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Nuclear Business Unit

JUN 27 1996

LR-N96151

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

**SALEM GENERATING STATION UNITS 1 AND 2
AND HOPE CREEK GENERATING STATION
FACILITY OPERATING LICENSES DPR-70, DPR-75 AND NPF-57
DOCKET NOS. 50-272, 50-311 AND 50-354
REQUEST FOR APPROVAL OF CHANGES TO THE QUALITY ASSURANCE PROGRAM**

In accordance with 10 CFR 50.54(a)(3), Public Service Electric & Gas Company (PSE&G) hereby submits for your approval a request to change the Quality Assurance (QA) Program for Salem and Hope Creek Generating Stations, as documented in Section 17.2 of the respective Updated Final Safety Analysis Reports (UFSAR).

This submittal consists of two proposed changes:

- 1) Eliminating the designation of Station Operations Review Committee (SORC) members by specific position titles.
- 2) Deleting the requirement for the Hope Creek SORC to review post-modification test procedures and test results for designated systems.

The NRC has previously approved the elimination of specific position titles for several other licensees including PECO Energy's Limerick Generating Station, Wisconsin Public Service Corporation's Kewaunee Nuclear Power Plant, and Florida Power and Light Company's (FP&L) St. Lucie Plant. The proposed requirements for Salem and Hope Creek SORC composition are similar to those for FP&L's St. Lucie Plant Facility Review Group.

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By letter dated January 11, 1996 (ref: LR-N95092), PSE&G submitted two license change requests (LCRs 95-13 and 95-06) to relocate the review and audit functions of Section 6.5 of the Salem and Hope Creek Generating Stations Technical Specifications (TS) to the PSE&G QA Program. This submittal covers the QA Program changes associated with relocating TS Sections 6.5.1 and 6.5.3 requirements. A separate letter was submitted under 10 CFR 50.54(a) to cover QA Program changes for relocating TS Sections 6.5.2 requirements. Approval of the license change request as an amendment to the Salem and Hope Creek TS and the 10 CFR 50.54(a) submittals is required for PSE&G implementation.

Attachment 1 contains a description of the proposed changes, the reasons for the proposed changes, and bases for concluding that the revised QA Program continues to satisfy the criteria of 10 CFR 50 Appendix B. Attachments 2 and 3 contain the changes to the Salem and Hope Creek UFSARs, respectively.

The proposed change to SORC membership requirements makes more effective use of the expertise available among members of the station staffs while ensuring that SORC members continue to possess the relevant site-specific technical and systems expertise required. The proposed change to Hope Creek SORC responsibilities will eliminate unnecessary duplication of work. Therefore, PSE&G requests approval of these changes by August 1, 1996.

If you have any questions regarding this submittal, please do not hesitate to contact us.

Sincerely,


J. A. Benjamin
Director -
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JUN 27 1996

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LR-N96151

ATTACHMENT 1
SALEM AND HOPE CREEK GENERATING STATIONS
FACILITY OPERATING LICENSES DPR-70, DPR-75 AND NPF-57
DOCKET NOS. 50-272, 50-311, AND 50-354
PROPOSED CHANGES TO THE QUALITY ASSURANCE PROGRAM

BACKGROUND

The proposed changes relocate specific review and audit functions from Sections 6.5.1 and 6.5.3 of the Salem and Hope Creek Generating Stations Technical Specifications (TS) to the Quality Assurance (QA) Program. In addition to relocating the functions, the proposal eliminates the designation of Station Operations Review Committee (SORC) members by specific position titles and deletes the requirement for the Hope Creek SORC to review post-modification test procedures and test results for designated systems.

The proposed change to SORC membership requirements makes more effective use of the expertise available among members of the station staffs while ensuring that SORC members continue to possess the relevant site-specific technical and systems expertise required. The proposed change to Hope Creek SORC responsibilities will eliminate unnecessary duplication of work.

By letter dated January 11, 1996 (ref: LR-N95092), PSE&G submitted two license change requests (LCRs 95-13 and 95-06) to relocate the review and audit functions of Section 6.5 of the Salem and Hope Creek Generating Stations TS to the PSE&G QA Program. This submittal covers the QA Program changes associated with relocating TS Sections 6.5.1 and 6.5.3 requirements. A separate letter was submitted under 10 CFR 50.54(a) to cover QA Program changes for relocating TS Section 6.5.2 requirements. Approval of the license change request as an amendment to the Salem and Hope Creek TS is required prior to implementing these changes.

SALEM AND HOPE CREEK SORC COMPOSITION

DESCRIPTION OF THE PROPOSED CHANGE

The proposed changes would eliminate the specific position titles for Station Operations Review Committee members in order to enable the plant manager to appoint any appropriately qualified member of the plant staff as a SORC member. The SORC would be chaired by the plant manager and composed of regular members designated in writing by the plant manager from the plant staff and from the station QA organization.

