



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

FEB 12 1999

LR-N99-0081

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT
SALEM UNIT NO. 2
DOCKET NO. 50-311**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for January 1999 is attached.

Sincerely,

A. C. Bakken III
General Manager -
Salem Operations

/rbk
Enclosures

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C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

9902170353 990131
PDR ADOCK 05000311
R PDR

The power is in your hands.

DOCKET NO.: 50-311
 UNIT: Salem 2
 DATE: 2/15/99
 COMPLETED BY: R. Knieriem
 TELEPHONE: (609) 339-1782

Reporting Period: January 1999

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
Maximum Dependable Capacity (MWe-Net)

No. of hours reactor was critical
No. of hours generator was on line (service hours)
Unit reserve shutdown hours
Net Electrical Energy (MWH)

| | | |
|---------------|---------------------|-------------------|
| 1115 | | |
| 1106 | | |
| Month | Year-to-date | Cumulative |
| 744 | 744 | 89362 |
| 744 | 744 | 86095 |
| 0.0 | 0.0 | 0.0 |
| 821484 | 821484 | 85794315 |

UNIT SHUTDOWNS

| NO | DATE | TYPE F=FORCED S=SCHEDULED | DURATION (HOURS) | REASON (1) | METHOD OF SHUTTING DOWN THE REACTOR (2) | CORRECTIVE ACTION/COMMENT |
|----|------|---------------------------------|---------------------|---------------|--|------------------------------|
| | | | | | | |

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

Summary:

Salem Unit 2 operated at full power throughout the month of January 1999.

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UNIT: Salem 2
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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM UNIT 2 GENERATING STATION

MONTH: January 1999

The following items completed during **January 1999** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant; nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes - Summary of Safety Evaluations

Minor Modification S97-016, 2B Emergency Diesel Generator (EDG) Turbo Boost Air Pressure Low Alarm and EDG Turbo Boost Air Receiver Pressure Switch Setpoint Change

This minor modification raised the 2B EDG turbo boost air pressure low alarm setpoint from 190 psig to 206 psig. This change will provide additional margin to assure that air pressure will be available for the loading of the EDG in all plant modes.

This change was reportable under 10CFR50.59 because the setpoint of the turbo boost low-pressure alarm is described in the UFSAR. This change did not increase the probability or the consequences of an accident previously evaluated in the SAR, nor increase the probability of occurrence or the consequences of a malfunction of equipment important to safety. The turbo boost air pressure low alarm provides indication only and does not provide any control function related to the EDGs. This change supports the availability of the 2B EDG by providing operators with an earlier warning of EDG turbo boost air receiver low-pressure conditions. The change did not increase the possibility of an accident of a

different type from any previously evaluated in the SAR because no new failure modes were created by the change. Finally, the change did not affect the margin of safety, as defined in the Technical Specification Bases.

Temporary Modifications - Summary of Safety Evaluations

There were no changes in this category implemented during January 1999.

Procedures - Summary of Safety Evaluations

Radiation Protection Program Procedure, NC.NA-AP.ZZ-0024(Q), Revision 9

This procedural change was made to align the Radiation Protection Program procedure with the current organizational structure of the Nuclear Business Unit. In addition, minor changes were made to reflect changes to 10 CFR 20 that changed the doses to minors and pregnant members of the public to 0.1 rem, making the doses consistent with doses to other members of the public.

This change did not increase the probability or the consequences of an accident previously evaluated in the SAR. The change did not increase the probability of occurrence or the consequences of a malfunction of equipment important to safety because no changes were made to any system or component required to safely operate or to shut down the reactor. The change did not increase the possibility of an accident of a different type from any previously evaluated in the SAR because no new failure modes were created by the change. Finally, the change did not affect the margin of safety, as defined in the Technical Specification Bases.

UFSAR Change Notices - Summary of Safety Evaluations

There were no changes in this category implemented during January 1999.

Deficiency Reports - Summary of Safety Evaluations

There were no changes in this category implemented during January 1999.

Other - Summary of Safety Evaluations

There were no changes in this category implemented during January 1999.