NRC FORM 366 U. S. NUCLEAR REGULATORY COMMISSION (7-77) LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) $\left| \left(1 \right) \right|$ V A S P S 1 2 0 0 -0 (5) CON'T REPORT 0 7 0 6 2 2 7 9 8 0 7 0 5 68 69 EVENT DATE 74 75 REPORT DATE 0 5 0 0 0 8 0 1 2 7 9 (9) SOURCE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) |Radiography of Surry Unit 1 Steam Generator Feedwater Nozzles and Reducers revealed 0 2 crack indications in the base metal at the counterbore regions of all three steam 0 3 generators. One weld metal crack was also revealed. Unit 1 is at cold shutdown 0 4 11nit The health and safety of the general public were not affected. 2 is defueled. 0 5 This event is reportable per Technical Specification 6.6.2.a.9. 0 6 0 7 8 SYSTEM CAUSE CAUSE COMP. VALVE CODE COMPONENT CODE CODE SUBCODE SUBCODE SUBCODE C (13) P = |X| |X| (14)(15) H E (12)Ρ Τ D Z (16) 12 13 18 19 SEQUENTIAL OCCURRENCE REPORT REVISION REPORT NO. CODE EVENT YEAR TYPE LER/RO NO. 9 (17) 0 2 1 3 0 Т REPORT 1 0 NUMBER 27 28 26 32 ATTACHMENT SUBMITTED NPRD-4 ACTION FUTURE TAKEN ACTION EFFECT ON PLANT SHUTDOWN METHOD PRIME COMP. COMPONENT HOURS (22) FORM SUB. SUPPLIER MANUFACTURER Z (20) Z (21 0 0 Y (23) (24 (25) 0 0 N N (26) W 36 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The cause of these cracks is undetermined at this time. The immediate corrective 1 0 action was to submit a station deviation report and contact Westinghouse to determine 1 1 necessary corrective actions. 1 2 1 3 14 9 80 FACILITY OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32) % POWER (31) 5 0 + 00 Specia 8 80 ACTIVITY CONTENT AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE RELEASED_OF RELEASE 33 <u>Z</u>34 6 Z N/ N 10 45 ΔΔ 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE <u>Z</u> (38) 10 (37) 7 0 10 NA 11 12 PERSONNEL INJURIES 80 DESCRIPTION (41) NUMBER 0 0 (40) NA 11 12 80 LOSS OF OR DAMAGE TO FACILITY (43) 7907090 384 S DESCRIPTION TYPE 9 Z (42) NA 10 80 PUBLICITY NRC USE ONLY DESCRIPTION (45 ISSUED (44) Various-Beginning 6/24/79 0 68 59 10 80 804-357-3184 W. L. Stewart NAME OF PREPARER -PHONE:

Surry Power Station, Unit 1 Docket No.: 50-280 Report No.: 79-023/01T-0 Event Date: 6-22-79

Title: Cracking in Steam Generator Feedwater Nozzles and Reducers

1. Description of Event:

Radiography of Surry Unit 1 Steam Generator Feedwater Nozzles and Reducers revealed circumferential crack indications in the base metal at the nozzle to reducer counterbore regions of all three steam generators. One weld metal crack was also revealed. Unit 1 is at cold shutdown, Unit 2 is defueled. The health and safety of the general public were not affected. This event is reportable per Technical Specification 6.6.2.a.9.

2. Probable Consequences and Status of Redundant Systems:

Since the defects were discovered with both units shutdown, and any necessary repairs will be made prior to unit operation, there are no consequences from this event.

3. Cause of Event:

The cause of these cracks is undetermined at this time.

4. Immediate Corrective Action:

The immediate corrective action was to submit a station deviation report and contact Westinghouse to determine necessary corrective actions.

5. Subsequent Corrective Action:

Replacement reducers have been ordered. Vepco and Westinghouse are continuing to evaluate required corrective measures.

6. Actions Taken to Prevent Recurrence:

Actions necessary to prevent recurrence are being evaluated at this time.

7. Generic Implications:

This problem may be generic to all PWR's.