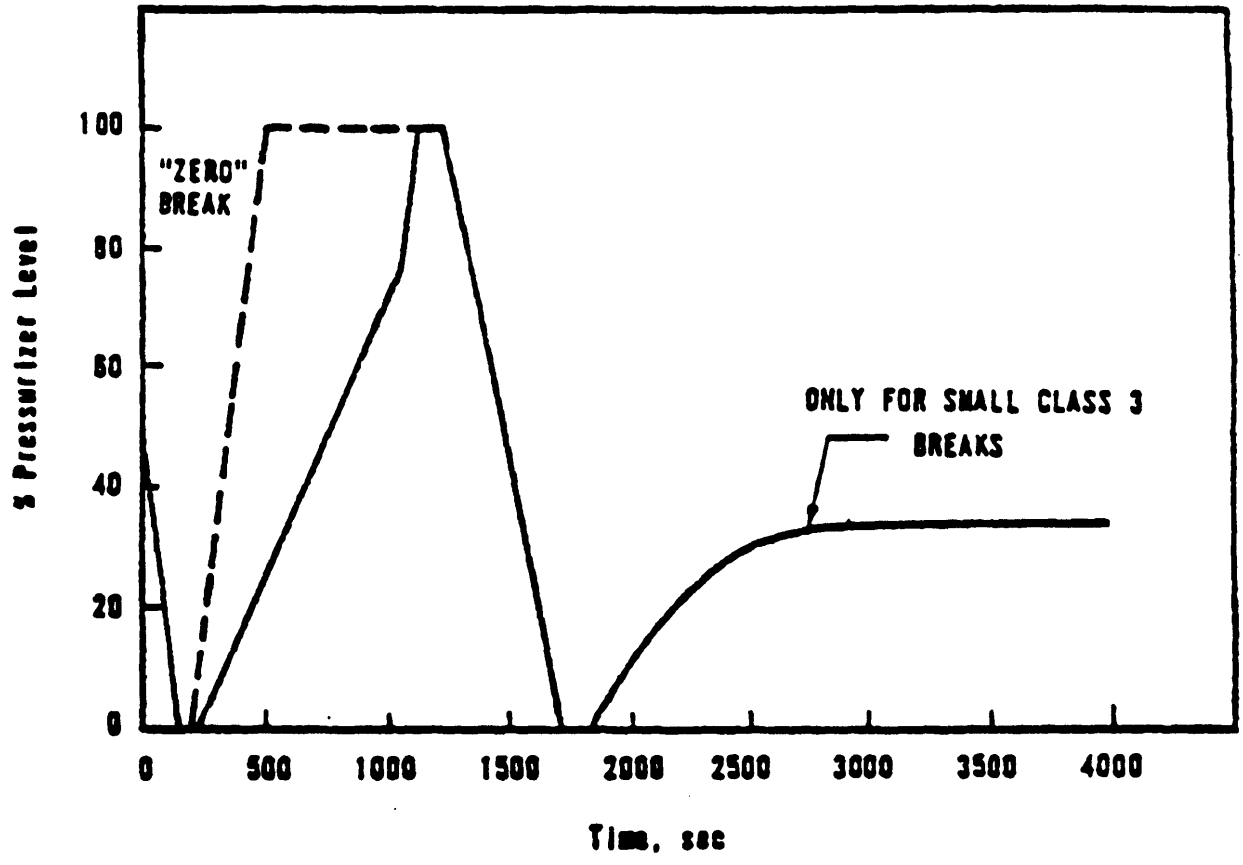
GPU NUCLEAR

SYSTEM PRESSURE VS. TIME - SMALL BREAKS W/O EMERGENCY FEEDWATER

CAD FILE: 6617RI4.DWG

