PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

November 14, 1995

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn.: Document Control Desk

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO: 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of October are being sent to you.

Sincerely yours,

C. Warren
General Manager -
Salem Operations

RH:vls
Enclosures

C Mr. Thomas T. Martin
Regional Administrator USNRC, Region I
631 Park Avenue
King of Prussia, PA 19046

8-1-7.R4

200046

IE24
1/1

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272
Unit Name: Salem #1
Date: 11/10/95
Telephone: 339-2735

Completed by: Robert Phillips

Month October 1995

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

OPERATING DATA REPORT

Docket No: 50-272
Date: 11/10/95
Completed by: Robert Phillips
Telephone: 339-2735

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	October 1995	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity(Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
12. Hours in Reporting Period	745	7296	160753
12. No. of Hrs. Rx. was Critical	0	2660.9	104380.4
13. Reactor Reserve Shutdown Hrs.	0	0	0
14. Hours Generator On-Line	0	2632.1	100388.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	8010326.4	318062229.2
17. Gross Elec. Energy Generated (MWH)	0	2689850	105301000
18. Net Elec. Energy Gen. (MWH)	-2112	2527390	100209554
19. Unit Service Factor	0	36.1	62.4
20. Unit Availability Factor	0	36.1	62.4
21. Unit Capacity Factor (using MDC Net)	0	31.3	56.4
22. Unit Capacity Factor (using DER Net)	0	31.1	55.9
23. Unit Forced Outage Rate	0	56.3	22.9

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

Scheduled refueling outage.

25. If shutdown at end of Report Period, Estimated Date of Startup:

Under review.

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH October 1995

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: 11/10/95
COMPLETED BY: Robert Phillips
TELEPHONE: 609-339-2735

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
3642	10-1-95	S	745	C	4	---------	ZZ	ZZZZZZ	Refueling Outage

1
F: Forced
S: Scheduled

2
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Requlatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

10CFR50.59 EVALUATIONS
MONTH: OCTOBER 1995

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: 11/10/95
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY

1. Design Change Packages (DCP)

The reporting of DCP related 10CFR50.59 evaluations is being modified to address these items only after they have been implemented and turned over to Operations. For the next several months, during this transition phase, we anticipate few new DCPs to report.

1EE-0151, Pkg. 1 — "Protection of Condensate Piping from Overpressurization" Rev. 0 - The purpose of this DCP is to install a 1" x 1" relief valve set at 15 psig on the 1 1/2" threaded connection on the suction strainer (1CNE28) located between the suction nozzle of condensate pump No. 12 (1CNE6) and gate valve (12CN5). The relief valve installation should be typical for strainer (1CNE27) of condensate pump No. 11 (1CNE5) and strainer (1CNE29) for condensate pump No. 13 (1CNE7). The relief valve is required to protect equipment from over pressurization during condensate pump isolation mode. The relief valve is sized to pass back leakage flow for the check valve located downstream of the condensate pump. A lift in the relief valve would visually alert the operator that leakage is occurring and isolation of the system should stop before an over pressure condition occurs in the piping system. These modifications do not reduce the margin of safety because the condensate system is not addressed in the Technical Specifications or their bases. (SORC 95-122)

2. Safety Evaluations (S/E)

W/O 940921184 — "Freeze Seal Of The Component Cooling Water Supply To The Boric Acid Evaporator Condenser Safety Relief Valve Discharge Piping (1CC40)" - The purpose of this proposal is to install a freeze seal on the 1-1/2" discharge piping of the boric acid evaporator condenser safety relief valve, 1CC40, in the

(Cont'd)

