Public Service
Electric and Gas
Company

Louis F. Storz
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United States Nuclear Regulatory Commission
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Washington, DC 20555

SUPPLEMENTAL INFORMATION
RADIOLOGICAL DOSE ASSESSMENT METHODOLOGY
SALEM GENERATING STATION NOS. 1 AND 2
FACILITY OPERATING LICENSES DPR-70 AND DPR-75
DOCKET NOS. 50-272 AND 50-311

Gentlemen:

This letter is being sent to provide updated information to the Staff pertaining to the radiological dose assessment that was performed in support of a requested change to the Technical Specifications on Control Room Emergency Air Conditioning System, (ref. LR-N96154, dated June 10, 1996). The attached information revises previously supplied information of June 24, 1996, (ref. LR-N96178), and July 1, 1996, (ref. LR-N96191) to reflect changed input assumptions and the resulting Control Room calculated doses.

The enclosed pages provide revised input assumptions, data and results pertaining to a postulated LOCA, fuel handling accident inside the Fuel Handling Building and Reactor Coolant Pump locked rotor accidents. The revised LOCA calculations include changes to the specified Control Room makeup flow from 2500 cfm to 2200 cfm as a result of including 10% margin and to the alignment time of the Auxiliary Building Ventilation System from thirty minutes to two hours. These were discussed in Public Service Electric & Gas Company's September 20, 1996 letter, (ref. LR-N96284). The revised calculations for fuel handling and locked rotor include these changes as well as Salem specific intake radiation monitor response data and a three second monitor response delay time due to the noise reject algorithm inherent to the monitor design.

The attached includes information pertaining to the calculation of Exclusion Area Boundary and Low Population Zone doses. This

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information, however, is not relevant to the Control Room dose analyses and should be disregarded. PSE&G is not requesting approval of the revised offsite doses since it does not include the application of the ARCON95 methodology for determining dispersion factors.

Please note that the attached 29 pages are not sequentially numbered and represent complete updates of Table 5 attached to LR-N96178 and all attachments to LR-N96191. References listed on the included pages are to the internal documents from which the calculational information was obtained.

If any further information pertaining to this supplement is needed, please call.

Sincerely,

Louis F. Storz

Attachments (2)

C Mr. H. J. Miller, Administrator - Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. L. Olshan, Licensing Project Manager - Salem
U. S. Nuclear Regulatory Commission
One White Flint North
11555 Rockville Pike
Mail Stop 14E21
Rockville, MD 20852

Mr. C. Marschall (X24)
USNRC Senior Resident Inspector

Mr. K. Tosch, Manager IV
Bureau of Nuclear Engineering
33 Arctic Parkway
CN 415
Trenton, NJ 08625

REF: LR-N96318

STATE OF NEW JERSEY)
) SS.
COUNTY OF SALEM)

L.F. Storz, being duly sworn according to law deposes and says:

I am Senior Vice President - Nuclear Operations of Public Service Electric and Gas Company - and as such, I find the matters set forth in the above referenced letter, concerning Salem Generating Station, Unit 1 and Unit 2, is true to the best of my knowledge, information and belief.

Louis F. Storz
L.F. Storz, Senior Vice President
Nuclear Operations

Subscribed and Sworn to before me
this 17th day of October 1996

Kimberly Jo Brown
Notary Public of New Jersey

KIMBERLY JO BROWN
NOTARY PUBLIC OF NEW JERSEY
My Commission Expires April 21, 1998

My Commission expires on _____.

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Attachment to LR-N96321

SALEM GENERATING STATION UNIT NOS. 1 AND 2
FACILITY OPERATING LICENSES DPR-70 AND DPR-75
DOCKET NOS. 50-272 AND 50-311
SUPPLEMENTAL INFORMATION
RADIOLOGICAL DOSE ASSESSMENT METHODOLOGY

ATTACHED ERRATA PAGES

The following calculation pages are enclosed:

Loss of Coolant Accident (Replaces Table 5 of LR-N96178)

Objective	Page 6 of 53
Assumptions	Page 15 of 53
Data	Pages 16 - 21 of 53
Results and Conclusions	Pages 52 & 53 of 53

Fuel Handling Accident in the Fuel Handling Building (Replaces all previously submitted pages attached to LR-N96191)

Objective	Page 5
Assumptions	Page 8
Data	Pages 9 - 11
Results/Conclusions	Page 21
Appendix A	Pages App-A(1 & 2 of 2)

Reactor Coolant Pump Locked Rotor Accident (Replaces all previously submitted pages attached to LR-N96191)

Objective	Page 5
Assumptions	Page 9
Data	Pages 10 - 14
Control Room Intake Monitor Response Time	Pages 19 & 20
Results and Conclusions	Pages 40 & 41