FIG 14.2-59



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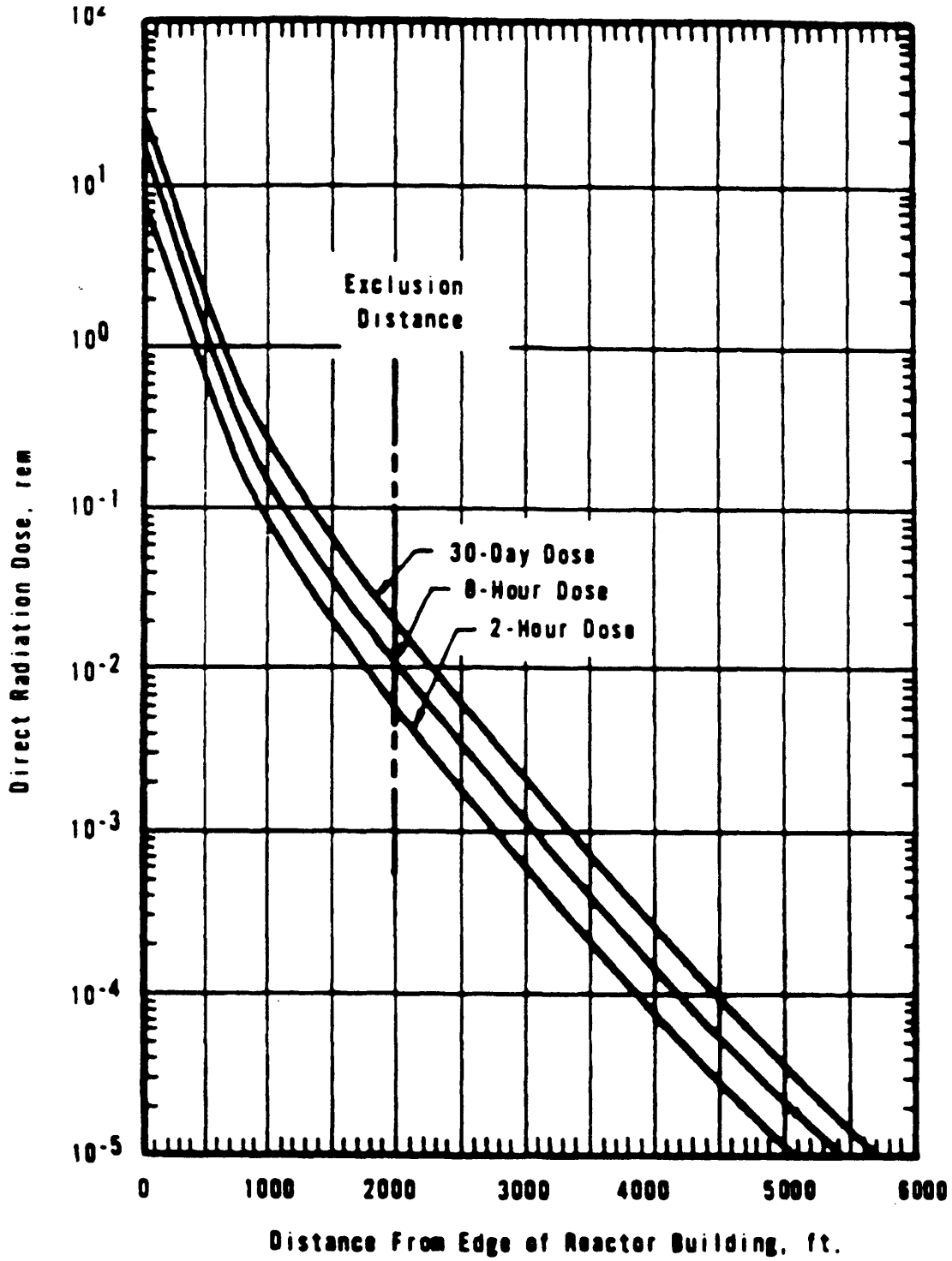
GPU NUCLEAR

