



PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

JAN 15 1998

LR-N98016

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT
SALEM UNIT NO. 1
DOCKET NO. 50-272**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original monthly operating report for December, 1997, is attached.

Sincerely,

A. C. Bakken III
General Manager -
Salem Operations

RBK/tcp
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

9801260429 971231
PDR ADOCK 05000272
R PDR



The power is in your hands.

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 1/15/98
COMPLETED BY: R. B. Knieriem
TELEPHONE: (609) 339-1782

MONTHLY OPERATING SUMMARY

MONTH DECEMBER 1997

The unit is in a refueling and a steam generator replacement outage and remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, the unit will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues.
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of the unit to operate in a safe, event free manner.
- After the restart review, meet with the NRC and communicate the results of that review.

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 01/09/98
COMPLETED BY: F. Todd
TELEPHONE: (609) 339-1316

OPERATING DATA REPORT
OPERATING STATUS

1	Reporting Period DECEMBER 1997	Hours in Report Period	<u>744</u>
2	Currently Authorized Power Level (MWt)		<u>3411</u>
	Max Dependable Capacity (MWe-Net)		<u>1106</u>
	Design Electrical Rating (MWe-Net)		<u>1115</u>
3	Power level to which restricted (if any) (MWe Net)		<u>None</u>
4	Reason For Restriction (if any)		

	<u>This Month</u>	<u>Yr To Date</u>	<u>Cumulative</u>	
5	No. of hours reactor was critical	0	0	104380
6	Reactor reserve shutdown hours	0.0	0.0	0.0
7	Hours generator on line	0	0	100388
8	Unit reserve shutdown hours	0.0	0.0	0.0
9	Gross thermal energy generated (MWH)	0	0	318062229
10	Gross electrical energy generated (MWH)	0	0	105301000
11	Net electrical energy generated (MWH)	0	0	100148489
12	Unit Service Factor	0.0%	0.0%	55.8%
13	Unit Availability Factor	0.0%	0.0%	55.8%
14	Unit Capacity Factor (MDC)	0.0%	0.0%	50.4%
15	Unit Capacity Factor (DER)	0.0%	0.0%	50.0%
16	Unit Forced Outage Rate	100.0%	100.0%	30.7%
17	Shutdowns scheduled over next 6 months (type, date, duration):			
18	If shutdown at end of report period, estimated date of Startup:			Under Review

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 01/09/98
COMPLETED BY: F. Todd
TELEPHONE: (609) 339-1316

OPERATING DATA REPORT
UNIT SHUTDOWNS AND POWER REDUCTIONS

MONTH DECEMBER 1997

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENT
3859	12/1 - 12/31	F	744	F, C	4	Steam Generator Replacement and Refueling Outage

(1) Reason

A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training/License Examination
F - Administrative
G - Operational Error (Explain)
H - Other

(2) Method

1 - Manual
2 - Manual Trip
3 - Automatic Trip/Scram
4 - Other

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 01/09/98
COMPLETED BY: F. Todd
TELEPHONE: (609) 339-1316

AVERAGE DAILY UNIT POWER LEVEL

MONTH DECEMBER 1997

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	<u>0</u>
3	<u>0</u>	19	<u>0</u>
4	<u>0</u>	20	<u>0</u>
5	<u>0</u>	21	<u>0</u>
6	<u>0</u>	22	<u>0</u>
7	<u>0</u>	23	<u>0</u>
8	<u>0</u>	24	<u>0</u>
9	<u>0</u>	25	<u>0</u>
10	<u>0</u>	26	<u>0</u>
11	<u>0</u>	27	<u>0</u>
12	<u>0</u>	28	<u>0</u>
13	<u>0</u>	29	<u>0</u>
14	<u>0</u>	30	<u>0</u>
15	<u>0</u>	31	<u>0</u>
16	<u>0</u>		

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 1/15/98
COMPLETED BY: R. B. Knieriem
TELEPHONE: (609) 339-1782

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM UNIT 1 GENERATING STATION

MONTH DECEMBER 1997

The following items completed during **December 1997** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