SORC members would meet the qualification requirements specified for their designated areas of expertise as follows:

	<u>Salem</u>	<u>Hope Creek</u>
Plant Operations	ANSI N18.1-1971	ANSI/ANS 3.1-1981
Engineering	ANSI N18.1-1971	ANSI/ANS 3.1-1981
Maintenance	ANSI N18.1-1971	ANSI/ANS 3.1-1981
Chemistry	ANSI N18.1-1971	ANSI/ANS 3.1-1981
Radiation Protection	Regulatory Guide 1.8, September 1975	
Quality Assessment	ANSI/ANS 3.1-1981	ANSI/ANS 3.1-1981

Quorum requirements would not be changed. A quorum consists of the Chairman or his designated alternate and four members including alternates. No more than two alternates shall participate as voting members at a SORC meeting.

REASONS FOR THE PROPOSED CHANGES

The requirements for SORC composition are currently specified in Technical Specification section 6.5.1.2. The SORC Chairman, Vice Chairmen and members are specified by position title:

Chairman:	Plant Manager
Member and Vice Chairman:	Maintenance Manager
Member and Vice Chairman:	Operations Manager
Member and Vice Chairman:	Technical Manager
Member:	Radiation Protection Manager
Member:	Operating Engineer
Member:	Technical Engineer
Member:	Maintenance Engineer
Member:	Chemistry Manager
Member:	Onsite Safety Review Engineer

Qualification requirements for members of the unit staff are specified in Technical Specification 6.3. Qualification requirements for the Onsite Safety Review Group are specified in Technical Specification 6.5.2.2. In a separate submittal dated May 22, 1996, PSE&G proposed to eliminate the Onsite Safety Review Group as a dedicated organization and to reassign its functions to the QA Department.

Specifying position titles for SORC members creates an administrative burden when titles are changed. Removing the specific position titles and allowing the plant manager to appoint any appropriately qualified member of the plant staff as a SORC member makes more effective use of the experience available among unit staff members.

BASIS FOR THE PROPOSED CHANGES

The quorum requirement for the SORC would be unchanged, and SORC members would continue to be required to meet the applicable qualification requirements for the appropriate discipline. At least one regular member would be designated in each area of expertise so that all areas of expertise currently represented on SORC would continue to be represented. Specifically, the members having experience in Engineering would provide the expertise currently provided by the Technical Manager and the Technical Engineer. The members from the station Quality Assessment organization would be required to meet the qualification requirements of ANSI/ANS 3.1-1981 and would provide the expertise currently provided by the Onsite Safety Review Group Engineer.

Allowing the plant manager to designate appropriately qualified members of the unit staff as members of the SORC will not degrade the effectiveness of the SORC. The proposed change ensures that relevant site-specific technical and systems expertise continues to reside among SORC members. The revised QA Program incorporating this change continues to satisfy the criteria of 10 CFR 50, Appendix B. Eliminating the specific position titles for Station Operations Review Committee members will have no adverse impact on the SORC function.

HOPE CREEK SORC RESPONSIBILITIESDESCRIPTION OF THE PROPOSED CHANGES

The proposed change would delete the requirement for the Hope Creek SORC to review post-modification test procedures and test results for designated systems.

REASONS FOR THE PROPOSED CHANGES

The Hope Creek SORC is currently required to review test procedures and test results after modifications to Q, F and R-designated systems in accordance with Hope Creek UFSAR section 17.2.11, Test Control. The Q designation refers to systems, structures, and components for which operational quality assurance requirements are applicable. Quality group classifications are described in Hope Creek UFSAR Table 3.2-1. The F designation refers to systems, structures, and components covered by fire protection quality assurance program. The R designation applies to portions of the Radioactive Waste Management System which prevent uncontrolled release of solid, liquid or gaseous radioactive waste to the environment.

The requirement to review post-modification test procedures and results is in addition to the requirement for the SORC to review all proposed changes to plant equipment or systems that affect nuclear safety.

The retest requirements necessary to assure that modifications have been accomplished correctly are included in the design change package. Test procedures are prepared and reviewed and test results are documented, evaluated and reviewed for acceptability in accordance with NBU procedures.

BASIS FOR THE PROPOSED CHANGES

The requirement for the Hope Creek SORC to review post-modification test procedures and results for designated systems can be deleted because the test control program includes other processes that will continue to ensure that post-modification tests are performed in accordance with written test procedures which incorporate appropriate acceptance criteria and that test results are reviewed to ensure all requirements have been met.

