PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

March 11, 1997

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO: 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original monthly operating report for the month of February is being sent to you.

Sincerely yours,

David F. Garchow

David F. Garchow
General Manager -
Salem Operations

VS:pc
Enclosures

240084

C Mr. H. J. Miller
Regional Administrator USNRC, Region1
475 Allendale Road
King of Prussia, PA 19046

IE24

10CFR50.59 EVALUATIONS
MONTH: FEBRUARY 1997

DOCKET NO: 50-272
NAME: UNIT 1
CONTACT: N. CONICELLA
TELEPHONE: 609-339-2124

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations

ITEM	SUMMARY

DESIGN CHANGE PACKAGE

(No 50.59 S/E) 1EA-1134 Pkg. 1, Rev. 0 on addition of Station Air Valves 1SA549 and 1SA550 in Auxiliary Building and Valves 1SA541 through 1SA548	This modification provides as-built documentation for the non-safety related portion of the station air in the auxiliary building, turbine building and in the clean facilities building. **SORC:** 97-022
50.59 S/E S97-026, 1EA-1126, Pkg. 1, Rev. 0 As-Built SGFP Suction Strainer Cleanout Valves, 11CN926 & 12CN926	This DCP provides as-built documentation for valves 11CN926 and 12CN926. The valves provide for controlled manual blowdown of strainers at the suction of both Unit 1 SGFP's. **SORC:** 97-021
50.59 S/E S97-032, 1EC-3486 Pkg. 2, Rev. 10CFR50 Appendix R alternate Shutdown Methodology - Installation of Transfer Switches	This modification addresses concerns regarding the Alternate Shutdown Methodology at Salem in the event of a Control Room evacuation due to fire in the Control Room, Relay Room, or ceiling area of the 460/230 Volt Switchgear Room, which calls for the use of electrical wiring modifications and jumpers. **SORC:** 97-019
50.59 S/E S97-043, 1EC-3495, Rev. 2 on Steam Trap Replacement Project	In order to enhance system performance and eliminate high maintenance components, twenty seven steam traps (1MSE21 through 1MSE35 and 1MSE38 through 1MSE49) and associated bypass orifices (where presently installed), bypass valve(s), drain valves and isolation valves will be removed (except four isolation valves required by procedure for Steam Generator Tube Rupture) and are being replaced. **SORC:** 97-025

10CFR50.59 EVALUATIONS
MONTH: FEBRUARY 1997

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The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations

ITEM	SUMMARY
DESIGN CHANGE PACKAGE	
50.59 S/E S97-049, 1EC-3620, Pkg. 1, Rev. 0 on Letdown Isolation Valve Bypass Key switch	This proposal will revise Section 5.6.4 and Figure 7.2-5 of the UFSAR to describe the new bypass circuitry of the low pressurizer level isolation signal so that operator control of the letdown isolation valves is possible during steam generator tube rupture conditions. **SORC:** 97-025
50.59 S/E S97-058, 1EC-3648, SSPS Phase A and containment Ventilation Isolation Modification	This modification adds relays and additional Main Control Room pushbuttons to both Train A and Train B to allow manual initiation of Containment Ventilation Isolation separately from Phase A Containment Isolation. **SORC:** 97-028
PROCEDURE	
50.59 S/E S97-023, TS1.SE-SU.CC-0003(Q), Rev. 0 "Component Cooling Pump Performance with 11CC16 & 12CC16 Throttled"	This procedure describes an "experimental" test which will demonstrate if the CC system is capable of reaching pump run-out flow and also whether the CC pump and motor are capable of surviving at maximum pump flow. **SORC:** 97-017
SAFETY EVALUATION	
50.59 S/E S96-047, S.C.ZZ-ESE-0841 Rev. 1 Separation distance of cables in free air installed in Salem U/1and U/2 relay rooms	This safety evaluation addresses separation of the free air cables installed in the Salem Relay Rooms. **SORC:** 96-099

OPERATING DATA REPORT

Docket No: 50-272
Date: 03/10/97
Completed by: Robert Phillips
Telephone: 339-2735

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	February 1997	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity(Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	672	1416	172417
12. No. of Hrs. Rx. was Critical	0	0	104380.5
13. Reactor Reserve Shutdown Hrs.	0	0	0
14. Hours Generator On-Line	0	0	100388.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	0	318062229
17. Gross Elec. Energy Generated (MWH)	0	0	105301000
18. Net Elec. Energy Gen. (MWH)	-2446	-5470	100169266
19. Unit Service Factor	0	0	58.2
20. Unit Availability Factor	0	0	58.2
21. Unit Capacity Factor (using MDC Net)	0	0	52.5
22. Unit Capacity Factor (using DER Net)	0	0	52.1
23. Unit Forced Outage Rate	100	100	27.0

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

Steam Generator replacement.

25. If shutdown at end of Report Period, Estimated Date of Startup:
Under review.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272
Unit Name: Salem #1
Date: 03/10/97
Telephone: 339-2735

Completed by: Robert Phillips

Month February 1997

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	
14	0	30	
15	0	31	
16	0		

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH February 1996

DOCKET NO.: 50-311
UNIT NAME: Salem #1
DATE: 02-10-97
COMPLETED BY: Robert Phillips
TELEPHONE: 609-339-2735

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
4092	2-1-97	F	744	F,C	4	----------	2422		Steam Generator Replacement

1
F: Forced
S: Scheduled

2
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Requlatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

Refueling Information
Month: February, 1997

Docket No. 50-272
Unit Name: Salem 1
Contact: D.Tisdall
Telephone: 609-339-1538

Month: February, 1997

1. Refueling information has changed from last month: Yes: No: X

2. Scheduled date for next refueling: To Be Determined

   Scheduled date for restart following refueling: To Be Determined

3. a. Will Technical Specification changes or other license amendments be required?

   Yes: No: Not Determined to Date: X

   b. Has the reload fuel design been reviewed by the Station Operating Review committee?

   Yes: No: X If no, when is it scheduled? To be Determined

4. Scheduled date (s) for submitting proposed licensing action: N/A

5. Important licensing considerations associated with refueling:

6. Number of Fuel Assemblies:
   a. Incore: 0

   b. In Spent Fuel Storage: 953

7. Present Licensed spent fuel storage capacity: 1632

   Future spent fuel storage capacity: 1632

8. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: April 2012

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
FEBRUARY 1997

SALEM UNIT 1

The unit is in a refueling and steam generator replacement outage and remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, the unit will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues.
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of the nit to operate in a safe, event free manner.
- After the restart review, meet with the NRC and communicate the results of that review.