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Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

March 15, 1996

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn.: Document Control Desk

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO: 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating report for the month of February is being sent to you.

Sincerely yours,

C. Warren
General Manager -
Salem Operations

RH:vl
Enclosures

C Mr. Thomas T. Martin
Regional Administrator USNRC, Region I
631 Park Avenue
King of Prussia, PA 19046

190053

9603200063 960229
PDR ADOCK 05000272
R PDR

The power is in your hands.

IEB4
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AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272
Unit Name: Salem #1
Date: 03/10/96
Telephone: 339-2735

Completed by: Robert Phillips

Month February 1996

Day Average Daily Power Level
(MWe-NET)

Day Average Daily Power Level
(MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u> </u>
31	<u> </u>

OPERATING DATA REPORT

Completed by: Robert Phillips

Docket No: 50-272
 Date: 03/10/96
 Telephone: 339-2735

Operating Status

1. Unit Name	<u>Salem No. 1</u>	<u>Notes</u>
2. Reporting Period	<u>February 1996</u>	
3. Licensed Thermal Power (Mwt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	<u>N/A</u>	

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
12. Hours in Reporting Period	<u>696</u>	<u>1440</u>	<u>163657</u>
12. No. of Hrs. Rx. was Critical	<u>0</u>	<u>0</u>	<u>104380.5</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>0</u>	<u>0</u>	<u>100388.3</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>318062229.2</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>0</u>	<u>105301000</u>
18. Net Elec. Energy Gen. (MWH)	<u>-2176</u>	<u>-4637</u>	<u>100200276</u>
19. Unit Service Factor	<u>0</u>	<u>0</u>	<u>61.3</u>
20. Unit Availability Factor	<u>0</u>	<u>0</u>	<u>61.3</u>
21. Unit Capacity Factor			
(using MDC Net)	<u>0</u>	<u>0</u>	<u>55.4</u>
22. Unit Capacity Factor			
(using DER Net)	<u>0</u>	<u>0</u>	<u>54.9</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>0</u>	<u>22.9</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)
We are presently in a scheduled extension of a refueling outage.

25. If shutdown at end of Report Period, Estimated Date of Startup:
Under review.

UNIT SHUTDOWN AND POWER REDUCTIONS
 REPORT MONTH February 1996

DOCKET NO.: 50-272
 UNIT NAME: Salem #1
 DATE: 03/10/96
 COMPLETED BY: Robert Phillips
 TELEPHONE: 609-339-2735

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
3824	02-01-96	S	696	C	4	-----	ZZ	ZZZZ	Schedule Extension of Refueling

¹
 F: Forced
 S: Scheduled

²
 Reason
 A-Equipment Failure (explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Continuation of Previous Outage
 5-Load Reduction
 9-Other

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit 1 - Same Source

10CFR50.59 EVALUATIONS
MONTH: FEBRUARY 1996

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: 03/10/96
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY
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1. Design Change Packages (DCP)

1EC-3323, Pkg. 3

“Service Water Pipe Replacement - Service Water Intake Structure Crosstie Piping” Rev. 0 - This package replaces service water (SW) piping associated with Unit 1 located in Intake Bays 1, 2 and 3. It replaces carbon steel cement or coal tar epoxy lined pipe (spec. 27C) with 6% molybdenum austenitic stainless steel pipe (spec. 27F). It maintains the current pipe routing. It changes out portions of the SW crosstie piping for the SW Intake Structure Bays 1, 2 and 3 and portions of the discharge piping from SW pumps 12 and 13. This modification upgrades service water piping which is not discussed in the Technical Specifications. There is no reduction in the margin of safety for any Technical Specification. (SORC 95-108)