Q, F, and R-designated equipment and components that have been replaced, modified, or repaired are tested by qualified personnel in accordance with written procedures that provide acceptance criteria based on requirements contained in applicable design and procurement documents. Retest requirements are provided by engineering specifications or the responsible engineer, or both. The Nuclear Engineering and Operations departments are responsible for preparation of test procedures incorporating the engineering parameters. Test results are documented and reviewed for acceptability by the qualified department representative.

QA performs assessments of selected post modification tests to assure compliance with the test procedure. Test results are reviewed for: presentation of proper documentation; assurance that tests meet objectives; and identification and reporting of unacceptable results and initiation of corrective measures.

The revised QA Program incorporating this change continues to satisfy the criteria of 10 CFR 50, Appendix B. Review of test procedures and results by the Hope Creek SORC can be eliminated with no adverse effect on the control of post-modification testing. Post-modification tests will continue to be performed in accordance with written procedures and test results will continue to be documented and evaluated in accordance with the requirements of 10 CFR 50 Appendix B.

CONCLUSION

PSE&G has determined that, while these changes have been evaluated as a reduction in commitment, the revised QA Program continues to satisfy the criteria of 10 CFR 50, Appendix B. The proposed change to SORC composition will ensure that relevant site-specific technical and systems expertise continues to reside among SORC members. The change in Hope Creek SORC's responsibilities will eliminate unnecessary duplication of work.

LR-N96151

ATTACHMENT 2
SALEM UFSAR CHANGES

13.4 REVIEW AND AUDIT

13.4.1 Administrative Control

Administrative control of plant operations is directed by the Senior Vice President - Nuclear Operations through the General Manager - Salem Operations. Further details on administrative control are provided in the Technical Specifications.

13.4.2 Routine Review

A daily review of station logs and other operating data will be made by the Operating Department. All nonroutine operations and conditions will also be reviewed by other responsible departments, as appropriate. In addition to these reviews, periodic station staff meetings will be held to keep all operating personnel advised of conditions in the station.

To establish and ensure formal review and evaluation of plant operations, a Station Operations Review Committee (SORC) and a Quality Assurance/Nuclear Safety Review Department have been established and are described in the Salem Technical Specifications, Section 6.0, Administrative Controls and UFSAR Section 13.1 for QA/NSR.

INSERT A

Insert A

UFSAR Section 17.2

SGS-UFSAR

13.4-1a

The DIR-QA/NSR fulfills the above qualifications with the addition of the following:

1. Knowledge and experience in quality assurance.
2. High level of leadership, with the ability to command the respect and cooperation of company personnel, suppliers, and construction forces.
3. Initiative and judgment to establish related policies to attain high achievements and economy of operations.

17.2.1.1.2 Operational Review

PSE&G letter LR-N96104
ADDRESSES CHANGES TO THE
SRG AND OSR

Three advisory groups, the Station Operations Review Committee (SORC), the Onsite Safety Review Group (SRG), and the Offsite Safety Review Group (OSR), are responsible for reviewing and evaluating items related to nuclear safety. The overall responsibilities of these groups are described in Section 13.4. The Manager - Quality Assessment is invited to all SORC meetings and receives the minutes of the meetings, attends the meetings periodically.

INSERT B

17.2.1.2 Maplewood Testing Services

The Manager Maplewood Testing Services reports to the Director-Business and Maintenance Services in fossil generation.

Maplewood Testing Services performs calibrations, analyses, and evaluations on systems, equipment, and materials, as requested by PSE&G departments, and maintains compliance with its quality assurance program.

Insert B

17.2.1.1.2.1 Technical Review and Control

ACTIVITIES - Procedures and programs required by Technical Specification 6.8 and other procedures which affect nuclear safety as determined by the plant manager, other than editorial or typographical changes shall be reviewed as follows:

PROCEDURE RELATED DOCUMENTS - Procedures, programs and changes thereto shall be reviewed as follows:

1. Each newly created procedure, program or change thereto shall be independently reviewed by an individual knowledgeable in the subject area other than the individual who prepared the procedure, program or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Procedures other than Station Administrative procedures will be approved by the appropriate Station Department Manager or by the plant manager. Each Station Department Manager shall be responsible for a predesignated class of procedures. The plant manager shall approve Station Administrative Procedures, Security Plan implementing procedures and Emergency Plan implementing procedures.
2. On-the-spot changes to procedures which clearly do not change the intent of the approved procedures shall be approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator License. Revisions to procedures which may involve a change in intent of the approved procedures shall be reviewed in accordance with item 1 above.
3. Individuals responsible for reviews performed in accordance with item 1 above shall be approved by the SORC Chairman and designated as Station Qualified Reviewers. A system of Station Qualified Reviewers, each of whom shall possess qualifications that meet or

Insert B (cont'd)

exceed the requirements of Section 4.4 of ANSI N18.1-1971, shall be maintained by the SORC Chairman. Each review shall include a written determination of whether or not additional cross-disciplinary review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel.

