PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

August 15, 1996

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn.: Document Control Desk

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO: 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating report for the month of July is being sent to you.

Sincerely yours,

H. J. Miller for

David F. Garchow
General Manager -
Salem Operations

RH:vls
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region I
475 Allendale Road
King of Prussia, PA 19046

200052

IE24 1/1

OPERATING DATA REPORT

Docket No: 50-272
Date: 08/10/96
Completed by: Robert Phillips
Telephone: 339-2735

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	July 1996	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity(Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
12. Hours in Reporting Period	744	5111	167328
12. No. of Hrs. Rx. was Critical	0	0	104380.5
13. Reactor Reserve Shutdown Hrs.	0	0	0
14. Hours Generator On-Line	0	0	100388.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	0	318062229.2
17. Gross Elec. Energy Generated (MWH)	0	0	105301000
18. Net Elec. Energy Gen. (MWH)	-2600	-16221	100188692
19. Unit Service Factor	0	0	60.0
20. Unit Availability Factor	0	0	60.0
21. Unit Capacity Factor (using MDC Net)	0	0	54.1
22. Unit Capacity Factor (using DER Net)	0	0	53.7
23. Unit Forced Outage Rate	100	100	24.2

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

Forced outage.

25. If shutdown at end of Report Period, Estimated Date of Startup:
To be determined.

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272
Unit Name: Salem #1
Date: 08/10/96
Telephone: 339-2735

Completed by: Robert Phillips

Month July 1996

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH June 1996

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: 08/10/96
COMPLETED BY: Robert Phillips.
TELEPHONE: 609-339-2735

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
4092	07-01-96	F	744	F,C	4	----------	HJ	HTEXCH	Nuclear Steam Generator Systems.

1
F: Forced
S: Scheduled

2
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

10CFR50.59 EVALUATIONS
MONTH: JULY 1996

DOCKET NO: 50-272
UNIT NAME: SALEM 1
CONTACT: R. HELLER
TELEPHONE: 609-339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY

1. Design Change Packages (DCP)

1EC-3505, Pkg. 2

"Control Area Emergency Air Conditioning System (EACS system) Condensate Drain Trap Seal" Rev. 0 - This DCP removed the existing Seismic Class 3 Condensate drain line from the Unit 2 EACS, located in the Auxiliary Building, elevation 122'-0", and installed a new Seismic Class 1 Condensate Drain piping with a fillable trap and a Seismic Class 3 site glass. Also this DCP removed two (2) 1" EACS housing drain lines located on the downstream side of the charcoal filter. There is no reduction in the margin of safety as defined in the basis for any Technical Specification because (1) there is no credible event analyzed in Chapter 15 of the UFSAR which can cause an unacceptable environment in the control room since core alterations are currently restricted; (2) fuel movement inside the Fuel Handling Building (FHB) is restricted unless FHB ventilation is operable, and (3) the beyond-design-basis event which might impact the control room (rupture of an ammonium hydroxide tanker) is precluded by procedure. (SORC 96-033)

2. Evaluations

Radiological Safety

"Radiological Safety Organizational Reporting Structure" (Organizational Change) - The revision changes the structural reporting requirements for Radiological Safety, encompassing the functional areas of emergency preparedness, radiation protection, chemistry, dosimetry and radiological instruments, from Nuclear Operations Support to the Nuclear Training Center. The functions and responsibilities remain the same, although the groups will have different reporting requirements. The qualifications of the Director of Nuclear Training and the

ITEM	SUMMARY
	General Manager of Nuclear Services are similar in nature, although there is a lesser requirement for time requirement for the director (8 versus 10 years). This lesser requirement is sufficiently rigorous and exceeds the ANSI standard so the net effect will not diminish quality. The proposed changes do not relate to design criteria, specifications, or operations of systems or components relating to the fuel cladding, Reactor Coolant System (RCS) boundary, or containment, and do not address any margin of safety. Therefore, the proposed changes do not reduce the margin of safety as defined in the bases for any Technical Specification. (SORC 96-095)
3. UFSAR	
S96-086	"Minimum Neutron Count Rate on Source Range Instrumentation" - This change revises Section 4.2.3.1 reducing the minimum required capability of the neutron sources from providing 2 counts per second to greater than 1 cps on the source range channels. The proposed changes will not impact the initial subcritical boration requirements or the capability of the instrumentation to detect changes in core multiplication. The source range monitors' operability requirements for all affected Modes remain unchanged and are not based on minimal indicated neutron count rate but on the prescribed channel calibrations and functional test. The change does not reduce the requirement that both channels provide continuous visual indication of the neutron flux and core multiplication. The proposed change does not reduce the margin of safety as defined in the basis for any Technical Specification. (SORC 96-096)
S96-093	"SCN Service Water System Modifications" - This safety evaluation supports changes to sections of the UFSAR relative to the Salem service water system. These changes are required to address issues raised by problem reports. The changes include corrections to address discrepancies / clarifications determined to be necessary as a result of both internal and external system reviews. The external review

10CFR50.59 EVALUATIONS
MONTH: JULY 1996

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(cont'd)

ITEM	SUMMARY

included comments from the NRC during the 5/96 licensing basis audit. The acceptance criteria for postulated design basis accidents affected by the Service Water system define the acceptable margin of safety. This proposal does not affect the design limits of the Service Water system, or design limits of components affected by this system. It is therefore concluded that this proposal will not reduce the margin of safety as defined in the basis for any Technical Specification. (SORC 96-098)

REFUELING INFORMATION
MONTH: JULY 1996

DOCKET NO: 50-272
UNIT NAME: SALEM 1
CONTACT: R. HELLER
TELEPHONE: 609-339-5162

MONTH : JULY 1996

. Refueling information has changed from last month: YES . ___ . NO . X .

. Scheduled date for next refueling: (to be determined)

. Scheduled date for restart following refueling: (to be determined)

. a. Will Technical Specification changes or other license amendments be required?

YES . ___ . NO . ___ .

NOT DETERMINED TO DATE . X .

b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES . ___ . NO . X .

If no, when is it scheduled? (to be determined)

. Scheduled date(s) for submitting proposed licensing action: . n/a .

. Important licensing considerations associated with refueling:

. Number of Fuel Assemblies:
a. Incore . 0 .
b. In Spent Fuel Storage . 989 .

. Present licensed spent fuel storage capacity: . 1632 .
Future spent fuel storage capacity: . 1632 .

. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September 2008

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
JULY 1996

SALEM UNIT NO. 1

The Unit is in a refueling and Steam Generator replacement outage and remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, the Unit will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of the Unit to operate in a safe, event free manner
- After the restart review, meet with the NRC and communicate the results of that review