1EC-3365, Pkg. 1

“Replacement of VC5 & VC6 Valves” Rev. 1 - The purpose of this DCP is to replace Pressure/Vacuum Relief containment isolation valves 1VC5 and 1VC6 and remove and delete valve 1VC15 and its associated equipment. These modifications reduce the amount of maintenance required to comply with leakage limitations for containment isolation valves 1VC5 and 1VC6, and reduce the seat leakage testing frequency. Valve 1VC15 is being removed because it no longer has a valid function. The function of 1VC15 was to prevent overpressurization of the Pressure/Vacuum Relief System ductwork. It is no longer used for this or any other purpose. There is no reduction in the margin of safety for any Technical Specification. (SORC 95-105)

10CFR50.59 EVALUATIONS
MONTH: FEBRUARY 1996

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: 03/10/96
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

(Cont'd)

ITEM	SUMMARY
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1EC-3450, Pkg. 1

"Containment Spray Support Steel Modification" Rev. 0 - This change modifies structural connections at three different locations on the support steel for the Containment Spray System (CS) Piping. The implementation of this change insures that the Containment Spray piping support framing members will be able to expand, thereby support member stresses and the associated containment liner embedment stresses will remain within design allowables during a main steam line break. These modifications do not impact the performance of the CS system, nor do they reduce the margin of safety that is designed into the system. There is no reduction in the margin of safety for any Technical Specification. (SORC 95-156)

1EC-3540, Pkg. 1

"1SJ1 & 1SJ2 SI RWST To Charging Pump Stop Valves Disc Weep Holes" Rev. 0 - This DCP drills weep holes in the valve discs (Pump side) of the Safety Injection Refueling Water Storage Tank to Charging Pump Stop Valves 1SJ1 and 1SJ2. This modification provides an internal relief path to preclude potential pressure locking due to a pressure buildup in the valves' bonnet cavities and in the space between the valve discs. This is consistent with preventive methods promulgated in NUREG-1275, Vol. 9. The margin of safety as defined in the bases for the Technical Specifications is not affected by this DCP. This modification will not reduce the safety margin of the Safety Injection System and will enhance the reliability of the system by providing an internal relief path for any fluid trapped in the valve bonnet, thus ensuring valve operability. (SORC 96-013)

2. Temporary Modifications (TMOD)

95-056

"Temporary Installation of Surge Monitoring Equipment at 4KV Bus 1H (1SWGR1HD)" - This TMOD installs surge monitoring equipment at the non-safety related 4KV SWGR

10CFR50.59 EVALUATIONS
MONTH: FEBRUARY 1996

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(Cont'd)

ITEM	SUMMARY
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bus 1H spare cubicle 7D in the turbine building. There are no expected effects on any plant systems. Failure of this equipment will not affect any safety related system, structure or component. There is no reduction in the margin of safety as defined in the basis for any Technical Specification.
(SORC 96-014)

3. UFSAR Change Notices (SCN)

SCN 96-04

“Salem RWST Drain Down and Cold Leg Recirculation Evaluation” - This SAR change revises Section 6.3.2.6 and Table 6.3-6 and deletes Table 6.3-8. These changes are being made to address the findings of a detailed plant specific evaluation for Salem Units 1 and 2 cold leg recirculation. This addresses the minimum time available for the operator to proceed through the EOPs to the switch over procedure before receiving a Refueling Water Storage Tank lo-level alarm. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 96-021)

SCN 96-05

“Quality Assurance During the Operations Phase” - These changes reflect the current organization structure and responsibilities and include enhancements and clarifications. These changes are administrative in nature and do not relate to or modify the margin of safety defined and maintained by Technical Specifications. There is no reduction in the margin of safety as defined in the basis for any Technical Specification.
(SORC 96-019)

4. License Change Request (LCR)

94-48

“Revision of Post-LOCA Containment Sump pH Design Basis Range” - This change requires the addition of sodium hydroxide for long term sump pH control, following a Small Break LOCA, which does not actuate spray. The margins of safety for this evaluation are defined by the design basis dose limits, the environmental qualification of safety related equipment inside

10CFR50.59 EVALUATIONS
MONTH: FEBRUARY 1996

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(Cont'd)