4. If the Department Manager determines that the documents involved require a 10 CFR 50.59 safety evaluation, the documents shall be forwarded for SORC review and also to the Nuclear Review Board for an independent review to determine whether or not an unreviewed safety question is involved. Pursuant to 10 CFR 50.59, NRC approval of items involving unreviewed safety questions or Technical Specification changes shall be obtained prior to implementation.

NON-PROCEDURE RELATED DOCUMENTS - Tests or experiments and changes to equipment or systems shall be forwarded for SORC review and also to the Nuclear Review Board for an independent review to determine whether or not an unreviewed safety question is involved. Recommendations for approval are made by SORC to the plant manager. Pursuant to 10 CFR 50.59, NRC approval of items involving unreviewed safety questions or requiring Technical Specification changes shall be obtained prior to implementation.

RECORDS AND REPORTS - Written records of reviews performed in accordance with item 1 above, including recommendations for approval or disapproval, shall be maintained.

Insert B (cont'd)

17.2.1.1.2.2 Station Operations Review Committee (SORC)

FUNCTION - The Station Operations Review Committee shall function to advise the plant manager on operational matters related to nuclear safety.

COMPOSITION - The Station Operations Review Committee (SORC) shall be chaired by the plant manager and shall be composed of regular members from the Salem Generating Station staff and from the Salem Quality Assessment organization having experience in each of the following areas:

1. Plant Operations
2. Engineering
3. Maintenance
4. Chemistry
5. Radiation Protection
6. Quality Assessment

The member having experience in the area of Radiation Protection shall meet the qualification requirements of Regulatory Guide 1.8, September 1975. The member having experience in Quality Assessment shall meet the requirements of ANSI/ANS 3.1-1981. All other members shall meet the requirements of ANSI N18.1-1971 for the appropriate discipline. All members shall be appointed in writing by the plant manager. The Vice Chairmen shall be drawn from the SORC members and shall be appointed in writing by the plant manager.

ALTERNATES - All alternate members shall be appointed in writing by the SORC Chairman. Only the designated Vice Chairmen or the plant manager may act as Chairman of a SORC meeting. No more than two alternates to members shall participate as voting members in SORC activities at any one meeting. Alternates for members will not make up part of the voting quorum when the member the alternate represents is also present.

MEETING FREQUENCY - The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

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QUORUM - The minimum quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of this section shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES - The Station Operations Review Committee shall be responsible for:

1. Review of: (1) Upper tier administrative procedures within the scope of Regulatory Guide 1.33 (2/78), and changes thereto; and (2) Newly created procedures or changes to existing procedures that require a 10 CFR 50.59 safety evaluation as described in Section 17.2.1.1.2.1.
2. Review of all proposed tests and experiments that affect nuclear safety.
3. Review of all proposed changes to Appendix "A" Technical Specifications.
4. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
5. Review of the safety evaluations that have been completed under the provisions of 10 CFR 50.59.
6. Investigation of all violations of the Technical Specifications including the reports covering evaluation and recommendations to prevent recurrence.
7. Review of all REPORTABLE EVENTS.

Insert B (cont'd)

8. Review of facility operations to detect potential nuclear safety hazards.
9. Performance of special reviews, investigations or analyses and reports thereon as requested by the plant manager.
10. Review of the Fire Protection Program and implementing procedures and changes thereto that require a 10 CFR 50.59 safety evaluation.
11. Review of all unplanned on-site releases of radioactivity to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence.
12. Review of changes to the PROCESS CONTROL MANUAL and the OFF-SITE DOSE CALCULATION MANUAL.

AUTHORITY - The Station Operations Review Committee shall:

1. Provide recommendations to the plant manager indicating written approval or disapproval of items considered under the SORC responsibilities above.
2. Provide written notification within 24 hours to the Senior Vice President - Nuclear Operations, and the Director - Quality Assurance/Nuclear Safety Review of disagreement between the SORC and the plant manager; however, the plant manager shall have responsibility for resolution of such disagreements pursuant to Technical Specification 6.1.1.

RECORDS AND REPORTS - The Station Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Senior Vice President - Nuclear Operations, the Director - Quality Assurance/Nuclear Safety Review and the Nuclear Review Board.

TABLE 17.2-1

SALEM Q-LIST

The listing below identifies those activities, services, structures, components and systems to which the Operational Quality Assurance Program applies.

