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Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

April 13, 1995

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn.: Document Control Desk

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO: 50-311

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of March are being sent to you.

Sincerely yours,

John C. Summers
General Manager -
Salem Operations

RH:vl
Enclosures

C Mr. Thomas T. Martin
Regional Administrator USNRC, Region I
631 Park Avenue
King of Prussia, PA 19046

8-1-7.R4

180642

9504180017 950331
PDR ADOCK 05000311
R PDR

The power is in your hands.

IF24
11

ERAGE DAILY UNIT POWER LEVEL

Completed by: Robert Phillips

Docket No.: 50-311
 Unit Name: Salem #2
 Date: 04-10-95
 Telephone: 339-2735

Month March 1995

Day Average Daily Power Level
 (MWe-NET)

Day Average Daily Power Level
 (MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>153</u>
10	<u>706</u>
11	<u>778</u>
12	<u>938</u>
13	<u>936</u>
14	<u>865</u>
15	<u>928</u>
16	<u>962</u>

17	<u>921</u>
18	<u>849</u>
19	<u>518</u>
20	<u>655</u>
21	<u>1030</u>
22	<u>1018</u>
23	<u>664</u>
24	<u>453</u>
25	<u>447</u>
26	<u>456</u>
27	<u>496</u>
28	<u>446</u>
29	<u>446</u>
30	<u>446</u>
31	<u>446</u>

OPERATING DATA REPORT

Completed by: Robert Phillips

Docket No: 50-311
 Date: 04/10/95
 Telephone: 339-2735

Operating Status

1. Unit Name	<u>Salem No. 2</u>	<u>Notes</u>
2. Reporting Period	<u>March 1995</u>	
3. Licensed Thermal Power (MWt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason <u>N/A</u>		

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744</u>	<u>2160</u>	<u>118033</u>
12. No. of Hrs. Rx. was Critical	<u>572.4</u>	<u>838.4</u>	<u>76453.6</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>542.4</u>	<u>631.6</u>	<u>73599.6</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1275559.2</u>	<u>1412445.6</u>	<u>182386229.8</u>
Gross Elec. Energy Generated (MWH)	<u>396480</u>	<u>426830</u>	<u>76876838</u>
18. Net Elec. Energy Gen. (MWH)	<u>369001</u>	<u>363224</u>	<u>73086768</u>
19. Unit Service Factor	<u>72.9</u>	<u>29.2</u>	<u>62.4</u>
20. Unit Availability Factor	<u>72.9</u>	<u>29.2</u>	<u>62.4</u>
21. Unit Capacity Factor (using MDC Net)	<u>44.8</u>	<u>15.2</u>	<u>56.0</u>
22. Unit Capacity Factor (using DER Net)	<u>44.5</u>	<u>15.1</u>	<u>55.5</u>
23. Unit Forced Outage Rate	<u>27.1</u>	<u>53.0</u>	<u>22.2</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)
None.

25. If shutdown at end of Report Period, Estimated Date of Startup:
N/A.

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH MARCH 1995

DOCKET NO: 50-311
UNIT NAME: Salem #2
DATE: 4-10-95
COMPLETED BY: Robert Phillips
TELEPHONE: 609-339-2735

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
2371	3-1-95	F	201.5	A	4	-----	CB	PUMPXX	NUCLEAR REACTOR COOLANT SEAL
2476	3-11-95	F	10	B	5	-----	HH	PUMPXX	FEEDWATER HEATER DRAIN PUMPS
2425	3-16-95	F	19	B	5	-----	HF	PUMPXX	CIRCULATING WATER PUMPS
2492	3-18-95	F	19.6	F	5	-----	ZZ	ZZZZZZ	MARSH FIRE UNDER 5015 LINE
2433	3-19-95	F	34.1	B	5	-----	HB	PUMPXX	NUCLEAR STEAM GEN. FEEDPUMP # 22
2439	3-23-95	F	209.8	B	5	-----	HB	PUMPXX	NUCLEAR STEAM GEN. FEEDPUMPS # 21

¹
F: Forced
S: Scheduled

²
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File
(NUREG-0161)

⁵
Exhibit 1 - Same
Source

10CFR50.59 EVALUATIONS
MONTH: MARCH 1995

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: 04/10/95
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY
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1. Design Change Packages (DCP)

