

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Salem Generating Station - Unit 1	DOCKET NUMBER (2) 05000272	PAGE (3) 1 OF 4
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TITLE (4) Use of ten (10) containment air temperature points to determine primary containment average air temperature (both units)

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	15	79	95	004	00	05	18	95	Salem - Unit 2	05000311
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 94%		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)
		20.405(a)(1)(iv)		50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)
		20.405(a)(1)(v)		50.73(a)(2)(iii)	50.73(a)(2)(x)

(Specify in Abstract below and in Text, NRC Form 366A)

LICENSEE CONTACT FOR THIS LER (12)	
NAME Michael J. Pastva, Jr. LER Coordinator	TELEPHONE NUMBER (include Area Code) 609 339-5165

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

Since 5/15/79, PSE&G has used 10 primary containment perimeter locations to determine the primary containment average air temperature to comply with Technical Specification (TS) SURVEILLANCE REQUIREMENT 4.6.1.5, on both Units. In 1992, an internal evaluation determined this practice was an acceptable approach to the surveillance requirement, which specifies use of any 5 of the 10 specified locations. On 3/21/95, PSE&G determined this practice was not in verbatim compliance with the surveillance requirement. This event resulted from interpretative oversight of the verbatim requirement to use any 5 of the specified locations. Procedure revisions have been implemented to satisfy the requirement. The average air temperature value will be obtained from use of 5 primary designated locations. These designated locations have been determined to be representative of containment average air temperature. The long term solution is to revise the existing TS Surveillance 4.6.1.5 to reflect these designated locations. The importance of procedure adequacy to ensure TS compliance will be stressed with appropriate plant groups. The late submittal of this report has been discussed with NRC Region I Management.

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PDR ADDCK 05000272
S PDR

REQUIRED NUMBER OF DIGITS/CHARACTERS
FOR EACH BLOCK

BLOCK NUMBER	NUMBER OF DIGITS/CHARACTERS	TITLE
1	UP TO 46	FACILITY NAME
2	8 TOTAL 3 IN ADDITION TO 05000	DOCKET NUMBER
3	VARIES	PAGE NUMBER
4	UP TO 76	TITLE
5	6 TOTAL 2 PER BLOCK	EVENT DATE
6	7 TOTAL 2 FOR YEAR 3 FOR SEQUENTIAL NUMBER 2 FOR REVISION NUMBER	LER NUMBER
7	6 TOTAL 2 PER BLOCK	REPORT DATE
8	UP TO 18 -- FACILITY NAME 8 TOTAL -- DOCKET NUMBER 3 IN ADDITION TO 05000	OTHER FACILITIES INVOLVED
9	1	OPERATING MODE
10	3	POWER LEVEL
11	1 CHECK BOX THAT APPLIES	REQUIREMENTS OF 10 CFR
12	UP TO 50 FOR NAME 14 FOR TELEPHONE	LICENSEE CONTACT
13	CAUSE VARIES 2 FOR SYSTEM 4 FOR COMPONENT 4 FOR MANUFACTURER NPRDS VARIES	EACH COMPONENT FAILURE
14	1 CHECK BOX THAT APPLIES	SUPPLEMENTAL REPORT EXPECTED
15	6 TOTAL 2 PER BLOCK	EXPECTED SUBMISSION DATE

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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Unit # 1 50-272 95-004-00

Plant and System Identification:

Westinghouse - Pressurized Water Reactor

Energy Industry Identification System (EIIS) codes appear in the text as {xx}

Identification of Occurrence:

Use of Ten (10) Containment Air Temperature Points To Determine Primary Containment Average Air Temperature (Both Units)

Event Date: May 15, 1979

Event Discovery Date: March 21, 1995

Report Date: May 18, 1995

This report was initiated by Incident Report No. 95-273.

Conditions Prior to Occurrence:

Unit 1
Mode 1 Reactor Power 94% Unit Load 1090 MWe

Unit 2
Mode 1 Reactor Power 96% Unit Load 1060 MWe

Description of Occurrence:

Since May 15, 1979, ten (10) primary containment perimeter locations have been used, on both Units, for the arithmetical average to determine the primary containment average air temperature. This practice does not constitute verbatim compliance with Technical Specifications (TS) SURVEILLANCE REQUIREMENT 4.6.1.5, which specifies use of any five (5) of the ten (10) specified locations. On March 21, 1995, this occurrence was confirmed by PSE&G as a TS compliance issue.

The late submittal of this report has been discussed with NRC Region I Management.

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Analysis of Occurrence:

Limitations on containment average air temperature ensure the overall containment average air temperature does not exceed the initial temperature condition assumed in the accident analysis for a loss of coolant accident or steam line break inside containment.

Since May 1979, ten (10) primary containment perimeter locations have been used in accordance with procedure by the P-250 and Doric process computers to determine primary containment average air temperature. This practice conflicts with the specified method to meet the TS surveillance requirement. On October 20, 1992, Nuclear Licensing & Regulation had determined this practice complied with the intent of the TS surveillance requirement. Following the subsequent identification of this occurrence on March 21, 1995, immediate action was taken to satisfy the TS surveillance requirement.

Apparent Cause of Occurrence:

The use of ten (10) locations to determine primary containment average air temperature is attributed to "Management/ Quality Assurance Deficiency", as classified in NUREG-1022, Appendix B. This occurred as a result of interpretative oversight of the verbatim requirement to use five (5) locations (averaging ten locations yields a truer average than averaging five locations).

Prior Similar Occurrence:

Review of documentation did not reveal a prior similar occurrence.

Safety Significance:

This occurrence is reportable pursuant to 10CFR50.73(a)(2)(i)(B), due to failure to comply with the requirements of TS SURVEILLANCE REQUIREMENT 4.6.1.5.

This occurrence had minimal safety significance as use of ten (10) primary containment perimeter locations results in

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Safety Significance: (cont'd)

a more accurate value of primary containment average air temperature, than use of the TS prescribed number of locations.

Corrective Action:

Following discovery, procedure revisions were implemented to satisfy the TS surveillance requirement by requiring use of designated five (5) primary of the ten (10) primary containment perimeter locations to determine primary containment average air temperature. These designated locations were determined by Nuclear Engineering to be representative of containment average air temperature.

The long-term solution, in progress, is to revise the existing TS Surveillance 4.6.1.5 to reflect these designated temperature detector locations.

The importance of procedure adequacy to ensure TS compliance will be stressed with appropriate plant groups.



J. C. Summers
General Manager -
Salem Operations

MJPJ:vs

REF: SORC Mtg. 95-049