



**PSEG**

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

**Nuclear Business Unit**

October 11, 1995

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Attn.: Document Control Desk

MONTHLY OPERATING REPORT  
SALEM NO. 1  
DOCKET NO: 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of September are being sent to you.

Sincerely yours,

C. Warren  
General Manager -  
Salem Operations

RH:vl  
Enclosures

C Mr. Thomas T. Martin  
Regional Administrator USNRC, Region I  
631 Park Avenue  
King of Prussia, PA 19046

8-1-7.R4

190053

The power is in your hands.

9510190067 950930  
PDR ADDCK 05000272  
R PDR

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272  
 Unit Name: Salem #1  
 Date: 10/10/95  
 Telephone: 339-2735

Completed by: Robert Phillips

Month September 1995

Day Average Daily Power Level  
 (MWe-NET)

Day Average Daily Power Level  
 (MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

OPERATING DATA REPORT

Docket No: 50-272  
 Date: 10/10/95  
 Telephone: 339-2735

Completed by: Robert Phillips

Operating Status

1. Unit Name	<u>Salem No. 1</u>	<u>Notes</u>
2. Reporting Period	<u>September 1995</u>	
3. Licensed Thermal Power (Mwt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	<u>N/A</u>	

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
12. Hours in Reporting Period	<u>720</u>	<u>6551</u>	<u>160008</u>
12. No. of Hrs. Rx. was Critical	<u>0</u>	<u>2660.9</u>	<u>104380</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>0</u>	<u>2632.1</u>	<u>100388.3</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>8010326.4</u>	<u>318062229.2</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>2689850</u>	<u>105301000</u>
18. Net Elec. Energy Gen. (MWH)	<u>-3133</u>	<u>2529502</u>	<u>100211666</u>
19. Unit Service Factor	<u>0</u>	<u>40.2</u>	<u>62.7</u>
20. Unit Availability Factor	<u>0</u>	<u>40.2</u>	<u>62.7</u>
21. Unit Capacity Factor (using MDC Net)	<u>0</u>	<u>34.9</u>	<u>56.6</u>
22. Unit Capacity Factor (using DER Net)	<u>0</u>	<u>34.6</u>	<u>56.2</u>
23. Unit Forced Outage Rate	<u>26.7</u>	<u>56.3</u>	<u>22.9</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)  
Scheduled refueling outage.

25. If shutdown at end of Report Period, Estimated Date of Startup:  
Under review.



10CFR50.59 EVALUATIONS  
MONTH: SEPTEMBER 1995

DOCKET NO: 50-272  
UNIT NAME: SALEM 1  
DATE: 10/10/95  
COMPLETED BY: R. HELLER  
TELEPHONE: 609-339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM

SUMMARY

1. Design Change Packages (DCP)

The reporting of DCP related 10CFR50.59 evaluations is being modified to address these items only after they have been implemented and turned over to Operations. For the next several months, during this transition phase, we anticipate few new DCPs to report.

2. Temporary Modifications (T-Mod)

T-Mod 95-058

"Installation of a Blind flange on 12SE72 Outlet Piping" - The purpose of this modification is to provide a secondary boundary for the leaking 12SW76 isolation valve. This temporary modification will add a blank flange in the 10" line 1-SW-P-1665 on the outlet pipe flange to the removed 12SW72 valve. The supply and return line to Fan Coil Unit 1SWE25 (No. 12 Containment Fan Coil Unit) was isolated and tagged. The tagged valves are 12SE54, supply, and 12SW76, return. Valve 12SW72 was removed for maintenance. Valve 12SW76 will not fully isolate and leaks by. The leakage is being drained through drain valve 12SW425 with the water flow being directed to the floor drain. With the discharge header being placed back into service, the flow through the leaking valve will be too much for the floor drain to handle. Therefore it is necessary to install this blank flange. The blank flange is equivalent to a tagged closed valve. No new equipment configurations are introduced by this T-Mod. The Technical Specifications were reviewed and found to be not applicable for the current mode (undefined) and configuration. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 95-107)

10CFR50.59 EVALUATIONS  
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TELEPHONE: 609-339-5162

(Cont'd)

ITEM	SUMMARY
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### 3. Procedures

