NRC FORM 366
(7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | V A S P S 1 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T
01 | REPORT SOURCE L (6) | 0 5 0 0 0 2 8 0 (7) | 1 2 2 0 7 8 (8) | 0 1 0 2 7 9 (9)
DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 12-20-78 Westinghouse notified Vepco of an input error in the currently applicable
03 LOCA-ECCS analysis. The error was estimated to result in an increase in peak clad
04 temperature of approximately 20°F. Further investigation revealed that the analysis
05 input was correct but a modeling methodology which was slightly different from the
06 methodology applied in the NRC approved Westinghouse ECCS Evaluation Model (Feb. 1978
07 version) was being used. This is reportable per Tech. Spec. 6.6.2.a.(8). The
08 health and safety of the public were not affected.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
09	X X (11)	X (12)	X (13)	Z Z Z Z Z Z (14)	Z (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
7 8	—	0 5 0	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	X (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	Z (25)	Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The margin to the Technical Specification limit on F_Q and to the 10 CFR 50.46 limits
11 was assessed and found to be sufficiently conservative to accomodate the above non-
12 conservatism. A new analysis with the February 1978 Westinghouse LOCA-ECCS evaluation
13 model, which includes more conservative modeling methodology was performed and has
14 been submitted to the Nuclear Regulatory Commission for approval.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
15	G (28)	0 0 0 (29)	NA	D (31)	NSS Vendor Notification

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
16	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
17	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
18	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
19	Z (42)	NA

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
20	N (44)	NA	

7901080191

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GPO 917-926

Surry Power Station, Unit 1
Docket No: 50-280
Report No: 78-050/01T-0
Title of Report: LOCA-ECCS Analysis

1. Description of Event:

On 12-20-78 Westinghouse notified Vepco of an input error in the currently applicable LOCA-ECCS analysis. The error was estimated to result in an increase in peak clad temperature of approximately 20°F. Further investigation revealed that the analysis input was correct but a modeling methodology which was slightly different from the methodology applied in the NRC approved Westinghouse ECCS Evaluation Model (Feb. 1978 version) was being used.

2. Probable Consequences:

Because there is sufficient margin in the currently applicable LOCA-ECCS to accomodate this non-conservatism, there are no probable consequences of this event.

3. Cause of Event:

The cause of this event is an alternate modeling methodology used in analyzing containment pressure response effects resulting from the broken loop accumulator flow.

4. Immediate Corrective Actions:

The margin to the Technical Specification Limit on F_Q and the 10 CFR 50.46 limits was assessed and found to be sufficiently conservative to accomodate the above non-conservatism.

5. Subsequent Corrective Actions:

A new analysis performed with the February 1978 Westinghuse LOCA-ECCS evaluation model, which includes more conservative modeling methodology has been submitted to the Nuclear Regulatory Commission for approval.

6. Actions Taken to Prevent Recurrence:

None necessary.

7. Generic Implications:

None.