NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

DOCKET NUMBER

FILE NUMBER INCIDENT REPORT

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO:	FROM:	DATE OF DOCUMENT
		DATE RECEIVED

☐ LETTER ☐ ORIGINAL ☐ COPY	☐ NOTORIZED ☐ UNCLASSIFIED	PROP	INPUT FORM	NUMBER OF COPIES RECEIVED

DESCRIPTION

PLANT NAME:

ENCLOSURE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:	Ziemann
W/3 CYS FOR ACTION	
LIC. ASST.:	Diss
W/ 1 CYS	
ACRS 16 CYS ~~HOLDING~~/SENT	As CAT B

INTERNAL DISTRIBUTION

REG FILE		
NRC PDR		
I & E (2)		
MIPC		
SCHROEDER/IPPOLITO		
HOUSTON		
NOVAK/CHECK		
GRIMES		
CASE		
BUTLER		
HANAUER		
TEDESCO/MACCARY		
EISENHUT		
BAER		
SHAO		
VOLLMER/BUNCH		
KREGER/J. COLLINS		

EXTERNAL DISTRIBUTION

		CONTROL NUMBER
LPDR: Ft Pierce, Fla		7707260368
TIC:		A04 60
NSIC:		

NRC FORM 195 (2-76)

NRC FORM 195 (2-76) U.S. NUCLEAR REGULATORY COMMISSION

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

DOCKET NUMBER: 50-335

FILE NUMBER:

TO: Mr. Norman C. Moseley

FROM: Florida Power & Light Company, Miami, Florida, A. D. Schmidt

DATE OF DOCUMENT: 3/4/77

DATE RECEIVED: 3/16/77

☒ LETTER ☐ NOTORIZED
☒ ORIGINAL ☒ UNCLASSIFIED
☐ COPY

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED: One signed

DESCRIPTION

Ltr. trans the following:

DO NOT REMOVE

ACKNOWLEDGED

PLANT NAME: (1-P)

St. Lucie Unit No. 1

RJL

ENCLOSURE

Licensee Event Report (RO 50-335-77-6) on 2/4/77 concerning Control Element Assembly #60 dropping, initiating a turbine run back and giving a "rod bottomed" indication.....

(2-P)

SAFETY		FOR ACTION/INFORMATION	ENVIRO
ASSIGNED AD:		ASSIGNED AD:	
BRANCH CHIEF:		BRANCH CHIEF:	
PROJECT MANAGER:		PROJECT MANAGER:	
LIC. ASST.:		LIC. ASST.:	

INTERNAL DISTRIBUTION

REG FILE	SYSTEMS SAFETY	PLANT SYSTEMS	SITE SAFETY &
NRC PDR	HEINEMAN	TEDESCO	ENVIRO ANALYSIS
I & E	SCHROEDER	BENAROYA	DENTON & MULLER
OELD		LAINAS	
GOSSICK & STAFF	ENGINEERING	IPPOLITO	ENVIRO TECH.
MIPC	MACARRY	KIRKWOOD	ERNST
CASE	BOSNAK		BALLARD
HANAUER	SIHWEIL	OPERATING REACTORS	YOUNGBLOOD
HARLESS	PAWLICKI	STELLO	
			SITE TECH.
PROJECT MANAGEMENT	REACTOR SAFETY	OPERATING TECH.	GAMMILL
BOYD	ROSS	EISENHUT	STEPP
P. COLLINS	NOVAK	SHAO	HULMAN
HOUSTON	ROSZTOCZY	BAER	
PETERSON	CHECK	BUTLER	SITE ANALYSIS
MELTZ		GRIMES	VOLLMER
HELTEMES	AT & I		BUNCH
SKOVHOLT	SALTZMAN		J. COLLINS
	RUTBERG		KREGER

EXTERNAL DISTRIBUTION

			CONTROL NUMBER
LPDR:	NAT. LAB:	BROOKHAVEN NAT. LAB.	
TIC:	REG V.IE	ULRIKSON (ORNL)	
NSIC:	LA PDR		
ASLB:	CONSULTANTS:		770760368
ACRS CYS HOLDING/SENT			

NRC FORM 195 (2-76)

P. O. BOX 013100, MIAMI, FL 33101

FLORIDA POWER & LIGHT COMPANY

March 4, 1977

PRN-LI-77-59

Mr. Norman C. Moseley, Director, Region II
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
230 Peachtree Street, N. W., Suite 1217
Atlanta, Georgia 30303

Dear Mr. Moseley:

REPORTABLE OCCURRENCE 335-77-6
ST. LUCIE UNIT 1
DATE OF OCCURRENCE: FEBRUARY 4, 1977

CEA 60

The attached Licensee Event Report is being submitted in accordance with Technical Specification 6.9 to provide 30-day notification of the subject occurrence.

Very truly yours,

A. D. Schmidt
Vice President
Power Resources

MAS/cpc

Attachment

cc: Robert Lowenstein, Esquire
Director, Office of Inspection and Enforcement (30)
Director, Office of Management Information and
Program Control (3)

7706036P

LICENSEE EVENT REPORT

CONTROL BLOCK: [ ] (1–6) (PLEASE PRINT ALL REQUIRED INFORMATION)

	LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01	FLSLS 1	00-0000-00	4111	03
(7–8, 9–14)		(15–25)	(26–30)	(31–32)

	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 CONT		L	L	050-0335	020477	030477
(7–8)	(57–58)	(59)	(60)	(61–68)	(69–74)	(75–80)

EVENT DESCRIPTION

02 With the plant operating at power, Control Element Assembly (CEA) #60 dropped,
03 initiating a turbine run back and giving a "rod bottomed" indication. During the
04 runback transient, T_{cold} exceeded the 542°F limit of Specification 3.2.5 by approxi-
05 mately 1°F but, within several minutes, was returned to within the limit. As required
06 by Specification 3.1.3.1.e.1 for a CEA out of alignment, reactor power was reduced to

	SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07	RB	F	CONROD	N	C490	N
(7–8, 9–10)		(11)	(12–17)	(43)	(44–47)	(48)

CAUSE DESCRIPTION

08 The cause is unknown. Investigation of the control circuits showed no loose connec-
09 tions, damage, or out of tolerance voltages. During realignment of this CEA, its
10 operation was closely monitored by Instrumentation and Control Personnel and no

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11	B	089	NA	A	NA
(7–8)	(9)	(10–12)	(13–44)	(45)	(46–80)

	FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12	Z	Z	NA	NA
(7–8)	(9)	(10)	(11–44)	(45–80)

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
13	000	Z	NA
(7–8)	(9–11)	(12)	(13–80)

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
14	000	NA
(7–8)	(9–11)	(12–80)

PROBABLE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
16	Z	NA
(7–8)	(9)	(10–80)

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 See page 2 for continuation of Event and Cause Descriptions.

19

NAME: M. A. Schoppman PHONE: 305/552-3779

GPO 861-667

EVENT DESCRIPTION (Continued)

<70% and the CEA was restored to its proper position within one hour. As a result of the reactivity insertion of the dropped CEA, azimuthal power tilt (T_q) increased. As required by Specification 4.2.2.2.c, total radial peaking factor (F_r^T) was determined to be within its limit within 4 hours. After 6 hours, T_q returned to normal.

This was the second "dropped rod" occurrence (LER 335-76-37), but the first one was due to a faulty power supply. The two events are not considered to be generically related. (335-77-6).

CAUSE DESCRIPTION (Continued)

abnormalities were observed. There have been no further instances of dropping CEA #60 since the February 4 event.