1. ACTIVITIES/SERVICES

1.1 Safety Related Activities Delineated in Regulatory Guide 1.33, App. A (See Regulatory Guide for further breakdown of activities)

1.1.1 Administrative Procedures

- a. Security Program (Regulatory Guide 1.17)
- b. Equipment Control (e.g., Locking and Tagging)
- c. Shift and Relief Turnover
- d. Bypass of Safety Functions and Jumper Control
- e. Maintenance of Minimum Shift Complement and Call-In of Personnel
- f. Fire Protection Program including Inspection by Fire Consultants
- g. Communication System

1.1.2 General Plant Operating Procedures

1.1.3 Startup, Operation, and Shutdown of Safety-Related Systems

1.1.4 Abnormal, Offnormal, or Alarm Conditions

1.1.5 Combating Emergencies and Other Significant Events

1.1.6 Control of Radioactivity

- a. Liquid Radioactive Waste System (including the contaminated floor and equipment drain systems)
- b. Solid Waste System
- c. PWR Gaseous Effluent System
- d. Radiation Protection including Occupational Radiation Exposure per Regulatory Guide 8.8
- e. Area Radiation Monitoring System Operation

Insert C

h. Station Operations Review Committee (SORC)

TABLE 17.2-1 (Cont)

- f. Process Radiation Monitoring System Operation
 - g. Meteorological Monitoring and Data Collection Program
 - h. Packaging and Transport of Radioactive Material per 10CFR71
 - i. Decontamination
- 1.1.7 Technical Specification Surveillance
 - 1.1.8 Performing Maintenance
 - 1.1.9 Chemical and Radiochemical Control
- 1.2 Additional NRC Requirements
- 1.2.1 Technical Specification Administrative Controls
 - a. ~~SORC~~
 - b. Reportable Occurrences
2. EQUIPMENT, COMPONENTS, AND STRUCTURES
- 2.1 The following are items and systems contained in commitment letters to the NRC.
 - 2.1.1 Accident Monitoring Instrumentation
 - 2.1.2 AC Control Power Buses and Inverters
 - 2.1.3 All Systems Which Penetrate Containment, up to and including the Containment Isolation Valve (Identified in UFSAR Section 6.2.4)
 - 2.1.4 Anticipatory Reactor Trip on Turbine Trip
 - 2.1.5 Auxiliary Building (including Control Room and Diesel Generator Area)
 - 2.1.6 Auxiliary Building Ventilation System (Supply and Exhaust Units)
 - 2.1.7 Auxiliary Feedwater Storage Tank
 - 2.1.8 Auxiliary Feedwater System
 - 2.1.9 Component Cooling System
 - 2.1.10 Chill Water System

LR-N96151

ATTACHMENT 3
HOPE CREEK UFSAR CHANGES

13.4 REVIEW AND AUDIT

The independent review and audit functions of ANSI N18.7 will be performed by the Station Operations Review Committee which reports to the General Manager - Hope Creek Operations, and the General Manager - QA and Nuclear Safety Review who reports to the Vice President and Chief Nuclear Officer. Reporting to the General Manager - QA and Nuclear Safety Review are the Manager - Nuclear Safety Review, and the Quality Assurance managers.* The QA and Nuclear Safety Review Department organization is indicated on Fig. 13.1-7. The equivalency to review groups identified in standard Technical Specifications is indicated below:

<u>PSE&G</u>	Equivalent Commonly Used Standard Technical Specification Terms
Station Operations Review Committee (SORC)	SORC or PORC or PRC or Unit Review Group (URG)
Onsite Safety Review Group (SRG)	ISEG
Offsite Safety Review (OSR)	Nuclear Review Board (NRB) or Offsite Safety Review Committee or Safety Review and Audit Board or Company Nuclear Review and Audit Group (CNRAG)

* The Hope Creek Onsite Safety Review Engineer and the Salem Onsite Safety Review Engineer report to the Manager - Nuclear Safety Review.

INSERT A

13.4-1

HCGS-UFSAR

Revision 5
May 11, 1993

Insert A

13.4 REVIEW AND AUDIT

The review and audit functions are described in section 17.2.

13.4.1 Station Operations Review Committee

The Station Operations Review Committee (SORC) was established and functional before the initial fuel loading. Its purpose, throughout the life of the plant, is to advise the General Manager - Hope Creek Operations on all matters related to nuclear safety.

13.4.1.1 Organization

Membership of the SORC is composed of the following:

1. Chairman - General Manager - Hope Creek Operations
2. Members:
 - a. Technical Manager - Vice Chairman
 - b. Operations Manager - Vice Chairman
 - c. Maintenance Manager - Vice Chairman

- d. Maintenance Engineer
- e. Radiation Protection/Chemistry Manager
- f. Chemistry Engineer or Radiation Protection Engineer
- g. Technical Engineer
- h. Onsite Safety Review Engineer
- i. Operating Engineer

Alternate appointees will only represent their respective department. Alternates for members will not make up part of the voting quorum when the member the alternate represents are also present.

