

TECHNICAL SPECIFICATION PAGES WITH PROPOSED CHANGES

The following Technical Specifications for Facility Operating License No. DPR-75 are affected by this change request:

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4.4.6.3	3/4 4-11
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REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

4.4.6.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The first inservice inspection shall be performed after 6 Effective Full Power Months but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. <sup>↑</sup>If two consecutive inspections following service under AVT conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.
- b. If the results of the inservice inspection of a steam generator conducted in accordance with Table 4.4-2 at 40 month intervals fall in Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.4.6.3.a; the interval may then be extended to a maximum of once per 40 months.
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 during the shutdown subsequent to any of the following conditions:
  1. Primary-to-secondary tubes leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.7.2.
  2. A seismic occurrence greater than the Operating Basis Earthquake.
  3. A loss-of-coolant accident requiring actuation of the engineered safeguards.
  4. A main steam line or feedwater line break.

FOR FUEL CYCLE 10 ONLY, THE INSPECTION INTERVAL SHALL BEGIN AT CRITICALITY.

**ALTERNATE INSPECTION SAMPLING PLAN  
FOR SALEM UNIT 2  
FOURTH REFUELING OUTAGE**

During the Salem Unit 2 Fourth Refueling Outage, indications associated with wall degradation were detected on the tubing of the No. 22 and No. 24 Steam Generators. The condition was established as occurring on the inside diameter of the Row 1 tubes in the tangential region of the u-bend.

The following alternate action may be taken in place of that required by Technical Specification Table 4.4-2 when the results of the initial sample requires that an additional sample or samples must be inspected and the condition for which the added inspection is required is limited to the Row 1 tubes. When examination results fall into a C-2 or C-3 Supplemental Sample Category, pursuant to Technical Specification Table 4.4-2 as a result of Row 1 and/or Row 2 u-bend defective or degraded tubes, additional samples may be limited to Rows 1 and 2. The results of the examination of the Row 1 and Row 2 tubes will be exempt from the additional sampling requirements of Technical Specification Table 4.4-2.

This change applies to only those steam generator eddy current inspections performed during the Salem Unit 2 Fourth Refueling Outage.

INSERT A

"INSERT A"

**STEAM GENERATOR SURVEILLANCE PERIOD AMENDMENT  
FOR SALEM NUCLEAR GENERATING STATION UNIT 2  
FUEL CYCLE 2R10**

Salem Unit 2 was removed from service in June of 1995 for a comprehensive review of plant methods and policies. In May of 1996, a 100% bobbin coil and additional specialty examinations inspection of the Salem Unit 2 steam generators was completed. Permission to restart Unit 2 was given by the NRC in June of 1997 and Mode 2 first achieved on August 17, 1997. After the May 1996 inspection, Unit 2 steam generators were placed in lay-up, using EPRI guidelines, to protect the steam generators from deterioration. PSE&G has a high level of confidence that corrosion growth and new corrosion initiation during the time of lay-up were essentially halted, and the condition of the steam generators has not changed since the May 1996 inspection.

Thus, in order to avoid an unnecessary mid-cycle steam generator inspection forced outage, Technical Specification 3/4.4.6 is hereby amended such that the next steam generator inspection will be required to be performed within 24 months of Mode 2 for Unit 2 fuel cycle 10. This would be by August 17, 1999. Subsequent steam generator inspections will be scheduled accordingly.