



PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

April 15, 1998

LR-N98-0193

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT
SALEM UNIT NO. 2
DOCKET NO. 50-311**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for March, 1998, is attached.

Sincerely,

A. C. Bakken III
General Manager -
Salem Operations

RBK/rbk
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
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The power is in your hands.

DOCKET NO.: 50-311
 UNIT: Salem 2
 DATE: 4/15/98
 COMPLETED BY: R. Knieriem
 TELEPHONE: (609) 339-1782

Reporting Period: March 1998

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
 Maximum Dependable Capacity (MWe-Net)

No. of hours reactor was critical
 No. of hours generator was on line (service hours)

Unit reserve shutdown hours
 Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
418	1427	82634
398	1406	79470
0.0	0.0	0.0
403024	1481224	78656809

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/COMMENT
3	3/1/98 - 3/15/98	F	346	A	4	Service Water System Biofouling

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

Summary:

Salem Unit 2 began the month of March shutdown. The unit was shutdown on February 11, 1998, to address the failure of the 2A Emergency Diesel Generator (EDG) turbocharger. This shutdown was reported in Licensee Event Report 311/98-005-00. The 2A EDG was returned to service on February 23, but Unit 2 remained shutdown until March 15, to address Pressurizer Safety valve and Power Operated Relief valve leakage; and biofouling of the Service Water system. Following its return to service, Unit 2 operated at full power until March 24, when an electrical fault on a

controls transformer caused a plant runback to 60% power. Unit 2 returned to full power on the following day and operated at full power for the remainder of the month.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM UNIT 2 GENERATING STATION

MONTH March 1998

The following items completed during **MARCH 1998** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

2EC-3298, Pkg. 1, Switchgear And Penetration Area Ventilation (SPAV) System Upgrade. This design incorporated enhancements to the SPAV system by modifying system components (fans, dampers, and instrumentation), modifying the system temperature control scheme, and by testing and balancing the system.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

Minor Modification, S97-147, Steam Line Radiation Monitors. This modification deleted a steam trap and rerouted the drain from the Steam Line Radiation monitors to the 22 Condenser via an existing connection in the Unit 2 Main Steam system.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

Temporary Modifications Summary of Safety Evaluations

There were no changes in this category implemented during March, 1998.

Procedures Summary of Safety Evaluations

There were no changes in this category implemented during March, 1998.

UFSAR Change Notices Summary of Safety Evaluations

UFSAR Change Notice 98-011, Control Room Ventilation (CRV) - Radiologically Contaminated Air Intrusion. This UFSAR Change Notice documented a change to the UFSAR description of CRV system operation for the Control Room Emergency Air Conditioning system (CREACS) in the single filtration train alignment, without pre-selecting an outside air intake.

This evaluation does not negatively impact any accident response. This procedure revision does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this procedure revision does not involve an Unreviewed Safety Question.

Deficiency Reports Summary of Safety Evaluations

There were no changes in this category implemented during March, 1998.

Other Summary of Safety Evaluations

Safety Evaluation S97-026, Offsite Dose Calculation Manual, Revision 11. Revision 11 to the Offsite Dose Calculation Manual (ODCM) corrected Safety Factor values used to calculate radioactive effluent waste release rates and radioactive liquid waste radiation monitor setpoints.

This evaluation does not negatively impact any accident response. This procedure revision does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this procedure revision does not involve an Unreviewed Safety Question.

Safety Evaluation S97-027, Steam Generator Tube Sentinel Plugs Incorrectly Repaired. This Safety Evaluation evaluated the operation of Salem Unit 2 with two Steam Generator tubes that were incorrectly repaired. This repair compromised the ability of sentinel plugs installed in those tubes to provide tube leak detection capability.

This evaluation does not negatively impact any accident response. This procedure revision does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this procedure revision does not involve an Unreviewed Safety Question.