1EC-3191, Pkg. 1, Turbine Driven Auxiliary Feed Pump(TAFP) Recirculation Relief Valve Modification. This modification installs a new pressure relief valve on the TAFP continuous recirculation line to re-establish over pressure protection for the 13 TAFP Woodward Governor/Turbine Bearing Jacket coolers.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-03237, Pkg. 1, No. 15 Service Water Pump Replacement. This design change replaces major portions of the cement lined carbon steel Turbine Building Service Water piping with 6% molybdenum austenetic stainless steel piping material.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an

accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3329, Pkg. 1, 14 Service Water Pump (1SWE4) Replacement

Modification. This design change replaces Service Water pump 1SWE4 with a pump that is manufactured of materials that are more resistant to the corrosive, erosive, and silt laden river water present in the Service Water system.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3349, Pkg. 1, Relocation of 12 Containment Fan Cooler Unit (CFCU)

Return Flow Control Valve 12SW223. This design change relocates the Service Water CFCU return flow control valve 12SW223 away from the 12SW76 stop valve to eliminate cavitational damage to 12SW76 caused by the throttling affect of 12SW223.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3390, Pkg. 1, Addition of Time Delay for 1H Group Bus Undervoltage.

This design change provides a time delay of 0.30 seconds in the undervoltage reactor trip circuit to prevent spurious reactor trips from momentary electrical power transients.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3390, Pkg. 2, Addition Of Time Delay For 1E Group Bus Undervoltage.

This design change interposes an external time delay element on the 4KV group bus 1E undervoltage trip to prevent spurious reactor trips from momentary electrical power transients.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3390, Pkg. 3, Addition Of Time Delay For 1F Group Bus Undervoltage.

This design change interposes an external time delay element on the 4KV group bus 1F undervoltage trip to prevent spurious reactor trips caused by momentary electrical power transients.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3390, Pkg. 3, Addition Of Time Delay For 1G Group Bus Undervoltage.

This design change interposes an external time delay element on the 4KV group bus 1G undervoltage trip to prevent spurious reactor trips caused momentary electrical power transients.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3405, Pkg. 1, Salem Unit 1 Waste Gas Analyzer Panel Backfit. This design change upgrades instrumentation and controls on Waste Gas Analyzer panel 110-1 and makes it identical to Waste Gas Analyzer Panel 110-2, which is the corresponding Unit 2 panel.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3407, Pkg. 1, 11 Service Water Pump (1SWE1) Replacement

Modification. This design change replaces Service Water pump 1SWE1 with a pump that is manufactured of materials that are more resistant to the corrosive, erosive, and silt laden river water present in the Service Water system..

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3409, Pkg. 1, 16 Service Water Pump (1SWE6)Replacement

Modification. This design change replaces Service Water pump 1SWE1 with a pump that is manufactured from materials that are more resistant to the corrosive, erosive, and silt laden river water present in the Service Water system..