PRESSURIZER LEVEL VS. TIME - CLASS 3 BREAKS W/O EMERGENCY FEEDWATER

CAD FILE: 6618RI4.DWG

FIG 14.2-60



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GPU NUCLEAR

INTEGRATED DIRECT DOSE FOLLOWING MHA WITH 3-1/2 FT. REACTOR BUILDING WALL THICKNESS

CAD FILE: 6619R14.DWG

FIG 14.2-61

Figure 14.2-62

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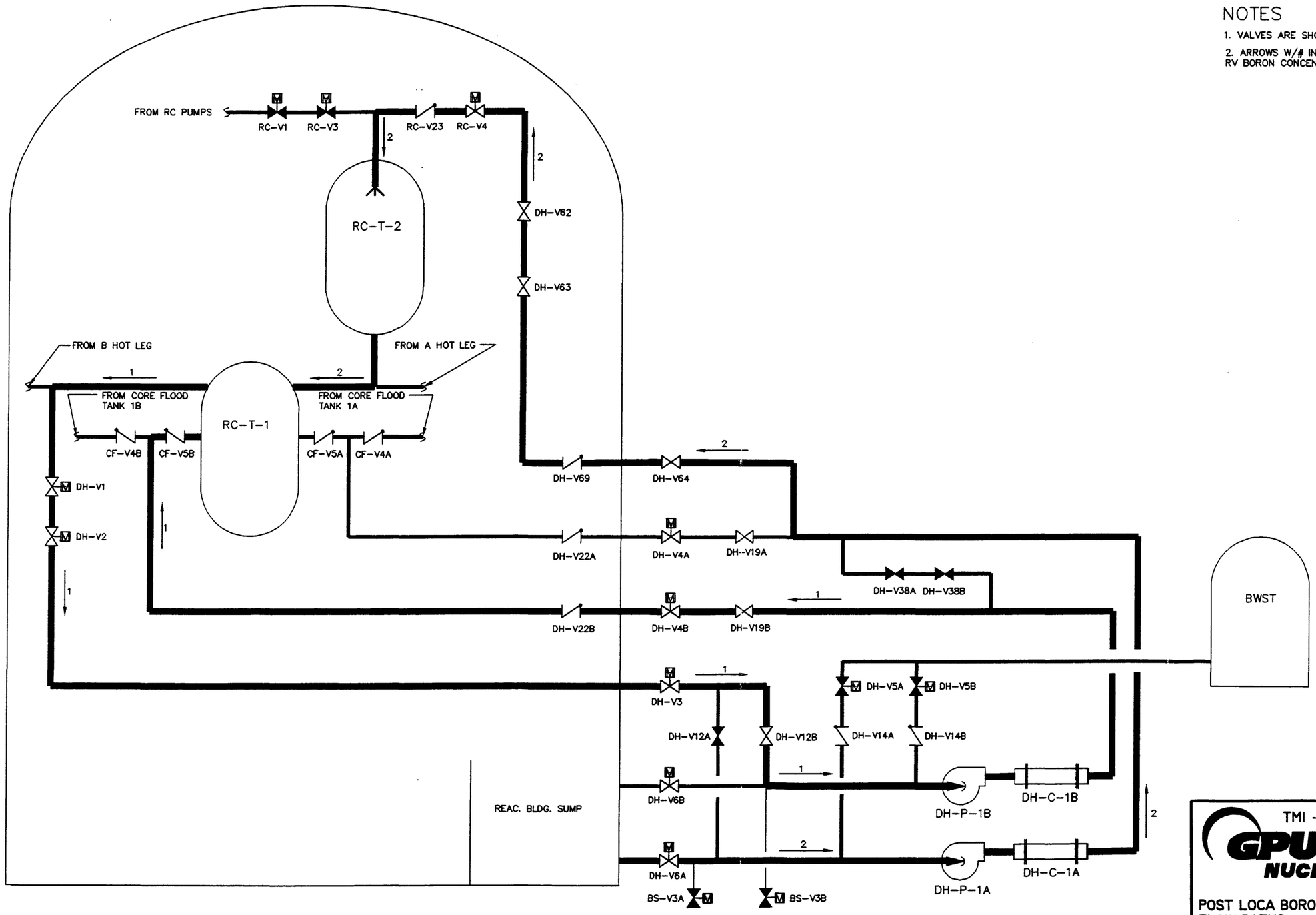

NOTES

1. VALVES ARE SHOWN IN TYPICAL POST LOCA POSITION
2. ARROWS W/# INDICATE TWO ACTIVE FLOW PATHS FOR RV BORON CONCENTRATION CONTROL

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GPU NUCLEAR

POST LOCA BORON CONCENTRATION CONTROL FLOW PATHS

SHEET 1 OF 1

CAD FILE: 6550R0.DWG

FIG 14.2-63