13.4.1.1.1 Alternates

All alternate members shall be appointed in writing by the Chairman.

13.4.1.1.2 Quorum

A quorum of the SORC shall consist of the Chairman or his designated alternate and four members including alternates*. However, no more than two alternates shall participate as voting members in SORC activities at any one time.

* Only designated Vice - Chairman or the General Manager - Hope Creek Operations may act as Chairman of a SORC meeting.

13.4.1.2 Meetings

The SORC will meet at least once per calendar month and as convened by the Chairman or Vice Chairman. Minutes of all formal meetings

shall be maintained. Copies of minutes from SORC meetings are sent to the Manager - Nuclear Safety Review, the General Manager - QA and Nuclear Safety Review and the Vice President and Chief Nuclear Officer.

13.4.1.3 Responsibility

The SORC is responsible for the following duties:

1. Review of: 1) Upper tier Administrative Procedures within the scope of Reg. Guide 1.38 (2/78), and changes thereto; and 2) Newly created procedures or changes to existing procedures that require a 10CFR50.59 safety evaluation.
2. Review of all proposed tests and experiments that affect nuclear safety.
3. Review of all proposed changes to Appendix A, Technical Specifications.
4. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
5. Review of the safety evaluations that have been completed under the provisions of 10CFR50.59.
6. Initiation or review of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the Vice President and Chief Nuclear Officer and to the General Manager - QA and Nuclear Safety Review.
7. Review of all Reportable Events.
8. Review of facility operations to detect potential nuclear safety hazards.

9. Performance of special reviews, investigations, or analyses and reports thereon, as determined by the SORC.
10. Review of the Emergency Plan and implementing procedures and changes thereto that require a 10CFR50.59 Safety Evaluation, or involve a potential decrease in the effectiveness of the plan, per 10CFR50.59⁴(q).
11. Review of the Security Plan and implementing procedures and changes thereto that require a 10CFR50.59 Safety Evaluation, or involve a potential decrease in the effectiveness of the plan, per 10CFR50.54(p).
12. Review of the Fire Protection Program and implementing procedures and changes thereto that require a 10CFR50.59 Safety evaluation.
13. Review of all unplanned onsite releases of radioactivity to the environs including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence.
14. Review of changes to the Process Control Manual and the Offsite Dose Calculation Manual.

13.4.1.4 SORC Review Process

A technical review and control system performs the reviews generally required by SORC in accordance with standard Technical Specifications. This system focuses the SORC effort on those areas where the collective expertise of the committee members can have the most substantial contribution to the safety review effort. Important elements of this system include:

1. Routine periodic review of procedures and changes thereto are performed within the station organization, and only

those items that require a 10CFR50.59 safety evaluation will be referred to SORC for review.

2. SORC reviews will concentrate on consideration of safe and reliable operation of the station. Independent reviews for determination or verification of USQ will be performed by Offsite Safety Review (OSR), and the results of OSR reviews will be provided to SORC.
3. A system of qualified reviewers within the station organization has been established to assist SORC review effort.

Review and approval of temporary changes to procedures is described in Section 13.5.

The SORC will review all events that require notification to the NRC, violations of Technical Specifications, and any other unplanned events that may have nuclear safety significance. A report shall be submitted to the SORC by a designated staff member. The conclusions and recommendations reached by the SORC shall be recorded in the minutes of the meeting and forwarded to the General Manager - Hope Creek Operations and the General Manager - QA and Nuclear Safety Review. Any unreviewed safety questions shall be reported to the General Manager - QA and Nuclear Safety Review for further action.

13.4.1.5 Authority

The SORC shall:

1. Recommend to the General Manager - Hope Creek Operations written approval or disapproval of items 1. through 5. under Section 13.4.1.3.
2. Provide written notification within 24 hours to the General Manager - QA and Nuclear Safety Review and the Vice President and Chief Nuclear Officer of any

disagreement between the SORC and the General Manager - Hope
Creek Operations; the General Manager - Hope Creek
Operations will have the responsibility for resolutions of
such disagreements.

17.2.1.1.2 Operational Review

Three advisory groups, the Station Operations Review Committee (SORC), the Onsite Safety Review group (SRG), and the Offsite safety review group (OSR), are responsible for reviewing and evaluating items related to nuclear safety. The overall responsibilities of these groups are provided in Section 13.4. The Manager - Station Quality Assurance is invited to all SORC meetings and receives the minutes of the meetings. He attends the meetings periodically.

As part of its independent review functions, the OSR is responsible for selected preplanned, independent audits of plant operations in accordance with Technical Specification requirements. These audits are generally conducted by QA under OSR cognizance.

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17.2.1.2 Research and Testing Laboratory

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PROPOSED CHANGE

The General Manager Research and Testing Laboratory reports to the Vice-President Transmission Systems.