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3417, Pkg. 1, 28 VDC Circuit Breaker Replacement. This design change replaces the 28 VDC Circuit Breakers, cables, and time delay relays to reduce the voltage drop associated with control power and indication in auxiliary system circuits. The modification enhances the voltage availability such that the margin of voltage available at the 28 VDC auxiliary system components and devices will improve, increasing system reliability and performance.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3486, Pkg. 1, 10CFR50 Appendix R Alternate Shutdown Methodology - Installation Of Transfer Switches. This design change installs local switches in motor control center and control panel circuits for 34 components (3 chillers, 15 valves, 4 vent fans, 8 room coolers, and 4 pumps) to allow isolation of component wiring from the Control Room, Relay Room, and ceiling area of the 460/230V Switchgear Room. This will preclude the necessity of electrical rewiring and jumper installation to operate those components to achieve Hot Standby in the event of a fire in those areas.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3486, Pkg. 2, 10CFR50 Appendix R Alternate Shutdown Methodology - Installation Of Transfer Switches. This design change installs local switches in motor control center and control panel circuits for 10 valves to allow isolation of component wiring from the Control Room, Relay Room, and ceiling area of the 460/230V Switchgear Room. This will preclude the necessity of electrical rewiring and jumper installation to operate those components to achieve Hot Standby in the event of a fire in those areas.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3495, Pkg. 2, Steam Trap Replacement Project. This design change installs a restricting orifice, a strainer, and a bypass valve in lieu of a steam trap in the Main Steam system to reduce the potential of dumping excessive quantities of steam into the condenser when steam traps are bypassed due to trap failure. The modification will also minimize the contribution of Main Steam piping condensate removal systems to the Reactor Coolant system excess cooling phenomenon.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3607, Pkg. 1, Pressurizer Overpressure Protection System (POPS) Relief Valves (1PR1/1PR2) Circuit Modification. This design change modifies the POPS control circuitry to permit manual action by the operator to open or close the POPS relief valves even when POPS is armed.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3622, Pkg. 1, Liquid Radwaste System Enhancements. This modification installs a Tubular Ultra Filtration system (TUF) unit to reduce the radioactivity released in liquid effluent.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3648, Pkg. 1, SSPS Phase A and Containment Ventilation Isolation Modification. This modification installs relays and switches to provide the capability to manually initiate either Containment Ventilation or Containment Phase A Isolation individually when required. The previous configuration provided a single switch which initiated a manual isolation of both Containment Ventilation and Containment Phase A Isolation valves simultaneously.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3668, Pkg. 4 and 5, Generic Letter 96-06 Modifications. This design change brings Salem Unit 1 into conformance with the Salem SAR and SER to eliminate the potential for two-phase flow, voiding, water hammer, and thermally induced overpressure in the Containment Fan Coil Unit Service Water piping. This safety evaluation evaluated the interim plant conditions during installation of the modifications under 1EC-3668.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3668, Pkg. 8, Generic Letter 96-06 Modifications, Service Water System 11 Header Check Valve Installation. This modification installed a redundant check valve in the 11 Service Water Nuclear header Containment Fan Coil Unit (CFCU) 16" supply piping downstream of existing check valve 11SW51. This check valve will prevent drainage of the 11 and 12 CFCU's supply piping in the event of check valve 11SW51 failure.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3668, Pkg. 19, Generic Letter 96-06 Modifications, Service Water System CFCU Thermal Overpressure Protection. This modification installed a 1" bypass line for each of the five CFCU Service Water return lines. This will eliminate the potential for overpressure in the fan coils in the event of a Loss of Offsite Power (LOOP) associated with a Main Steam Line Break (MSLB) or Loss of Coolant Accident (LOCA).

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3696, Pkg. 1, 1PR1 and 1PR2 Power Operated Relief Valve (PORV) Control Modifications. This design change modified the PORV control circuitry to provide separation/isolation of safety-related from non-safety-related functions to prevent a credible single failure from affecting both PORV's. This change supports License Amendment 194 to credit the PORV's for inadvertent safety injection.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an

accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EA-1242, Pkg. 2, Salem Unit 1 Steam Generator (SG) Replacement Project: Steam Generator Blowdown Sample Line Modification. This modification installed a grab sample valve and a portion of the tubing and tube tray for the SG Blowdown sample line. The original SG (Model 51 SG) sample tubing was located in painted and insulated carbon steel tube tray. A portion of the tubing and tube tray were removed from each Model 51 SG in order to effect the SG replacement with an unused Model F SG.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EA-1273, Pkg. 1, Salem Unit 1 Steam Generator (SG) Replacement Project. This modification replaced the original SGs (Model 51 SG) with Model F SGs. The replacement Model F SGs are similar in design to the Model 51 SGs notwithstanding the number of tubes and differences in system connection points and other appurtenances.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0027, Pkg. 1, 13 Charging Pump Packing Seal Tank Overflow Line Flowmeter and Flow Totalizer Installation. This modification installed a turbine flowmeter and a digital flow totalizer on the 13 Charging Pump packing seal tank overflow line. The installed instrumentation will be used to measure pump seal leakage so that this leakage can be classified as an identified leakage.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0034, Pkg. 1, Circulating Water Standpipe Sample Pump Upgrade. This modification upgrades the Circulating Water Standpipe Sample pumps to enhance the reliability and maintainability of the chlorination system.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an

accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0066, Pkg. 1, 11A Circulating Water Traveling Screen Modifications.

This modification upgrades the 11A Circulating Water Traveling Screen to improve reliability and performance.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0067, Pkg. 1, 11B Circulating Water Traveling Screen Modifications.