ITEM	SUMMARY
	Component Cooling Water (CCW) System. The freeze seal is needed to allow replacement of the 1CC40 relief valve for testing. The installation of a freeze seal will prevent "back leakage" of CCW from the CCW supply line through the relief valve discharge line while the valve is being replaced. A freeze seal failure is unlikely because its installation is controlled by a procedure which has been successfully performed several times at Salem and throughout the industry. In the unlikely event that the procedure fails (which causes the freeze seal to fail), the result would be a minor loss of CCW inventory. This would quickly be mitigated since the written contingency plan requires personnel to be present and instructs them on corrective actions to be taken. Loss of Spent fuel Pool Cooling was considered. The event is not considered to be plausible due to the following barriers: Operators are stationed at the 1CV40 and in communication with the Control Room for the duration of the freeze seal evolution; Ability to cross connect to the Unit 2 Spent Fuel Pool Cooling System; and Alarm Response and Abnormal Operating procedures. There is no reduction in the margin of safety as defined in the bases for any Technical Specifications. (SORC 95-122)
3. Procedures	
SC.CH-SA.WG-0231(Q)	"Waste Gas Decay Tank Sampling" Rev. 2 - Chemistry procedure SC.CH-SA.WG-0231(Q), Rev. 2, directs Chemistry personnel to sample the Waste Gas Decay Tanks (WGDTs) at Unit 1 and Unit 2, Auxiliary Building elevation 64', Panel 108. There are two primary reasons that samples are drawn at this location. One, samples must be drawn and analyzed prior to release from the WGDTs to the environment to verify release limits as prescribed in Tech Spec are not exceeded. Two, samples are taken to determine if there are explosive gas mixtures in the WGDTs when the Waste Gas Analyzer is out of service. The sample location in this procedure is different than the sample location described in Sections 11.3.2 and 11.3.4 in the UFSAR. Samples taken at the Waste Gas Analyzer as

ITEM	SUMMARY

described in the UFSAR, present the possibility of being cross contaminated by other gasses being drawn into the analyzer. Therefore, representativeness of activity from the respective WGDT may not be obtained. However, sampling for oxygen (explosive gas mixture) is not affected by sampling at the WGA. This procedure describes sampling each individual Gas Decay Tank separately by using the specific vent valve 11, 12, 13, 14, 21, 22, 23 or 24WG132 stemming off the pressurized instrument line. Each vent has two isolation valves downstream from the respective WGDT. A pressure regulator is connected to the vent valve using quick disconnects. A sample apparatus is then connected to the outlet of the regulator and the sample is purged into a 5 gallon bottle partially filled with DM water with particulate and iodine filters on the vent side. The design and operation of the WGDTs is not affected. The only change is to the location of the sample points. There is no reduction in the margin of safety as defined in the bases for any Technical Specifications. (SORC 95-124)

4. UFSAR Change Notice (SCN

SCN 91-35

"Containment Penetrations - Over Current Protection Devices" - The proposed change to the UFSAR is to remove the tables listing the containment penetration/conductor over current protective devices (Tables 8.3-4A and 8.3-4B) from the UFSAR and to revise UFSAR Section 8.3.2.4 to include references to Engineering Calculation Numbers ES-13.010(Q) & ES-13.005(Q) for Salem Units 1 and 2 respectively. These calculations contain information with regard to the size, equipment power, setpoint settings, etc. And include references to the location for each device. These calculations are controlled and any revisions to the calculations will be accompanied with a 10CFR50.59 evaluation. This change to the UFSAR is to avoid having to change the UFSAR each time the information in the calculations change. In this proposed UFSAR change notice, no changes are being made to the devices or their descriptions. The only change is the location of

10CFR50.59 EVALUATIONS
MONTH: OCTOBER 1995

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: 11/10/95
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

(Cont'd)

ITEM	SUMMARY

the information with regard to the devices. This change notice also addresses revision to the description of these devices in the Technical Specifications Bases Section 3/4.8 "Electrical Equipment Protective Devices." This revision provides references to the UFSAR for discussions of these devices, and the UFSAR will now provide a discussion as well as references to the Engineering Calculations for details with regard to the devices. There is no reduction in the margin of safety as defined in the bases for any Technical Specification. (SORC 95-127)

REFUELING INFORMATION
MONTH: OCTOBER 1995

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: 11/10/95
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

MONTH : OCTOBER 1995

. Refueling information has changed from last month: YES__X.__NO ___

. Scheduled date for next refueling: 09/10/95

. Scheduled date for restart following refueling: 2nd quarter 1996

. a. Will Technical Specification changes or other license amendments be required?

YES ___ NO ___

NOT DETERMINED TO DATE __X__

b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES _____ NO __X__

If no, when is it scheduled? (to be determined)

. Scheduled date(s) for submitting proposed licensing action: _____N/A_____

. Important licensing considerations associated with refueling:

_____

_____

_____

_____

. Number of Fuel Assemblies:
a. Incore 0
b. In Spent Fuel Storage 1005

. Present licensed spent fuel storage capacity: 1632
Future spent fuel storage capacity: 1632

. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September 2008

8-1-7.R4

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
~~NOVEMBER~~ 1995
October

SALEM UNIT NO. 1

The Unit is in a refueling outage and remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, both Units will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of each Unit to operate in a safe, event free manner
- After the restart review, meet with the NRC and communicate the results of that review