PSE&G

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

MAY 0 9 1996

LR-N96119

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

LER 272/95-018-01
SALEM GENERATING STATION - UNIT 1
FACILITY OPERATING LICENSE NO. DPR-70
DOCKET NO. 50-272

This Licensee Event Report Supplement entitled "Improper Range Gauges For Inservice Testing" is being submitted pursuant to the requirements of the Code of Federal Regulations 10CFR50.73(a)(2)(i)(B).

Sincerely,

Clay Warren
General Manager -
Salem Operations

Attachment

SORC Mtg. 96-057

DVH/tcp

C Distribution
LER File 3.7

Attachment A

The following item represents commitments that Public Service Electric & Gas (PSE&G) made to the Nuclear Regulatory Commission (NRC) relative to this LER (272/95-018-01). The commitments are as follows:

1. A stop work order was issued by QA on July 31, 1995 for all IST surveillance testing. The stop work order specified that no testing shall be conducted without prior review and approval by the IST Engineer. The stop work order was lifted on December 12, 1995 with interim work controls in place. The interim controls will remain in place until the procedure revisions stated in Item 4 below are completed.

2. The IST Manual will be revised prior to restart.

3. A programmatic Standard describing conduct of the Salem IST program will be developed and issued prior to restart.

4. Procedure revisions to ensure adequate IST program maintenance and implementation will be completed prior to restart of each Salem unit. This includes Operations procedures implementing Technical Specification 4.0.5, procedures NC.NA-AP.ZZ-0001, NC.NA-AP.ZZ-0070 and SC.TE-TI.ZZ-0028.

NRC FORM 366 (4-95) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 04/30/98

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
SALEM GENERATING STATION, UNIT 1	05000272	1 OF 3

TITLE (4)
IMPROPER RANGE GAUGES USED FOR INSERVICE TESTING

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
07	20	95	95	-- 018 --	01	05	10	96	SALEM, UNIT 2	05000311
									FACILITY NAME	DOCKET NUMBER 05000

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)			20.2201(b)		20.2203(a)(2)(v)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)
			20.2203(a)(1)		20.2203(a)(3)(i)		50.73(a)(2)(ii)		50.73(a)(2)(x)
			20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71
			20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER
			20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A
			20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Dennis V. Hassler, LER Coordinator	609-339-1989

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

In July 1995, while conducting a routine biennial Quality Assurance (QA) audit of the Salem Inservice Testing (IST) Program, a reportable deficiency was identified concerning the failure to use gauges with a proper range for pump testing. This deficiency constitutes a violation of Technical Specification 4.0.5 in that the testing of certain components was not performed in accordance with ASME Section XI requirements. The deficiency involved the failure to use the proper gauges for the testing of 11, 12, 13, 21, 22, and 23 Component Cooling (CC) pumps, 12 and 22 Boric Acid Transfer (BAT) pumps and 12 Residual Heat Removal (RHR) pump. This issue is reportable under 10 CFR 50.73(a)(2)(i)(B), any condition prohibited by the plant's Technical Specifications.

This LER supplement provides an overview of the assessments of the IST program which were performed as one of the corrective actions in LER 95-018-00. These assessments identified deficiencies within the IST Program. However, these deficiencies did not result in reportable conditions.

Corrective actions for the identified IST deficiencies include procedure revisions, an upgrade to the IST manual, and a new programmatic IST standard.

PLANT and SYSTEM IDENTIFICATION:

Westinghouse - Pressurized Water Reactor

Component Cooling Pumps {CC/P}*
Boric Acid Transfer Pumps {CB/P}
Residual Heat Removal Pump {BP/P}

* Energy Industry Identification System (EIIS) codes and component function identifier codes appear in the text as {SS/CC}.

CONDITIONS PRIOR TO OCCURRENCE:

At the time of identification, Salem Units 1 and 2 were in a self-imposed extended shutdown.

Mode: 5 Reactor Power: -0-% Unit Load: -0- MWe

DESCRIPTION OF OCCURRENCE:

In July 1995, while conducting a routine bienni
al Quality Assurance (QA) audit of the Salem Inservice Testing (IST) Program, a reportable programmatic deficiency was identified concerning the failure to use gauges with a proper range for pump testing. This deficiency constitutes a violation of Technical Specification 4.0.5 in that the testing of certain components was not performed in accordance with ASME Section XI requirements.

The deficiency involved the failure to use the proper range gauges for the testing of 11, 12, 13, 21, 22, and 23 Component Cooling (CC) {CC/P} pumps, 12 and 22 Boric Acid Transfer (BAT) {CB/P} pumps and 12 Residual Heat Removal (RHR) {BP/P} pump. ASME Section XI, subsection IWP 4120 requires that, "the full scale range of each instrument be three times the reference value or less." The IST audit concluded that the range of the gauges used for testing in accordance with Technical Specification section 4.0.5 exceeded three times the reference value.

This LER supplement provides an overview of the assessment of the IST program which was performed as one of the corrective actions in LER 95-018-00. An assessment of the IST program was performed as one of the corrective actions stated in LER 95-018-00. The assessment identified deficiencies within the IST Program; however, it was determined that the deficiencies did not result in any additional Technical Specification systems being declared inoperable. Therefore, these deficiencies did not constitute reportable conditions.

CAUSE OF OCCURRENCE

The causes for the IST Program deficiencies, including the use of improper gauges, were an inadequate IST Program, and a lack of adequate IST program maintenance and implementation processes and associated controls.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PRIOR SIMILAR OCCURRENCE

A review of LERs for Salem Units 1 and 2 in the past two years identified two problem areas similar to the IST Program deficiencies. LER 272/95-028-00 addressed a Technical Specifications required program, leakage monitoring of primary coolant outside containment, not being controlled as an integrated program. In this case the elements of the program were being implemented separately. A leakage monitoring program that will meet the Technical Specifications will be implemented prior to entering Mode 4.

The second similar area was a lack of adequate controls for the development and maintenance of Technical Specification surveillance procedures. This concern was addressed in LERs 272/96-005-01, 272/96-005-00, 272/96-004-00, 272/94-008-00, and 311/95-008-00. A Technical Specification Surveillance Improvement Program has been initiated to ensure that surveillances are adequately proceduralized.

SAFETY CONSEQUENCES AND IMPLICATIONS

The IST Program deficiencies did not result in a failure to identify degradation of the affected components nor were any components identified which would have failed to perform their intended safety function. Therefore the health and safety of the public was not affected.

CORRECTIVE ACTIONS

1. A stop work order was issued by QA on July 31, 1995 for all IST surveillance testing. The stop work order specified that no testing shall be conducted without prior review and approval by the IST engineer. The stop work order was lifted on December 12, 1995 with interim work controls in place. The interim controls will remain in place until the procedure revisions stated in Item 5 below are completed.

2. An assessment of the IST program was performed to ensure that programmatic and procedure deficiencies within the IST program were identified and corrective actions initiated.

3. The IST Manual will be revised prior to restart.

4. A programmatic Standard describing conduct of the Salem IST program will be developed and issued prior to restart.

5. Procedure revisions to ensure adequate IST program maintenance and implementation will be completed prior to restart of each Salem unit. This includes Operations procedures implementing Technical Specification 4.0.5, procedures NC.NA-AP.ZZ-0001, NC.NA-AP.ZZ-0070 and SC.TE-TI.ZZ-0028.