This modification upgrades Circulating Water Traveling Screens to improve reliability and performance.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0151, Pkg. 1, Protection Of Condensate Piping From Over

Pressurization. This modification added a relief valve to protect piping and equipment from over pressurization when the Condensate pump is isolated.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0159, Pkg. 1, Local Pressure Gauges - Seismic Mounting. This modification was performed to enhance and qualify supports for various non-safety related pressure gauges in the Chemical and Volume Control, Service Water, Residual Heat Removal, Control Air, Chilled Water, Component Cooling, Safety Injection, Containment Spray, and Auxiliary Feedwater systems to conform to Seismic Class 1 criteria.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0161, Pkg. 1, Coating For CFCU Cooling Coil And Motor Cooler Water

Boxes. This modification implemented the refurbishment of the corrosion deteriorated CFCU cooling coil and motor cooler water boxes by either repair or

replacement of water box components and the application of a protective coating to resist further corrosion.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0176, Pkg. 1, Disconnect Motor Operated Valve (MOV) Heaters From 13 MAC Distribution Panel. This modification removed power to the motor heaters on MOV's. The MOV heaters are non-safety related, unqualified, and not required to maintain environmental qualification. Prior to this modification the breakers for these heaters were tagged open. Disconnection of the heaters will eliminate the need for plant staff to periodically evaluate the necessity to keep or remove the tags as required by the tagging procedure.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0196, Pkg. 1, 11 Reactor Coolant Pump (RCP) Seal Injection Flow Transmitter Replacement . This modification replaced the original equipment Seal Injection Flow transmitter (1FIT144) with a Rosemount 3051C "Smart" Model transmitter.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0333, Pkg. 2, Replace Service Water Pump Strainer Backwash Valve 12SW24. This modification replaced the existing Service Water pump strainer backwash diaphragm valve (12AW24) with a ball valve which has a longer service life between preventive maintenance and is better suited for the frequent cycling of backwash service.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0335, Pkg. 1, Main Steam Coil Drain Tank (MSCDT) Discharge Line Check Valve Additions. This modification installed check valves in the drain lines from the MSCDT to the sixth point (No. 16) Feedwater heaters and

replaced small sections of carbon steel pipe with chrome molybdenum pipe to support the erosion/corrosion program.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0349, Pkg. 1, Circulating Water Pump Bearing Lubrication Relief Valve Replacement. This modification replaces the relief valve in the Circulating Water pump bearing lubrication water pump header with a newer model relief valve of increased size and with increased set pressure.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EE-0350, Pkg. 1, 12 Service Water Pump Replacement. This modification replaced the 12 Service Water pump with an upgraded pump that is interchangeable with the existing pumps.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

Temporary Modifications Summary of Safety Evaluations

There were no changes in this category implemented during December, 1997.

Procedures Summary of Safety Evaluations

S1.OP-SO.RC-0002(Q), Reactor Coolant System (RCS) Vacuum Refill System. This evaluated procedure S1.OP-SO.RC-0002(Q) prior to its use to vacuum fill the RCS.

This procedure revision does not negatively impact any accident response. This procedure revision does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this procedure revision does not involve an Unreviewed Safety Question.

UFSAR Change Notices Summary of Safety Evaluations

UFSAR Change Notice 97-130. The UFSAR change revises SGS-UFSAR Sections 6.3.2, to add clarification of Emergency Core Cooling system (ECCS) modes of operation by including the hot leg flowpath from the RHR pump. The hot leg flow path from the RHR pump was previously removed from the UFSAR when it was eliminated in the Emergency Operating Procedures (EOP) for a LOCA occurring in Modes 1-3.

This UFSAR change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

UFSAR Change Notice 97-133. The UFSAR change shifts reporting relationships for the Salem Operations Organization and changes the management title for the Salem Chemistry Department without introducing any new or deleting any former responsibilities. It also revises the station succession of authority.

This UFSAR change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

Deficiency Reports Summary of Safety Evaluations

There were no changes in this category implemented during December, 1997.

Other Summary of Safety Evaluation

Safety Evaluation S97-137. Nuclear Engineering Reorganization. This review evaluated the organizational changes imposed by the December 1997 engineering reorganization.

This evaluation does not negatively impact any accident response. This procedure revision does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this procedure revision does not involve an Unreviewed Safety Question.