



PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

November 14, 1996

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn.: Document Control Desk

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO: 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating report for the month of October is being sent to you.

Sincerely yours,

David F. Garchow
General Manager -
Salem Operations

RH:vl
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region I
475 Allendale Road
King of Prussia, PA 19046

9611260046 961031
PDR ADOCK 05000272
R PDR

The power is in your hands.

IF241,

AVERAGE DAILY UNIT POWER LEVEL

Completed by: Robert Phillips

Docket No.: 50-272
 Unit Name: Salem #1
 Date: 11/10/96
 Telephone: 339-2735

Month October 1996

Day Average Daily Power Level
 (MWe-NET)

Day Average Daily Power Level
 (MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

OPERATING DATA REPORT

Completed by: Robert Phillips

Docket No: 50-272
 Date: 11/10/96
 Telephone: 339-2735

Operating Status

1. Unit Name	<u>Salem No. 1</u>	<u>Notes</u>
2. Reporting Period	<u>October 1996</u>	
3. Licensed Thermal Power (MWt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason <u>N/A</u>		

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
12. Hours in Reporting Period	<u>745</u>	<u>7320</u>	<u>169537</u>
12. No. of Hrs. Rx. was Critical	<u>0</u>	<u>0</u>	<u>104380.5</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>0</u>	<u>0</u>	<u>100388.3</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>318062229</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>105301000</u>
18. Net Elec. Energy Gen. (MWH)	<u>-2903</u>	<u>-24341</u>	<u>100180572</u>
19. Unit Service Factor	<u>0</u>	<u>0</u>	<u>59.2</u>
20. Unit Availability Factor	<u>0</u>	<u>0</u>	<u>59.2</u>
21. Unit Capacity Factor			
(using MDC Net)	<u>0</u>	<u>0</u>	<u>53.4</u>
22. Unit Capacity Factor			
(using DER Net)	<u>0</u>	<u>0</u>	<u>53.0</u>
23. Unit Forced Outage Rate	<u>100</u>	<u>100</u>	<u>25.4</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

Refueling extension.

25. If shutdown at end of Report Period, Estimated Date of Startup:
Under review.

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH October 1996

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: 11/10/96
COMPLETED BY: Robert Phillips
TELEPHONE: 609-339-2735

[illegible]

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

4 Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File
(NUREG-0161)

5
Exhibit 1 - Same
Source

10CFR50.59 EVALUATIONS
MONTH: OCTOBER 1996

DOCKET NO: 50-272
UNIT NAME: SALEM 1
CONTACT: R. HELLER
TELEPHONE: 609-339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY
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1. Design Change Packages (DCP)

- | | |
|------------------|--|
| 1EC-3360, Pkg. 5 | "Control Room Architectural Upgrade" Rev. 0 - This DCP renovated various architectural aspects of the Control Room complex. This specific package provided a new Operators' Ready Room that is contained within the EACS Pressure Boundary. This area also contains a Meteorological Information Data Acquisition System modem and the primary and secondary communications stations. There is no change to the power sources for this equipment. This package also removed the existing card readers, balance magnetic switches and electric locks associated with security doors nos. 331 and 131 and welded the doors shut. This package also removed the two Fire Protection hose reels and provided a wall mounted cabinet for the storage of the two 75' long fire hoses. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-082) |
| 1EC-3639, Pkg. 1 | "Fuel Handling Building (FHB) Sump Pump Issues" Rev. 0 - This DCP installed replacement FHB sump pump level switches along with float actuated mechanisms, with redesigned mounting hardware to guide the float arms. This DCP also installed a new local mounted run time meter for the sump pump motor. This modification increased the reliability of the sump level switches for the auto start of the pump and the high level alarm. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-127) |

10CFR50.59 EVALUATIONS
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ITEM	SUMMARY
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2. UFSAR Change Notices (SCN)

- | | |
|------------|--|
| SCN 96-150 | "Clarification of the Accuracies used in the Reactor Coolant System Subcooling Margin Monitor" - This change reconciles the UFSAR incorporating changes that were developed as a result of the Salem Unit 1 & 2 EOP Design Basis Verification Program. The change does not delete, revise, or modify the intent of any procedures either listed or described in the SAR and does not change the intent or means of operating the subcooling margin monitor. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-138) |
| SCN 96-170 | "Steam Generator Tube Rupture - Allocated Time to Terminate Break Flow" - The revision reflects two changes: 1.) The text identifying the estimated time it would take operators to complete the isolation procedure following a tube rupture is being changed from 30 to 50 minutes. This is to accommodate communication standards which are now being used in the control room, and 2.) During the cooldown stage, procedures direct operators to achieve a margin of subcooling between the RCS and the faulted steam generator pressures. The text in the UFSAR identifying the amount of this margin has been changed from 50F to 20 F to align with assumptions made in the Emergency Response Guidelines. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-147) |
| SCN 96-146 | "Significant Event Response Team Management" - This change deleted the reference to Significant Event Response Team (SERT) Management from the list of topics in UFSAR Section 13.5.1.1 for which nuclear administrative procedures (NAPs) have been written. (Hope Creek has no equivalent reference to SERT Management.) Accordingly, the SERT Management procedure, NC.NA-AP.ZZ-0061(Q) (NAP-61) will be eliminated. In the future, in lieu of using NAP-61 for evaluating |

