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Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

LR-N99-0416

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT
SALEM UNIT NO. 2
DOCKET NO. 50-311**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for August 1999 is attached. As indicated in the Salem Unit 2 July Monthly Operating Report, the summary of safety evaluations for design changes implemented during July and August 1999 are included in this report.

Sincerely,

M. B. Bezilla
Vice President - Operations

9909170015 990831
PDR ADDCK 05000311
R PDR

/rbk
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

JE24/1

The power is in your hands.

170010

DOCKET NO.: 50-311
 UNIT: Salem 2
 DATE: 9/15/99
 COMPLETED BY: R. Knieriem
 TELEPHONE: (856) 339-1782

Reporting Period: August 1999

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
 Maximum Dependable Capacity (MWe-Net)

No. of hours reactor was critical
 No. of hours generator was on line (service hours)
 Unit reserve shutdown hours
 Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
744	4558	93176
744	4502	89853
0.0	0.0	0.0
787038	4742738	89715569

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/COMMENT

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

Summary:

Salem Unit 2 began the month of August 1999 operating at full power. Full power operation continued until August 21, when power was reduced to 37% to perform Turbine Valve Testing and to effect repairs to the 23RS6, Turbine Intercept Valve. Salem Unit 2 returned to full power on August 22, and continued full power operation through the end of the month.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM UNIT 2 GENERATING STATION

MONTH: August 1999

The following items completed during **August 1999** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant; nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes - Summary of Safety Evaluations

Modification 2EC-3654, Pkg. 1, Auxiliary Annunciator (AAS) Replacement Project

This modification implemented design changes to the AAS to address Year 2000 concerns. The modification included replacement of Rochester Instrument Systems (RIS) 3800 series equipment and replacement of an existing typewriter/printer unit.

Review of this modification under 10CFR50.59 was required because the replacement of the AAS equipment constituted a change to the facility as described in the UFSAR. This evaluation concluded that this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Temporary Modifications - Summary of Safety Evaluations

There were no changes in this category implemented during August 1999.

Procedures - Summary of Safety Evaluations

There were no changes in this category implemented during August 1999.

UFSAR Change Notices - Summary of Safety Evaluations

UFSAR Change Notice SCN99-048, Reactor Coolant System Chemical Degassing Operation During Shutdown

This UFSAR change involved the addition of hydrogen peroxide to the Reactor Coolant System (RCS) following plant shutdown and cooldown for the purpose of chemically depleting RCS dissolved hydrogen inventory. Chemical degassing will occur only in operational Mode 5 and below.

Review of this temporary modification under 10CFR50.59 was required because chemical degassing of the RCS constituted a change to the facility as described in the UFSAR and constituted a change to procedures as described in the UFSAR. The evaluation concluded that this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Deficiency Reports - Summary of Safety Evaluations

There were no changes in this category implemented during August 1999.

Other - Summary of Safety Evaluations

There were no changes in this category implemented during August 1999.