



**PSEG**

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

**Nuclear Business Unit**

July 14, 1995

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Attn.: Document Control Desk

MONTHLY OPERATING REPORT  
SALEM NO. 2  
DOCKET NO: 50-311

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of June are being sent to you.

Sincerely yours,

John C. Summers  
General Manager -  
Salem Operations

RH:vl  
Enclosures

C Mr. Thomas T. Martin  
Regional Administrator USNRC, Region I  
631 Park Avenue  
King of Prussia, PA 19046

8-1-7.R4

200022

The power is in your hands.

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PDR ADOCK 05000311  
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AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-311  
 Unit Name: Salem #2  
 Date: 07-10-95  
 Telephone: 339-2735

Completed by: Robert Phillips

Month June 1995

Day Average Daily Power Level  
 (MWe-NET)

Day Average Daily Power Level  
 (MWe-NET)

1	<u>1072</u>	17	<u>0</u>
2	<u>1087</u>	18	<u>0</u>
3	<u>740</u>	19	<u>0</u>
4	<u>1037</u>	20	<u>0</u>
5	<u>1076</u>	21	<u>0</u>
6	<u>1073</u>	22	<u>0</u>
7	<u>903</u>	23	<u>0</u>
8	<u>0</u>	24	<u>0</u>
9	<u>0</u>	25	<u>0</u>
10	<u>0</u>	26	<u>0</u>
11	<u>0</u>	27	<u>0</u>
12	<u>0</u>	28	<u>0</u>
13	<u>0</u>	29	<u>0</u>
14	<u>0</u>	30	<u>0</u>
15	<u>0</u>	31	<u>0</u>
16	<u>0</u>		

OPERATING DATA REPORT

Docket No: 50-311  
 Date: 07/10/95  
 Telephone: 339-2735

Completed by: Robert Phillips

Operating Status

1. Unit Name	<u>Salem No. 2</u>	<u>Notes</u>
2. Reporting Period	<u>JUNE 1995</u>	
3. Licensed Thermal Power (MWt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	<u>N/A</u>	

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>720</u>	<u>4343</u>	<u>120216</u>
12. No. of Hrs. Rx. was Critical	<u>167</u>	<u>2468.40</u>	<u>78083.62</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>166.97</u>	<u>2261.60</u>	<u>75229.52</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>528849.6</u>	<u>6807220.8</u>	<u>187781005.0</u>
Gross Elec. Energy Generated (MWH)	<u>174980</u>	<u>2198890</u>	<u>78648898</u>
18. Net Elec. Energy Gen. (MWH)	<u>157494</u>	<u>2053875</u>	<u>74777419</u>
19. Unit Service Factor	<u>23.2</u>	<u>52.1</u>	<u>62.6</u>
20. Unit Availability Factor	<u>23.2</u>	<u>52.1</u>	<u>62.6</u>
21. Unit Capacity Factor (using MDC Net)	<u>19.8</u>	<u>42.8</u>	<u>56.2</u>
22. Unit Capacity Factor (using DER Net)	<u>19.6</u>	<u>42.4</u>	<u>55.8</u>
23. Unit Forced Outage Rate	<u>76.8</u>	<u>35.9</u>	<u>22.3</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)  
To be determined.

25. If shutdown at end of Report Period, Estimated Date of Startup:  
To be determined.



10CFR50.59 EVALUATIONS  
MONTH: JUNE 1995

DOCKET NO: 50-311  
UNIT NAME: SALEM 2  
DATE: 07/10/95  
COMPLETED BY: R. HELLER  
TELEPHONE: 609-339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY
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1. Design Change Packages (DCP)

The reporting of DCP related 10CFR50.59 evaluations is being modified to address these items only after they have been implemented and turned over to Operations. For the next several months, during this transition phase, we anticipate few new DCPs to report.

2. Procedures

TS2.OP-PT.RHR-0001(Q) "RH29 Testing and Cross Checking of Flow Devices" Rev. 0 - The purpose of this special test for the RHR system is in support of the operability determination for 21 & 22 RHR trains. The test essentially comprises the following two parts: 1) To determine the function response of the RHR pump miniflow valves 21RH29 and 22RH29 by conducting a dynamic flow test; and 2.) to obtain/record the flow data for the existing permanent flow orifices at the pump discharge and the downstream orifices in the safety injection headers. In Mode 5, two RHR loops are required to be operable for single failure considerations. Two steam generators with the secondary side water level greater than 5% narrow range will be substituted for the loop being tested. Therefore, the redundancy requirements will be met. Only one loop will be tested at a time and it will not affect the operation of the other train. The operation of one RHR pump will provide adequate flow to ensure mixing, prevent boron stratification, and produce gradual reactivity changes during any boron concentration reductions. These gradual reactivity changes will be within the capability of operator recognition and response consistent with the

10CFR50.59 EVALUATIONS  
MONTH: JUNE 1995

DOCKET NO: 50-311  
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TELEPHONE: 609-339-5162

(Cont'd)

ITEM	SUMMARY
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Technical Specification bases for the RHR system operation. There is no reduction in the margin of safety as defined in the bases for any Technical Specifications.  
(SORC 95-067)

REFUELING INFORMATION  
MONTH: JUNE 1995

DOCKET NO: 50-311  
UNIT NAME: SALEM 2  
DATE: 07/10/95  
COMPLETED BY: R. HELLER  
TELEPHONE: 609-339-5162

MONTH: JUNE 1995

1. Refueling information has changed from last month: YES X NO     

2. Scheduled date for next refueling: (to be determined)

Scheduled date for restart following refueling: (to be determined)

3. a. Will Technical Specification changes or other license amendments be required?

YES      NO     

NOT DETERMINED TO DATE X

b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES      NO X

If no, when is it scheduled? (to be determined)

5. Scheduled date(s) for submitting proposed licensing action: N/A

6. Important licensing considerations associated with refueling:

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7. Number of Fuel Assemblies:

a. Incore 193  
b. In Spent Fuel Storage 556

8. Present licensed spent fuel storage capacity: 1632

Future spent fuel storage capacity: 1632

9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March 2012

SALEM GENERATING STATION  
MONTHLY OPERATING SUMMARY - UNIT 2  
JUNE 1995

SALEM UNIT NO. 2

The Unit began the period operating at 100% power and continued to operate at that level until 6/3 when load was reduced to 50% to perform turbine valve testing. The Unit returned to 100% power following the test. A Unit shutdown was initiated on 6/7 to comply with Technical Specification LCO 3.0.3 due to both residual heat removal systems being declared inoperable. An automatic reactor trip occurred at 23:01 due to the trip of two reactor coolant pumps. Operators were manually shutting down the Unit when a 500KV circuit breaker failed, causing the loss of two 4KV group busses (power to the two reactor coolant pumps). According to commitments from PSE&G and a subsequent confirmatory action letter from the NRC, both Units will remain shutdown pending completion of the following actions:

- Complete the Significant Event Response Team (SERT) review of the reactor trip
- Appropriately address long standing equipment reliability and operability issues
- After defining the work scope and approach to the outages, meet with the NRC to discuss these plans
- After the work is completed, conduct a restart readiness review to determine for ourselves the ability of each Unit to operate in a safe, event free manner
- After the restart review, meet with the NRC and communicate the results of that review