10CFR50.59 EVALUATIONS
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ITEM	SUMMARY
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plant events, the Action Request Process NC.NA-AP.ZZ-0000(Q) (NAP-0) and the Corrective Action Program NC.NA-AP.ZZ-0006(Q) (NAP-6) will be used. There is nothing in the Technical Specification that indicates or refers to how significant events and conditions adverse to quality are to be investigated or evaluated. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-135)

3. Procedures and Revisions

- | | |
|---------------------|--|
| NC.NA-AP.ZZ-0013(Q) | "Control of Temporary Modifications" Rev. 3 - This revision deletes unauthorized modifications and provides general editing improvements. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-154) |
| NC.NA-AP.ZZ-0003(Q) | "Document Management Program" Rev. 5 - This revision is a complete rewrite and restructuring of the procedure. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-154) |

4. Safety Evaluations

- | | |
|---------------------|--|
| Radiological Safety | "Reorganization of Radiological Safety: Splitting Emergency Preparedness from Radiation Protection and Chemistry and changing the reporting structure." - This revision changed the structure and reporting requirements of Radiological Safety making Emergency Preparedness separate from Radiation Protection, Chemistry, Dosimetry and Radiological Instruments. The functions and responsibilities remain intact, although the groups have different reporting requirements. Both groups report to the Director of Nuclear Training, and the previous Safety Evaluation (Salem 96-060, Hope Creek 96-009) determined that the qualifications of the Director - Nuclear Training exceed all of the ANSI standards and regulatory |
|---------------------|--|

10CFR50.59 EVALUATIONS
MONTH: OCTOBER 1996

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(cont'd)

ITEM	SUMMARY
	<p>guidance. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-135)</p>
S-C-CBV-MDC-1637	<p>"Containment Fan Cooler Unit (CFCU) Capabilities" Rev. 0 - This evaluation changes the design basis for the CFCUs but does not impact the technical specifications. The proposal makes UFSAR Chapter 6 consistent with the remainder of the UFSAR and corrects minor discrepancies in Chapter 15. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-136)</p>
Diesel Generator	<p>"Safety Evaluation on Diesel Generator Lube Oil Discrepancy in the UFSAR" - PR 950823075 identified a discrepancy in the description of the operation of the Diesel Generator prelube oil pump described in UFSAR Section 9.5.7. This change notice revises the description of the prelube oil pump to indicate that the prelube oil pump provides backup to the main shaft driven pump when the diesel generator is tripped and the prelube oil pump is started when jacket water pressure reaches its low pressure setpoint. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-140)</p>
Section 9.4.4.4	<p>"Containment Pressure - Vacuum Relief System" - This safety evaluation supports an FSAR change which removes an ambiguous statement from the tests-and-inspections section of the Containment Ventilation System Description. This statement appears in the Tests and Inspections section of UFSAR section 9.4.4, Containment Ventilation System. The statement that the system operates for 15 minutes to relieve the positive or negative pressure may have impacted initial start-up test procedures but otherwise has no operational significance based on a detailed reading of the technical specifications and the USNRC safety evaluation report (SER). It makes no physical changes to any plant system, equipment, or components, and makes no changes to procedures or other</p>

10CFR50.59 EVALUATIONS
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ITEM	SUMMARY
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controls. There is no reduction in the margin of safety as defined in the basis for any Technical Specification.
(SORC 96-136)

5. Miscellaneous

Security Plan

"Salem - Hope Creek Security Plan" Rev. 8 - This plan reflects change in the following areas: 1.) Security organizational change; 2.) Clarification of language describing the basis of the vital area configuration of Salem Units 1 & 2; 3.) Change Salem control room access portals; 4.) Added language relative to land vehicle barrier system; 5.) Added a provision for the electronic submittal of area access authorizations, and 6.) Deleted provisions for temporary designated vehicles. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-144)

REFUELING INFORMATION
MONTH: OCTOBER 1996

DOCKET NO: 50-272
UNIT NAME: SALEM 1
CONTACT: R. HELLER
TELEPHONE: 609-339-5162

MONTH : OCTOBER 1996

- . Refueling information has changed from last month: YES ____ NO X
- . Scheduled date for next refueling: (to be determined)
- . Scheduled date for restart following refueling: (to be determined)
- . a. Will Technical Specification changes or other license amendments be required?

YES ____ NO ____

NOT DETERMINED TO DATE X

- b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES ____ NO X

If no, when is it scheduled? (to be determined)

- . Scheduled date(s) for submitting proposed licensing action: n/a
- . Important licensing considerations associated with refueling:

- . Number of Fuel Assemblies:
 - a. Incore 0
 - b. In Spent Fuel Storage 989
- . Present licensed spent fuel storage capacity: 1632
Future spent fuel storage capacity: 1632
- . Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: October 2008

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
OCTOBER 1996

SALEM UNIT NO. 1

The Unit is in a refueling and Steam Generator replacement outage and remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, the Unit will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of the Unit to operate in a safe, event free manner
- After the restart review, meet with the NRC and communicate the results of that review