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12. CONDUCT OF OPERATIONS

12.1 Organization and Responsibility

This section covered the positions and personnel at the time of initial plant startup and operation. This information can be found in the original docketed FSAR and this is also addressed in the plant operating license (Technical Specifications).

12.2 Training

This section covered the training program at the time of initial plant startup and operation. This information can be found in the original docketed FSAR and this is also addressed in the Technical Specifications.

12.3 Procedures

The operating procedures for startup, normal operations, and anticipated emergency operating conditions is addressed in the original docketed FSAR and current requirements indicated in the Technical Specifications and in this chapter. The Emergency Plan in effect for Turkey Point is issued as a separate document.

12.4 Records

The procedure for maintaining plant operating, maintenance, QA, personnel, training, and instrumentation and control record is addressed in the original docketed FSAR and current requirements indicated in the Technical Specifications and in this chapter.

12.5 Administrative Control

The necessary administrative procedures are addressed in the original docketed FSAR and current requirements indicated in the Technical Specifications and in this chapter.

12.6 Plant Security Plan

Turkey Point maintains a Plant Security Plan and is issued as a separate document.

The FPL Quality Assurance Program is described by FPL Quality Assurance Topical Report (QATR) FPL-1. This corporate level document was supplemented by UFSAR Sections 12.7 through 12.10 which contained plant specific details of the FPL QA program related to Administrative Controls. These sections were originally located in the Technical Specifications and were relocated to the UFSAR by Technical Specification Amendment No. 201/195 (NRC Safety Evaluation report related to Amendment No. 201/195, dated October 6, 1999). These sections are subject to the regulatory requirements of 10 CFR 50.54 (a) with respect to changes to the approved QA program description. Sections 12.7, 12.8, and 12.9 were subsequently relocated from the UFSAR and placed into the QATR in accordance with the QATR Change Management Plan, NRC SER dated December 29, 2006, and Condition Report 2006-12055. C23

12.7 Deleted

12.8 Deleted

12.9 Deleted

12.10 Process Control Program (PCP)

12.10.1 The Process Control Program (PCP) shall contain the current formulas, sampling, analyses, tests and determinations to ensure that processing and packaging of solid radioactive wastes based on demonstrated processing of actual or simulated wet solid wastes will be accomplished in such a way as to assure compliance with 10 CFR Parts 20, 61, and 71, State regulations, burial ground requirements, and other requirements governing the disposal of solid radioactive waste.

12.10.2 Licensee-initiated changes to the PCP:

a. Shall be documented and records of reviews performed shall be retained as required by the QTAR. This documentation shall contain: C23

1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

2) A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.

b. Shall become effective after review and acceptance by the PNSC and the approval of the Plant Manager.

12.11 Emergency Planning

The Emergency Plan describes Florida Power & Light Company's plans for responding to emergencies that may develop at the Turkey Point Plant. The plan has been prepared to meet the requirements of 10 CFR 50.47(b), 10 CFR 50.72, and 10 CFR50 Appendix E. The purpose of this plan is to define and assign authority and responsibility in order to protect the health and safety of the public and plant personnel.

This plan applies to all plant emergencies which have resulted in, or which increase the risk of the accidental release of radioactive materials.

The Emergency Plan defines emergency conditions and delineates the responsibilities and duties of the FPL Emergency Response Organization. Associated with this Emergency Plan are implementing procedures which provide a detailed source of pertinent information and data required by the response organization during an emergency.

An Onsite Technical Support Center (TSC), an Onsite Operations Support Center (OSC), and an Offsite Emergency Operations Facility (EOF) have been established. Emergency support facilities meet the requirements of NUREG 0737, Item III.A.1.2 The Offsite Emergency Operations Facility (EOF) is located at the FPL General Office Building (9250 W. Flagler in Miami). This facility is approximately 25 miles north of Turkey Point station. The Technical Support Center is located in a separate building at the back of the property near the Circulating Water Inlet Bay. The Operations Support Center is located in the Maintenance Building.

12.12 Relocated Technical Specifications Requirements

This section of the UFSAR contains requirements (limiting conditions for operation, applicability, action statements, and surveillance requirements) that have been relocated from the Technical Specifications. License amendments approved removal of these requirements from the Technical Specifications on the basis that they do not meet the criteria in 10 CFR 50.36 for inclusion in the Technical Specifications. These relocated requirements are controlled under 10 CFR 50.59. The frequencies of the surveillance requirements included in this section may be extended up to 25% of the specified time interval. C28

12.12.1 Communications

The requirement for communications capability ensures that refueling station personnel can be promptly informed of significant changes in the facility status or core reactivity conditions during CORE ALTERATIONS.

Limiting Condition for Operation:

Direct communications shall be maintained between the control room and personnel at the refueling station.

Applicability:

During CORE ALERTATIONS

Action:

When direct communications between the control room and personnel at the refueling station cannot be maintained, suspend all CORE ALTERATIONS. C28

Surveillance Requirements:

Direct communications between the control room and personnel at the refueling station shall be demonstrated within 1 hour prior to the start of and in accordance with the Surveillance Frequency Control Program during CORE ALTERATIONS.

12.12.2 Manipulator Crane

The requirements for the manipulator cranes ensure that: (1) manipulator cranes will be used for movement of drive rods and fuel assemblies, (2) each crane has sufficient load capacity to lift a drive rod or fuel assembly, and (3) the core internals and reactor vessel are protected from excessive lifting force in the event they are inadvertently engaged during lifting operations.

The requirement that the auxiliary hoist load indicator be used to prevent lifting excessive loads will require a manual action. The auxiliary hoist load indicator does not include any automatic mechanical or electrical interlocks that prevent lifting loads in excess of 600 pounds.

Limiting Condition for Operation:

The manipulator crane and auxiliary hoist shall be used for movement of drive rods or fuel assemblies and shall be OPERABLE with:

a. The manipulator crane used for movement of fuel assemblies having:

   1) A minimum capacity of 2750 pounds, and

   2) An overload cutoff limit less than or equal to 2700 pounds.

b. The auxiliary hoist used for latching and unlatching drive rods having:

   1) A minimum capacity of 610 pounds, and

   2) A load indicator which shall be used to prevent lifting loads in excess of 600 pounds.

Applicability:

During movement of drive rods or fuel assemblies within the reactor vessel.

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Action:

With the requirements for crane and/or hoist OPERABILITY not satisfied, suspend use of any inoperable manipulator crane and/or auxiliary hoist from operations involving the movement of drive rods and fuel assemblies within the reactor vessel.

Surveillance Requirements:

1. At least once each refueling, each manipulator crane used for movement of fuel assemblies within the reactor vessel shall be demonstrated OPERABLE within 100 hours prior to the start of such operations by performing a load test of at least 2750 pounds and demonstrating an automatic load cutoff when the crane load exceeds 2700 pounds.

2. At least once each refueling, each auxiliary hoist and associated load indicator used for movement of drive rods within the reactor vessel shall be demonstrated OPERABLE within 100 hours prior to the start of such operations by performing a load test of at least 610 pounds.