The Research and Testing Laboratory performs calibrations, analyses, and evaluations on systems, equipment, and materials, as requested by PSE&G departments, and maintains compliance with its quality assurance program.

17.2.1.3 Deleted

Insert B

17.2.1.1.2.1 Technical Review and Control

ACTIVITIES - All programs and procedures required by Technical Specification 6.8, and changes thereto, and any other proposed procedures or changes thereto which affect plant nuclear safety as determined by the plant manager, other than editorial or typographical changes shall be reviewed as follows:

PROCEDURE RELATED DOCUMENTS - Procedures, programs and changes thereto shall be reviewed as follows:

1. Each newly created procedure, program or change thereto shall be independently reviewed by an individual knowledgeable in the area affected other than the individual who prepared the procedure, program or procedure change, but who may be from the same organization as the individual/group which prepared the procedure or procedure change. Procedures other than Station Administrative procedures will be approved by the appropriate station Department Manager or by the plant manager. Each station Department Manager shall be responsible for a predesignated class of procedures. The plant manager shall approve Station Administrative Procedures, Security Plan implementing procedures and Emergency Plan implementing procedures.
2. On-the-spot changes to procedures which clearly do not change the intent of the approved procedures shall be approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator License. Revisions to procedures which may involve a change in intent of the approved procedures shall be reviewed in accordance with item 1 above.
3. Individuals responsible for reviews performed in accordance with item 1 above shall be approved by the SORC Chairman and designated as Station Qualified Reviewers. A system of Station Qualified Reviewers shall be maintained by the SORC Chairman. Each review

Insert B (cont'd)

shall include a written determination of whether or not additional cross-disciplinary review is necessary. If deemed necessary, such review shall be performed by the appropriate designated review personnel. The Station Qualified Reviewers shall meet or exceed the qualifications described in Section 4.1 and 4.7 of ANS 3.1 - 1981.

4. If the Department Manager determines that the documents involved require a 10 CFR 50.59 safety evaluation, the documents shall be forwarded for SORC review and also to the Nuclear Review Board for an independent review to determine whether or not an unreviewed safety question is involved. Pursuant to 10 CFR 50.59, NRC approval of items involving unreviewed safety questions or Technical Specification changes shall be obtained prior to implementation.

NON-PROCEDURE RELATED DOCUMENTS - Tests or experiments and changes to equipment or systems shall be forwarded for SORC review and also to the Nuclear Review Board for an independent review to determine whether or not an unreviewed safety question is involved. Recommendations for approval are made by SORC to the plant manager. Pursuant to 10 CFR 50.59, NRC approval of items involving unreviewed safety questions or requiring Technical Specification changes shall be obtained prior to implementation.

RECORDS AND REPORTS - Written records of reviews performed in accordance with item 1 above, including recommendations for approval or disapproval, shall be maintained.

Insert B (cont'd)

17.2.1.1.2.2 Station Operations Review Committee (SORC)

FUNCTION - The Station Operations Review Committee shall function to advise the plant manager on all matters related to nuclear safety.

COMPOSITION - The Station Operations Review Committee (SORC) shall be chaired by the plant manager and shall be composed of regular members from the Hope Creek Generating Station staff and from the Hope Creek Quality Assessment organization having experience in each of the following areas:

1. Plant Operations
2. Engineering
3. Maintenance
4. Chemistry
5. Radiation Protection
6. Quality Assessment

The member having experience in the area of Radiation Protection shall meet the qualification requirements of Regulatory Guide 1.8, September 1975. All other members shall meet the requirements of ANSI/ANS 3.1-1981 for the appropriate discipline. All members shall be appointed in writing by the plant manager. The Vice Chairmen shall be drawn from the SORC members and shall be appointed in writing by the plant manager.

ALTERNATES - All alternate members shall be appointed in writing by the SORC Chairman. Only the designated Vice Chairmen or the plant manager may act as Chairman of a SORC meeting. No more than two alternates to members shall participate as voting members in SORC activities at any one meeting. Alternates for members will not make up part of the voting quorum when the member the alternate represents is also present.

MEETING FREQUENCY - The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

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QUORUM - The quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of this section shall consist of the Chairman or his designated alternate and at least four members including alternates.

RESPONSIBILITIES - The Station Operations Review Committee shall be responsible for:

1. Review of: (1) Upper tier administrative procedures within the scope of Regulatory Guide 1.33 (2/78), and changes thereto; and (2) Newly created procedures or changes to existing procedures that require a 10 CFR 50.59 safety evaluation as described in Section 17.2.1.1.2.1.
2. Review of all proposed tests and experiments that affect nuclear safety.
3. Review of all proposed changes to Appendix "A" Technical Specifications.
4. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
5. Review of the safety evaluations that have been completed under the provisions of 10 CFR 50.59.
6. Initiation or review of investigations of all violations of the Technical Specifications including the reports covering evaluation and recommendations to prevent recurrence.
7. Review of all REPORTABLE EVENTS.

