

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

September 11, 1995

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Attn.: Document Control Desk

MONTHLY OPERATING REPORT SALEM NO. 1 DOCKET NO: 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of August are being sent to you.

Sincerely yours,

C. Warren General Manager -Salem Operations

RH:vls Enclosures

C Mr. Thomas T. Martin Regional Administrator USNRC, Region I 631 Park Avenue King of Prussia, PA 19046

8-1-7.R4

# 100117

The power is in your hands.

AVERACE DAILY UNIT POWER LEVEL



Docket No.:	50-272
Unit Name:	Salem #1
Date:	09/11/95
Telephone:	339-2735

Completed by: <u>Robert Phillips</u>

Month <u>August 1995</u>

Day Average Daily Power Level (MWe-NET) Day Average Daily Power Level (MWe-NET)

1	0		17	00
2	0		18	0
3	00		19	0
4	0		20	
5	0		21	0
6	0		22	00
7	00		23	0
8	0		24	0
9	0	,	25	0
10	00		26	
11	0		27	0
12	0		28	00
13	00		29	0
14	00		30	0
15	00		31	0
16	0			

#### P. 8.1-7 R1

			Dockot No.	50-272
			Docket No: Date:	09/11/95
Com	pleted by: <u>_Robert Phillips_</u>		Telephone:	
COM	pieced by. <u>_Robert millips</u> _		terephone.	
<u>Ope</u>	rating Status			
1. 2. 3. 4. 5. 6. 7.	Unit Name Reporting Period <u>A</u> Licensed Thermal Power (MWt) Nameplate Rating (Gross MWe) Design Electrical Rating (Net M Maximum Dependable Capacity(Gro Maximum Dependable Capacity (Net If Changes Occur in Capacity Ra Report, Give Reason <u>N/A</u>	$\begin{array}{c} & \underline{1170} \\ \text{IWe} & \underline{1115} \\ \text{oss MWe} & \underline{1149} \\ \text{ot MWe} & \underline{1106} \\ \text{atings (items } \end{array}$	3 through 7)	
9.	Power Level to Which Restricted	l, if any (Net	: MWe)	N/A
10.	Reasons for Restrictions, if an	ıy	N/A	
	·	· · · · · · · · · · · · · · · · · · ·	•	
		<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
	Hours in Reporting Period	744	<u> </u>	159288
	No. of Hrs. Rx. was Critical	0	2660.9	104380.5
	Reactor Reserve Shutdown Hrs.	0	0	0
	Hours Generator On-Line	00	2632.1	100388.3
	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	0	8010326.4	318062229.2
17.	Gross Elec. Energy Generated	······		
	(MWH)	0	2689850	105301000
	Net Elec. Energy Gen. (MWH)	-5500	2532635	100214799
	Unit Service Factor	0	45.1	63.0
	Unit Availability Factor	0	45.1	63.0
21.	Unit Capacity Factor (using MDC Net)	0	39.3_	56.9
22.	Unit Capacity Factor			
	(using DER Net)	0	39.0	56.4
	Unit Forced Outage Rate	100	54.9	22.8
23.		·		ration of oad
	Shutdowns scheduled over next 6	5 months (type	e, date and du	
	Shutdowns scheduled over next 6	5 months (type	e, date and du	
		5 months (type	, date and du	

To be determined.

8-1-7.R2

UNIT SHUTDOWN AND POWER REDUCTIONS REPORT MONTH AUGUST 1995

DOCKET NO.: 50-272 UNIT NAME: <u>Salem #1</u> DATE: <u>9-10-95</u> COMPLETED BY: <u>Robert Phillips</u> TELEPHONE: <u>609-339-2735</u>

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
2671	8-1-95	F	744	D	4		SH	CKTBRK	TECH. SPEC. 3.0.3
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1 F: Forced 2

S: Scheduled

Reason A-Equipment Failure (explain) B-Maintenance or Test C-Refueling D-Requiatory Restriction E-Operator Training & License Examination F-Administrative G-Operational Error (Explain) H-Other (Explain)

3 Method: 1-Manual 2-Manual Scram 3-Automatic Scram 4-Continuation of Previous Outage 5-Load Reduction 9-0ther

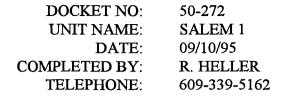
4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5 Exhibit 1 - Same Source



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# 10CFR50.59 EVALUATIONS MONTH: AUGUST 1995



The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

## ITEM SUMMARY

1. Design Change Packages (DCP)

The reporting of DCP related 10CFR50.59 evaluations is being modified to address these items only after they have been implemented and turned over to Operations. For the next several months, during this transition phase, we anticipate few new DCPs to report.

