



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

**Nuclear Business Unit**

February 13, 1997

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Attn: Document Control Desk

MONTHLY OPERATING REPORT  
SALEM NO. 2  
DOCKET NO: 50-311

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original monthly operating report for the month of January is being sent to you.

Sincerely yours,

David F. Garchow  
General Manager -  
Salem Operations

VS:pc  
Enclosures

C Mr. H. J. Miller  
Regional Administrator USNRC, Region I  
475 Allendale Road  
King of Prussia, PA 19046

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PDR ADOCK 05000311  
R PDR

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The power is in your hands.

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations

ITEM	SUMMARY
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**DESIGN CHANGE PACKAGES**

2EA-1124 "Addition to the Gland Sealing Condensate Heating Heater at E1.88 TGA"  
This DCP documents an as-built drain valve. The valve has no function during operation of the gland seal system, it is only used when the plant is shutdown to drain a header.

SORC: 97-014

2EC-3269, Pkg. 2 "Auxiliary Building Ventilation Upgrade" This DCP rebalances the ventilation air flow rate to the 21, 22 and 23 component cooling water (CCW) pump rooms to assure design temperatures are maintained upon a postulated failure of 22 CCW pump room cooler.

SORC: 97-014

2EC-3285, Pkg. 2, Rev 1 "Condensate System Re-Rate and Relief Valve Replacement" This DCP involves modifications to the Steam Generator Feed and Condensate System. The proposed change to the system includes replacement of 18 relief valves. The effects on the reactor and its associated margins of safety due to postulated secondary pipe line ruptures or relief valve failure are not addressed in the basis of any Tech. Spec. section, nor do they effect any systems discussed in the Tech. Specs. As such, there are no requirements imposed on this system by the Tech. Specs. and no reduction in the margin of safety. The proposed modification does not reduce the margin of safety as the Steam Generator Feed and Condensate System are not addressed in the Tech. Specs. or Bases.

SORC: 97-001

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations

ITEM	SUMMARY
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**DESIGN CHANGE PACKAGES**

2EC-3554, Pkg. 1, Rev. 1,	"Heater Drain Pump start and Heater Drain Valve (HD 15) Open Interlock Modification" This DCP will provide an anti-reset windup feature and will interlock the starting of the bleed steam heater drain pumps with their associated bleed steam heater drain pump discharge air operated globe valves to prevent the HD15 valve from being full open when the Heater Drain Pump is started. The anti-reset windup feature and interlock will ensure that the HD15 valve closes when the Heater Drain Pump is started and then opens and returns to automatic control. This will prevent the feedwater heater and moisture separator reheater drain tank from draining down too rapidly prior to the HD15 valve responding under automatic level control and will eliminate the present "work around" requiring manual control of these valves. A review of the Tech. Spec. and their bases was performed and it was determined that there were no Technical Specification applicable to the proposal, therefore, the proposal does not reduce the margin of safety as defined in the basis for any Tech. Spec
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SORC: 97-007

2EC-3602, Pkg. 1	"Addition of Relief Valve to the Tubing Section between Valve 2SS103 and the SS26 Header" The DCP will eliminate the potential for over-pressurization of the tubing between valve 2SS103 and the SS26 header during a postulated LOCA or steam line break by allowing potentially trapped water to thermally expand and be relieved. In addition, reversing the 2SS103 connections enables the valve to accommodate higher pressures.
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SORC: 97-002

2EE-0281, Pkg. 1, Rev. 1	"Circulating Water Pump Bearing Lube Relief Valve Replacement" This DCP incorporates the change in scope for the heat trace circuit used for the new relief valve. The new section of heat trace will add 0.6 amps in load. This minimal increase is well within the capability of the 20 amp breaker and does not affect load management.
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SORC: 97-016

10CFR50.59 EVALUATIONS  
MONTH: JANUARY 1997

DOCKET NO: 50-311  
NAME: UNIT 2  
CONTACT: N. CONICELLA  
TELEPHONE: 609-339-2124

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations

ITEM	SUMMARY
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**PROCEDURE**

S-2-DG-ESE-0872

"Credit The Diesel Generator 2000hr Rating For Accident Mitigation"  
Recent modifications and testing developments (SWP BHP measurements, SI pump BHP measurements, CCW pump runout and Control Room Ventilation Mods) have indicated that the 2C DG may enter the 2000 Hour rating and remain in this rating for the duration of a large break LOCA - LOOP accident. The 2000 hour rating for the DG was provided by the OEM at the time of purchase and is 2750 KW. This rating is required only when the DG is operating at the maximum Technical Specification limit (LCR S95-36) of 60.5 hertz. The 2000 Hr rating is the length of time and load (2750 KW) the manufacturer recommends that the DG can operate before a major overhaul should be completed. When the DG is operating at the design setpoint of 60 hertz, the 2000 Hr rating is not entered.

SORC: 97-011

Refueling Information  
Month: January 1997

Docket No. 50-311  
Unit Name: Salem 2  
Contact: N. Conicella  
Telephone: 609-339-2124

MONTH: January 1997

Refueling information has changed from last month: YES:        NO: X

Scheduled date for next refueling: (currently in outage)

Scheduled date for restart following refueling: (to be determined)

a. Will Technical Specification changes or other license amendments be required?

YES:        NO: X NOT DETERMINED TO DATE:

b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES: X NO:

If no, when is it scheduled?

Scheduled date(s) for submitting proposed licensing action: N/A

Important licensing considerations associated with refueling:

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Number of fuel assemblies:

a. Incore:	193
b. In Spent Fuel Storage:	584

Present licensed spent fuel storage capacity:	1632
Future spent fuel storage capacity:	1632

Date of last refueling that can be discharged to the spent fuel pool  
assuming the present licensed capacity:

March 2012

OPERATING DATA REPORT

Docket No: 50-311  
 Date: 02/10/97  
 Telephone: 339-2735

Completed by: Robert Phillips

Operating Status

1. Unit Name	<u>Salem No. 2</u>	<u>Notes</u>
2. Reporting Period	<u>January 1997</u>	
3. Licensed Thermal Power (MWt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	<u>N/A</u>	

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744</u>	<u>744</u>	<u>134.151</u>
12. No. of Hrs. Rx. was Critical	<u>0</u>	<u>0</u>	<u>7808.6</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>0</u>	<u>0</u>	<u>75229.5</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>187781005</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>78648898</u>
18. Net Elec. Energy Gen. (MWH)	<u>-6303</u>	<u>-6303</u>	<u>74696331</u>
19. Unit Service Factor	<u>0</u>	<u>0</u>	<u>56.1</u>
20. Unit Availability Factor	<u>0</u>	<u>0</u>	<u>56.1</u>
21. Unit Capacity Factor (using MDC Net)	<u>0</u>	<u>0</u>	<u>50.3</u>
22. Unit Capacity Factor (using DER Net)	<u>0</u>	<u>0</u>	<u>49.9</u>
23. Unit Forced Outage Rate	<u>100</u>	<u>100</u>	<u>100</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

Refueling extension.

25. If shutdown at end of Report Period, Estimated Date of Startup:

Second quarter of 1997.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-311  
Unit Name: Salem #2  
Date: 02/10/97  
Telephone: 339-2735

Completed by: Robert Phillips

Month January 1997

Day Average Daily Power Level  
(MWe-NET)

Day Average Daily Power Level  
(MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>





SALEM GENERATING STATION  
MONTHLY OPERATING SUMMARY - UNIT 2  
JANUARY 1997

SALEM UNIT NO. 2

The unit remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, the unit will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of the unit to operate in a safe, event free manner
- After the restart review, meet with the NRC and communicate the results of that review