



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

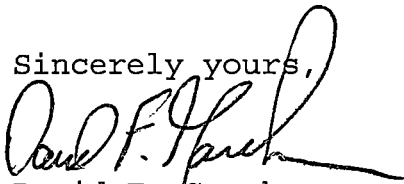
MAY 15 1997
LR-N970310

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

MONTHLY OPERATING REPORT
SALEM UNIT NO. 1
DOCKET NO. 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical specifications, the original monthly operating report for April, 1997, is attached.

Sincerely yours,

David F. Garchow
General Manager -
Salem Operations

RAR:tcp
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

111
Le24

9705220281 970430
PDR ADOCK 05000272
R PDR
2200



The power is in your hands.

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 05/15/97
COMPLETED BY: R. Ritzman
TELEPHONE: (609) 339-1445

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM GENERATING STATION

MONTH APRIL 1997

The following items completed during **April 1997** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes **Summary of Safety Evaluations**

There were no changes in this category implemented during April, 1997.

Temporary Modifications **Summary of Safety Evaluations**

There were no changes in this category implemented during April, 1997.

Procedures **Summary of Safety Evaluations**

SC.CH-SA.WD-0244(Q), Rev. 18, Plant Vent Sampling. This procedure revision provides a temporary, auxiliary continuous sampling method to meet Technical Specification requirements. This method of meeting these Technical Specification requirements is being used while the normal

method is unavailable to support outage activities. This procedure is only intended to be used while the 1R45 sample shed is out of service. This temporary sampling method maintains the ability to continuously sample for radioactive particulate and iodine releases.

Therefore, this design change does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

UFSAR Change Notices Summary of Safety Evaluations

CN 97-039, Nuclear Business Unit (NBU) Organization Change. This CN shifts, clarifies, and consolidates responsibilities for NBU organizations, revises the succession of authority and responsibility for Salem station operations, establishes new positions, and corrects editorial errors. The changes provide better control over NBU activities, and do not involve any reduction in the qualifications or training requirements, technical support, management control, commitments, or the quality assurance program.

Therefore, this CN does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

CN 97-044, Nuclear Engineering Consolidation. This CN reorganizes the functions within the Nuclear Engineering Department. The description of the entire Nuclear Engineering Department function is now consolidated under the control of the Senior Vice President - Nuclear Engineering. This provides flexibility in transferring responsibilities between direct reports to the Senior Vice President - Nuclear Engineering.

Therefore, this CN does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

CN 97-056, Residual Heat Removal Pumps Net Positive Suction Head in Recirculation Modes. This CN clarifies the Net Positive Suction Head (NPSH) design basis for the Residual Heat Removal (RHR) pumps in the post-LOCA recirculation mode. The design basis for RHR pump NPSH in the recirculation mode of operation is to ensure that adequate NPSH exists to meet the NPSH required for the maximum postulated flows. A new design basis calculation was completed. The calculation evaluated the suction piping pressure drop; and as a result, the design basis has been

updated. The NPSH Available continues to exceed the NPSH Required.

Therefore, this CN does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

Deficiency Reports Summary of Safety Evaluations

There were no changes in this category implemented during April, 1997.

Other Summary of Safety Evaluation

S-C-ZZ-ESE-0841, Rev 2, Safety Evaluation for Separation Distance of Cables in Free Air Installed in Salem Unit 1 and Unit 2 Limited Hazard Areas. This evaluation clarifies the free air separation distance criteria for the Salem Generating Station Limited Hazard Areas. The separation criteria is based on industry testing that was used as the technical basis for the 1992 revision of IEEE 384. IEEE 384-1992 indicates that the specified separation criteria for open to open configurations will establish independence of 1E equipment and circuits. Open to open configurations applicable to Salem are free air cable to free air cable and free air cable to open cable trays. This safety evaluation provides additional information and does not result in a reduction in the margin of safety. Compliance with IEEE 384-1992 ensures that faults will not propagate to adjacent cables nor cause a loss of redundant functions.

Therefore, this safety evaluation does not increase the probability or consequences of an accident previously described in the UFSAR and does not involve an Unreviewed Safety Question.

OPERATING DATA REPORT

Docket No: 50-272
 Date: 05/10/97
 Telephone: 339-2735

Completed by: Robert Phillips

Operating Status

1. Unit Name	<u>Salem No. 1</u>	<u>Notes</u>
2. Reporting Period	<u>April</u>	<u>1997</u>
3. Licensed Thermal Power (MWt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	<u>N/A</u>	

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>719</u>	<u>2879</u>	<u>177284</u>
12. No. of Hrs. Rx. was Critical	<u>0</u>	<u>0</u>	<u>104380.5</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>0</u>	<u>0</u>	<u>100388.5</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>318062229</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>10530100</u>
18. Net Elec. Energy Gen. (MWH)	<u>-3021</u>	<u>-10937</u>	<u>100157788</u>
19. Unit Service Factor	<u>0</u>	<u>0</u>	<u>56.5</u>
20. Unit Availability Factor	<u>0</u>	<u>0</u>	<u>56.5</u>
21. Unit Capacity Factor			
(using MDC Net)	<u>0</u>	<u>0</u>	<u>51.0</u>
22. Unit Capacity Factor			
(using DER Net)	<u>0</u>	<u>0</u>	<u>50.6</u>
23. Unit Forced Outage Rate	<u>100</u>	<u>100</u>	<u>29.6</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)
Steam Generator replacement.

25. If shutdown at end of Report Period, Estimated Date of Startup:
Under review.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272
 Unit Name: Salem #1
 Date: 05/10/97
 Telephone: 339-2735

Completed by: Robert Phillips

Month April 1997

Day Average Daily Power Level
(MWe-NET)

Day Average Daily Power Level
(MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

UNIT SHUTDOWN AND POWER REDUCTIONS
 REPORT MONTH April 1997

DOCKET NO.: 50-272
 UNIT NAME: Salem #1
 DATE: 05/10/97
 COMPLETED BY: Robert Phillips
 TELEPHONE: 609-339-2735

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
3859	4-1-97	F	744	F,C	4	-----	ZZ	ZZZZ	Steam Generator Replacement

¹
 F: Forced
 S: Scheduled

²
 Reason
 A-Equipment Failure (explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Continuation of
 Previous Outage
 5-Load Reduction
 9-Other

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG-0161)

⁵
 Exhibit 1 - Same
 Source

Refueling Information
Month: April, 1997

Docket No. 50-272
Unit Name: Salem 1
Contact: D.Tisdell
Telephone: 609-339-1538

Month: April, 1997

1. Refueling information has changed from last month: Yes: No: X
2. Scheduled date for next refueling: To Be Determined
Scheduled date for restart following refueling: To Be Determined
3. a. Will Technical Specification changes or other license amendments be required?
Yes: X No: Not Determined to Date:
b. Has the reload fuel design been reviewed by the Station Operating Review Committee?
Yes: No: X If no, when is it scheduled? To be Determined
4. Scheduled date (s) for submitting proposed licensing action: To be Determined
5. Important licensing considerations associated with refueling:

6. Number of Fuel Assemblies:
 - a. Incore: 0
 - b. In Spent Fuel Storage: 953
7. Present Licensed spent fuel storage capacity: 1632
Future spent fuel storage capacity: 1632
8. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: April 2012

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
APRIL 1997

SALEM UNIT 1

The unit is in a refueling and steam generator replacement outage and remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, the unit will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues.
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of the unit to operate in a safe, event free manner.
- After the restart review, meet with the NRC and communicate the results of that review.