#### 2. Temporary Modifications (T-Mod)

95-028

"Boiler Feed Iron Sampling Modification" Rev. 0 - An iron entrainment sample system, consisting of an isolation valve, a pressure regulator, a flow meter, a heat exchanger, a filter assembly, and interconnecting tubing and fittings, will be connected to the Unit 1 Boiler Feedwater Drain Valve 12BF39. This system will be used to obtain data on iron entrained in the feedwater en route to the 16B Feedwater Heater. This modification is in accordance with PSE&G pipe specs and will meet the pressure and temperature limitations of the system. A second isolation valve will be installed to improve modification isolation. This modification is installed on a secondary plant system with no safety related equipment. The equipment important to safety near this modification is weather tight and will function as per design and would not be impacted from a total failure of the temporary modification. There is no reduction in the margin of safety as defined in the bases for any Technical Specifications. (SORC 95-097)

#### 3. Procedures

SECG/HECG E-Plan "Salem ECG Sections I - 11, Hope Creek ECG Sections I-11, Emergency Plan Section 5" - The Salem and Hope Creek Event Classification Guides are being revised to provide an improved set of Emergency Action Levels (EALs) and Reporting Action

10CFR50.59 EVALUATIONS	DOCKET NO:	50-272
MONTH: AUGUST 1995	UNIT NAME:	SALEM 1
	DATE:	09/10/95
	COMPLETED BY:	R. HELLER
	TELEPHONE:	609-339-5162
(Cont'd)		
ITEM	SUMMARY	

Levels (RALs) which are consistent with the revised NUMARC methodology on emergency classification. In addition to revising all EALs, a new document titled "Event Classification Guide Technical Basis", (ECGTB), is being provided for each ECG which will provide the ECG users with a basis and discussion for each EAL. The ECGTB will also provide the calculations and assumptions used to provide the EAL threshold values. Implementation of the NUMARC EALs does not reduce the effectiveness of the Emergency Plan based on NRC acceptance of the methodology. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 95-090)

#### 4. Safety Evaluations

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SCN 95-037 "Revision of Organization Titles and Duties for Chief Nuclear Officer, Director Quality Assurance and Nuclear Safety Review, Chemistry and Radiation Protection Managers IAW Provisions of T.S. Amendment 168/150" - This UFSAR revision changes the position titles for the Chief Nuclear Officer and President -Nuclear Business Unit and for the Director - Quality Assurance and Nuclear Safety Review. There are no changes to the functional responsibilities for either position. The proposed reorganization splits the Radiation Protection and Chemistry organization into distinct departments (as reflected in Technical Specification Amendments 168/150). This change is anticipated to improve the oversight in both areas. These changes do not conflict with any Technical Specification requirement or reduce the margin of safety required by the Technical Specifications. The specific programs and policies are still required by UFSAR Section 12.3.5, Procedures. (SORC 95-092) SCN 95-39 "Increase in Tech Spec Minimum Volume Required in Diesel

Fuel Oil Storage Tanks (Salem Amendments 170/152)" - The proposed revisions clarify implementation of the design basis for the storage capacity of the diesel fuel oil storage system. This clarification is necessary to resolve a potential deviation

## 10CFR50.59 EVALUATIONS MONTH: AUGUST 1995

DOCKET NO: 50-272 UNIT NAME: SALEM 1 09/10/95 DATE: COMPLETED BY: R. HELLER **TELEPHONE:** 

609-339-5162

#### (Cont'd)

#### ITEM SUMMARY

identified by the NRC during an EDSFI performed at Salem during August and September of 1993. The EDSFI identified that the SAR currently states each 30,000 gallon diesel fuel oil storage tank can supply one EDG with enough fuel oil to operate it for seven days at full load; however, the ability of the as-built system to meet this requirement could not be verified. The proposed revisions are being made to provide an accurate description of the diesel fuel oil storage system design and operation in the SAR and to raise the Technical Specification fuel oil limit to an acceptable level. These changes, and the identified procedure changes, are being made to enable the diesels to operate for seven days at full load. The EDG fuel Oil Storage System meets the design basis goal by supplying approximately 4 1/2 days from the Diesel Fuel Oil Storage Tanks for 2 EDG operation or 2 1/2 days if 3 EDGs are in operation. The EPIP provides an action plan for alternate fuel oil supply to replenish the Diesel Fuel Oil Storage Tanks from on-site storage or offsite sources to ensure the required fuel supply for seven days. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 95-094)

REFUELING INFORMATION MONTH: AUGUST 1995 DOCKET NO: 50-272 UNIT NAME: SALEM 1 DATE: 09/10/95 COMPLETED BY: R. HELLER TELEPHONE: 609-339-5162

#### MONTH : AUGUST 1995

- . Refueling information has changed from last month: YES\_X\_NO \_\_\_\_
- . Scheduled date for next refueling: 07/10/95
- . Scheduled date for restart following refueling: <u>1st quarter 1996</u>
- a. Will Technical Specification changes or other license amendments be required?

YES \_\_\_\_NO \_\_\_\_

# NOT DETERMINED TO DATE X

b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES \_\_\_\_\_ NO \_\_X\_\_\_

If no, when is it scheduled? (to be determined)

Scheduled date(s) for submitting proposed licensing action:	<u>N/A</u>

Important licensing considerations associated with refueling:

	Number of Fuel Assemblies: a. Incore b. In Spent Fuel Storage	
•	Present licensed spent fuel storage capacity: Future spent fuel storage capacity:	<u>    1632</u> <u>    1632</u>
	Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:	September 2008

8-1-7.R4

# SALEM GENERATING STATION MONTHLY OPERATING SUMMARY - UNIT 1 AUGUST 1995

## SALEM UNIT NO. 1

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The Unit is in a refueling outage and remained shutdown for the entire period. According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, both Units will remain shutdown pending completion of the following actions:

- Appropriately address long standing equipment reliability and operability issues
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of each Unit to operate in a safe, event free manner
- After the restart review, meet with the NRC and communicate the results of that review