

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	V	A	S	P	S	1	2	0	0	-	0	0	0	0	0	0	-	0	0	3	4	1	1	1	0	4	5
7	8	9	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39
		LICENSEE CODE						LICENSE NUMBER										LICENSE TYPE					CAT 58					

CONT

0	1	6	0	5	0	0	0	2	8	0	7	0	6	2	1	7	9	8	0	7	0	5	7	9	9
7	8	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80	81	82	83
		REPORT SOURCE			DOCKET NUMBER						EVENT DATE						REPORT DATE								

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | On 6/21/79, Westinghouse notified Vepco of a non-conservatism in the accident analysis.

0 3 | Specifically, the increase in steam generator reference leg temperature resultant from

0 4 | a rise in containment temperature was not properly accounted for. Consequently,

0 5 | safeguards actuation may be delayed after a high energy line break. This event is

0 6 | reportable as per Technical Specifications 6.6.2.a.8. The health and safety of the

0 7 | general public were not affected.

0	9	C	H	B	A	I	N	S	T	R	U	T	Z	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	
		SYSTEM CODE		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE						COMP. SUBCODE	VALVE SUBCODE	

17	7	9	—	0	2	2	—	0	1	T	—	0
21	22	23	24	25	26	27	28	29	30	31	32	33
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.		

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | Unit 1 is at cold shutdown, Unit 2 is defueled. The event was caused by a deficiency

1 1 | in the accident analysis. The corrective action is being evaluated by Westinghouse

1 2 | and Vepco and will be reported in a later revision to this report. Any required cor-

1 3 | rective action will be completed prior to returning either unit to service. Westing-

1 4 | house has submitted a 10CFR21 Report.

1	5	G	0	0	0	U-1 S/D, U-2 Defueled	D	NSSS Notification
7	8	9	10	11	12	13	14	15
FACILITY STATUS		% POWER			OTHER STATUS		METHOD OF DISCOVERY	

1	6	Z	Z	NA	NA
7	8	9	10	11	12
ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY	

1	7	0	0	0	Z	NA
7	8	9	10	11	12	13
PERSONNEL EXPOSURES NUMBER		TYPE		DESCRIPTION		

1	8	0	0	0	NA
7	8	9	10	11	12
PERSONNEL INJURIES NUMBER		DESCRIPTION			

1	9	Z	NA
7	8	9	10
LOSS OF OR DAMAGE TO FACILITY TYPE		DESCRIPTION	

2	0	N	NA
7	8	9	10
PUBLCITY ISSUED		DESCRIPTION	

7907090380

NAME OF PREPARER: W. L. Stewart

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NRC USE ONLY

Surry Power Station, Unit 1
Docket No: 50-280
Report No: 79-022/01T-0
Event Date: 06-21-79

Attachment Page 1 of 1

Title of Report: Non-Conservatism in Accident Analysis

1. Description of Event:

On 6-21-79, Westinghouse notified VEPCO of a non-conservatism in the accident analysis. Specifically, the potential increase in the steam generator reference leg temperature resultant from a rise in containment temperature was not properly accounted for in the analysis. The increased reference leg water temperature will result in a decrease of the water column density which gives an apparent increase in steam generator level. This event is reportable as per Technical Specifications 6.6.2.a.8.

2. Probable Consequences and Status of Redundant Systems:

This potential level bias could result in delayed protection signals which are based on Low-Low steam generator water level. Backup signals were available from the Overtemperature Delta T, pressurizer pressure, containment pressure and safety injection systems. Therefore, the health and safety of the general public were not affected.

3. Cause:

This event was caused by the identification of a deficiency in the accident analysis for High Energy Line Break.

4. Immediate Corrective Action:

The corrective action is being evaluated by Westinghouse and VEPCO. Westinghouse has prepared and submitted a 10 CFR 21 review. Any required corrective action will be completed prior to returning either unit to service.

5. Subsequent Corrective Action:

Unknown at this time.

6. Actions Taken to Prevent Recurrence:

Unknown at this time.

7. Generic Implications:

This error may be generic to Westinghouse PWR's.