ITEM	SUMMARY
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containment and the integrity of ECCS and containment spray components. The new sump pH range discussed in bases Sections 3/4.1.2 and 3/4.5.4 under RWST volume and boron concentration limits, will not affect these margins of safety. The licensing basis radiological dose limits for a design basis LOCA will continue to be met. There is no impact on the current analysis assumptions or results. There is no reduction in the margins of safety as currently defined in the bases for Salem Technical Specifications. (SORC 96-028)

5: Procedures

S1.CH-AD.CN-1143(Q)

“Non-Routine Unit 1 Steam Generator Fill and Chemical Feed” Rev. 0 - This procedure provides a method for filling Unit 1 Steam Generators (SG) using demineralized (DM) water when Auxiliary Feedwater or Auxiliary Chemical Feed Systems are unavailable during Mode 6 or Mode Undefined. This method provides a location to safely add chemicals while filling SGs when auxiliary feedwater or auxiliary chemical feed systems are cleared and tagged or otherwise unavailable. This method will only be performed in Modes 6 or undefined when the steam generators are not in service. Operating the steam generator fill and chemical feed apparatus during these mode restraints will not impact the ability for operations to make up water to the Spent Fuel Pool due to other makeup sources available nor will any pressures or flows exceed system design limits. Due to this redundancy, the required Spent Fuel Pool level needed to maintain $K_{eff} < 0.95$ will not be impacted. There is no reduction in the margin of safety. (SORC 96-020)

Security Plan

“Security Training & Qualification” Rev. 7 - This revision incorporates changes to the management organization and changes reflecting additional responsibilities for access training and processing, background screening and the medical group. There is no reduction in the margin of safety. (SORC 96-022)

10CFR50.59 EVALUATIONS
MONTH: FEBRUARY 1996

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(Cont'd)

ITEM	SUMMARY
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6. Safety Evaluation (S/E)

EACS

“Control Room Emergency Air Conditioning System (EACS) Out of Service” - This work activity disables the Control Room Emergency Air Conditioning System (EACS) in preparation for a subsequent DCP (1EC-3505) which will increase both the capacity and reliability of the system. The work activity affects areas adjacent to the existing EACS system boundaries such that the entire system will be unavailable. Attachment 1 “EACS Pre-DCP Work Process” of this safety evaluation describes the approach taken to perform these work activities in accordance with the Work Control Process (NC.NA-AP.ZZ-0009). The system will not cool, filter, or isolate the control room from the outside environment. It is possible to disable the system at this time because (1) all fuel assemblies from both Salem units are in their respective spent fuel pools, and (2) spent fuel activity has decayed substantially. The proposal does not reduce the margin of safety as defined in the basis for the Technical Specifications. (SORC 96-028)

REFUELING INFORMATION
MONTH: FEBRUARY 1996

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: 03/10/96
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

MONTH : FEBRUARY 1996

. Refueling information has changed from last month: YES . X . NO

. Scheduled date for next refueling: 09/10/95

. Scheduled date for restart following refueling: (to be determined)

. a. Will Technical Specification changes or other license amendments be required?

YES NO

NOT DETERMINED TO DATE . X .

b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES NO . X .

If no, when is it scheduled? (to be determined)

. Scheduled date(s) for submitting proposed licensing action: . n/a .

. Important licensing considerations associated with refueling:

. Number of Fuel Assemblies:

a. Incore . 0 .
b. In Spent Fuel Storage . 1005 .

. Present licensed spent fuel storage capacity: . 1632 .
Future spent fuel storage capacity: . 1632 .

. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September 2008

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
FEBRUARY 1996

SALEM UNIT NO. 1

The Unit is in a refueling outage and remained shutdown for the entire period. Evaluation and assessment of the Steam Generator tube indications continues. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, the Unit will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of each Unit to operate in a safe, event free manner
- After the restart review, meet with the NRC and communicate the results of that review