- S1.MD-FR.ZZ-0003(Z) "Alternate Power Source for Cafeteria" Rev. 0 - The purpose of this procedure is to provide the instructions necessary to temporarily install power cables between 2HL (280/120 Volt Bus) and breaker 1HL4Y (output side). With breaker 2HL4Y closed and breaker 1HL4Y open, power is provided to cafeteria power panels CPP1 and CPP2 during #11 Station Power Transformer (SPT) outage or when 4160 280/120V Transformer 1XFR1H3DAY is out of service. All equipment affected by the procedure is located within the turbine buildings (U1 and U2). The 1HL/2HL 208/120V AC busses do not supply any equipment important to safety nor do they affect any equipment important to safety. In addition, the loading on the alternate power source (2HL) will be limited to ensure overload/overcurrent conditions do not develop. The 1HL/2HL 208/120V AC busses are not covered by the Technical Specifications. Neither the busses nor the equipment supplied by the busses affect the margin of safety as defined in the bases of the Technical Specifications. (SORC 95-100)
- S1.MD-FR.230-0001(Q) "1F-1H 230 Volt Bus Crossfeed," Rev. 0 - This procedure provides the instructions necessary to install temporary power cables between 1H 230 Volt group bus and 1F 230 Volt group bus during respective Station Power Transformer (SPT) outages or 4160-240/139V transformer outages. During 12SPT outage or 4160-240/139V Transformer 1XFR1F5DBY outage, breaker 1HY is closed and breaker 1FY is open, resulting in 1F 230 Volt bus being powered from 1H 230 Volt bus. During 11SPT outage or 4160-240/139V Transformer 1XFR1H5DBY outage, breaker 1FY is closed and breaker 1HY is open, resulting in 1H 230 Volt bus being powered from 1F 230 Volt bus. Busses 1H and 1F (230V AC group busses) are not covered by Technical Specifications. Neither the busses nor the equipment supplied by the busses affect the margin of safety as defined in the bases of the Technical Specifications. (SORC 95-100)

10CFR50.59 EVALUATIONS  
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TELEPHONE: 609-339-5162

(Cont'd)

ITEM	SUMMARY
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Security Procedure 9

"Control of Packages and Materials," Rev. 0 - The revision deletes the requirement for controlling personnel and material access to the primary reactor containment from within the adjacent vital area. Personnel access control will continue to be applied for access from the protected area to Salem's containments through the equipment hatches. The proposal involves no plant equipment and does not reduce the quality of oversight of the Security Program. There is no reference to the security control of primary reactor containment. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 95-101)

Security Procedure SP7

"Personnel Access Control," Rev. 7 - Security Procedure SP7 is being revised in response to a change to the Federal Regulations - deletion of 10CFR73.55(d)(8). This has eliminated the requirement for controlling personnel and material access to primary reactor containment from within an adjacent vital area. Personnel access control will continue to be applied for access from the protected area to Salem's containments through the equipment hatches. The proposal involves no plant equipment and does not reduce the quality of oversight of the Security Program. There is no reference to the security control of primary reactor containment. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 95-110)

4. Updated Final Safety Analysis Report (UFSAR) Change Notice

SCN 95-36

"Sections 17.2.2 & Table 17.2-1.1.1.1a" - The purpose of this submittal is to change the station's UFSAR by deleting the security program (administrative procedures) from the Q-List in Table 17.2.1, and reference to Regulatory Guide 1.17, Protection of Nuclear Plants Against Industrial Sabotage, (which has been revoked by the NRC) from Section 17.2.2. Regulatory Guide 1.33 Appendix A cites Security & Visitor Control as activities that should be covered by written administrative procedures. There is no reference to the security

10CFR50.59 EVALUATIONS  
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(Cont'd)

ITEM	SUMMARY
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system. Quality requirements applicable to Security are contained in 10CFR73.55 and in the Security Plan (UFSAR Section 13.6). Therefore the Q-Listing of the Security Program is redundant. Its removal from the Q-List will have no effect on the quality or oversight of the Security Program. The proposal involves no plant equipment and does not reduce the quality of oversight of the Security Program. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 95-101)



REFUELING INFORMATION  
MONTH: SEPTEMBER 1995

DOCKET NO: 50-272  
UNIT NAME: SALEM 1  
DATE: 10/10/95  
COMPLETED BY: R. HELLER  
TELEPHONE: 609-339-5162

MONTH : SEPTEMBER 1995

Refueling information has changed from last month: YES X NO     

Scheduled date for next refueling: 07/10/95

Scheduled date for restart following refueling: 1st quarter 1996

a. Will Technical Specification changes or other license amendments be required?

YES      NO     

NOT DETERMINED TO DATE X

b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES      NO X

If no, when is it scheduled? (to be determined)

Scheduled date(s) for submitting proposed licensing action: N/A

Important licensing considerations associated with refueling:

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Number of Fuel Assemblies:

a. Incore

0

b. In Spent Fuel Storage

1005

Present licensed spent fuel storage capacity:

1632

Future spent fuel storage capacity:

1632

Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

September 2008

SALEM GENERATING STATION  
MONTHLY OPERATING SUMMARY - UNIT 1  
OCTOBER 1995

SALEM UNIT NO. 1

The Unit is in a refueling outage and remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, both Units will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of each Unit to operate in a safe, event free manner
- After the restart review, meet with the NRC and communicate the results of that review