Insert B (cont'd)

8. Review of facility operations to detect potential nuclear safety hazards.
9. Performance of special reviews, investigations or analyses and reports thereon as requested by the SORC.
10. Review of the Fire Protection Program and implementing procedures and changes thereto that require a 10 CFR 50.59 safety evaluation.
11. Review of all unplanned on-site releases of radioactivity to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence.
12. Review of changes to the PROCESS CONTROL MANUAL and the OFF-SITE DOSE CALCULATION MANUAL.

AUTHORITY - The Station Operations Review Committee shall:

1. Provide recommendations in writing to the plant manager indicating approval or disapproval of items considered under the SORC responsibilities above.
2. Provide written notification within 24 hours to the Senior Vice President - Nuclear Operations, and the Director - Quality Assurance/Nuclear Safety Review of disagreement between the SORC and the plant manager; however, the plant manager shall have responsibility for resolution of such disagreements pursuant to Technical Specification 6.1.1.

RECORDS AND REPORTS - The Station Operations Review Committee shall maintain minutes of each meeting and copies shall be provided to the Senior Vice President - Nuclear Operations, the Director - Quality Assurance/Nuclear Safety Review and the Nuclear Review Board.

Test results, including verification of above items, are documented and reviewed for acceptability by the qualified department representative. System tests performed following modifications to O, F, and R-designated systems require review of test procedures and test results by the SORC.

In addition, the Nuclear Administrative Procedures Manual provides for the use of temporary changes which are controlled in accordance with Technical Specifications. Detailed instructions for implementation of temporary changes are provided.

QA monitors the conduct of selected post modification tests to assure compliance with the test procedure. Test results are reviewed for the following:

1. Presentation of proper documentation
2. Assurance that tests meet objectives
3. Identification and reporting of unacceptable results and initiation of corrective measures.

17.2.12 Control of Measuring and Test Equipment

Test equipment, instrumentation, and controls used to monitor and measure activities affecting quality and personnel safety are identified, controlled, and calibrated at specific intervals by cognizant Nuclear Department personnel. Written procedures for meeting these requirements include provisions for:

1. Specifying calibration frequency
2. Recording and maintaining calibration records

TABLE 17.2-1

HOPE CREEK Q ACTIVITIES/SERVICES

The listing below identifies those activities and services, to which the Operational QA program applies during operations:

A. Safety-related activities delineated in Regulatory Guide 1.33, Appendix A (See Regulatory Guide for further breakdown of activities)

1. Administrative procedures

- (a) Security Program, (Regulatory Guide 1.77)
- (b) Equipment Control, e.g., locking and tagging
- (c) Shift and Relief Turnover
- (d) Bypass of Safety Functions and Jumper Control
- (e) Maintenance of Minimum Shift Complement and Call-In of Personnel
- (f) Fire Protection Program (FPP) including Inspection by Fire Consultants
- (g) Communication System.

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- 2. The General plant operating Procedures at HCGS will be called Integrated Operating Procedures (IOPs). Refer to Section 13.5.2.1.2 for a listing of the IOPs.
- 3. The procedures for startup, operation and shutdown of safety related BWR systems at HCGS will be called System Operating Procedures (SOPs). Refer to Section 13.5.2.1.1.
- 4. Offnormal or Alarm Conditions
- 5. The procedures for combating emergencies and other significant events, at HCGS will be broken down into

Insert C

h. Station Operations Review Committee (SORC)

TABLE 17.2-1 (Cont)

two categories. The Emergency Operating Procedures (EOPs) developed from the BWROG Emergency Procedure Guidelines, and the Abnormal Operating Procedures (AOPs) which will cover the additional items in Regulatory Guide 1.33, Appendix A, item 6. Refer to Section 13.5.2.1.3 for a listing of EOPs and AOPs.

6. Control of radioactivity

- (a) Liquid Radioactive Waste System
- (b) Solid Waste System
- (c) BWR Gaseous Effluent System
- (d) Radiation Protection, including Occupational Radiation Exposure per Regulatory Guide 8.8
- (e) Area Radiation Monitoring System Operation
- (f) Process Radiation Monitoring System Operation
- (g) Meteorological Monitoring and Data Collection Program
- (h) Packaging and Transport of Radioactive Material per 10CFR71
- (i) Decontamination.

7. Technical Specification Surveillance

8. Performing maintenance

9. Chemical and Radiochemical Control.

B. Additional NRC requirements

1. Technical Specification Administrative Controls

(a) Station Operations Review Committee (SORC)