- | | |
|------------------|--|
| 2EE-0088, Pkg. 1 | "No. 22 Charging Safety Injection Pump Inboard Mechanical Seal Housing Replacement" Rev. 0 - This DCP entails the replacement of a leaking inboard mechanical seal assembly and seal housing on No. 22 Charging Safety Injection (C/SI) pump. The seal housing of the new replacement mechanical seal assembly, which is a later design, does not have jacket water cooling connections or a design requirement for these cooling connections. As noted in a Dresser Pacific (pump manufacturer) letter dated 8/24/84, the later design seal housings were revised to eliminate the cooling water jackets in 1976 and all pumps manufactured since have no seal housing water jackets. At this time, the proposed change to 22 C/SI pump will not change the outboard mechanical seal which still has active connections for cooling the seal housing. The operability and reliability of the C/SI pump is not affected by the installation of the replacement seal assembly. The acceptance criteria for the design basis accidents continues to be met and the margin of safety as defined in the Technical Specifications is unchanged. (SORC 95-025) |
| 2EC-3352, Pkg. 1 | "Waste Gas Setpoint Modification" Rev. 0 - This DCP changes the waste gas setpoints to vendor recommended values. The overall task of this DCP is to implement the necessary setpoint and piping changes to allow stable continuous Waste Gas Compressor operation with full recirculation capability. All plant modifications for this DCP are confined to recalibration of selected Waste Gas instrumentation and rerouting of existing compressor recirculation piping. There are no additional failure modes |

10CFR50.59 EVALUATIONS
MONTH: MARCH 1995

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: 04/10/95
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

(Cont'd)

ITEM	SUMMARY
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introduced by this modification. The Waste Gas System setpoints, compressor recirculation line, compressor output capacity and WGD T storage volume, which are changed by this DCP, are not specifically addressed in the Technical Specifications. There is no reduction in the margin of safety as defined in the bases for any Technical Specification. (SORC 95-028)

2EC-3303, Pkg. 1

"Circulating Water Bearing Lubrication Screen Wash Setpoint & System Changes" Rev. 0 - Rev. 0 - The plant modifications to be accomplished under this DCP include: 1) recalibration of header pressures and flows of the Circulating Bearing Lubrication system and the Screen Wash system, 2) Lubrication Pump and Screen Wash Pump strainer control changed from timed to continuous, 3) replacement of the lubrication water and screen wash pressure gages, and, 4) replacement of control valves with manual valves to simplify the system. The affected components do not change in regard to their function. No new failure modes will be introduced by these modifications. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 95-032)

2. Temporary Modifications (T-Mods)

95-022

"Boric Acid Evaporator Level Transmitter Replacement" Rev. 0 - The T-Mod will replace the level transmitter with one that does not have a sealed reference leg. The modified instrument and process sensing line will have a configuration like the 1LT315 on the Unit 1 Boric Acid Evaporator (BAE). The tubing that will now contain process fluid will be heat-traced to prevent crystallization of the boric acid, ensuring that the line does not get clogged thus rendering the instrument inoperable. The tubing for this part of the BAE is Schedule SP33C, which is non-safety related, non-nuclear, and seismic III. The operation and

10CFR50.59 EVALUATIONS
MONTH: MARCH 1995

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: 04/10/95
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

(Cont'd)

ITEM	SUMMARY
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control of the BAE will not be changed because of this temporary modification. No procedure changes are required. The BAE is non-safety related equipment. Neither the BAE or any associated equipment is addressed any Technical Specifications or their bases. In addition, there is no credible way in which the PASS would be affected by this T-Mod. Thus the margin of safety is unaffected. (SORC 95-027)

3. Safety Evaluations (S/E)

S-2-SW-MSE-0848 "Salem Unit 2 Service Water Pump Fastener Material"
Rev. 0 - This S/E addresses concerns pertaining to fasteners used on four Salem Unit 2 Service Water (SW) pumps. The conditions became apparent through Problem Report No. 950119152 which described the potential for Service Water pumps having stainless steel (SS) fasteners in place of the Monel fasteners originally supplied in the pumps. The fasteners have been found to be acceptable for short term use, as discussed in this evaluation. Therefore, the condition does not reduce the margin of safety as defined in the basis for any Technical Specification. Being that the design integrity of the four applicable SW pumps is maintained and the remaining two pumps have the correct fasteners, operability of the six SW pumps is not affected by this condition. (SORC 95-031)

REFUELING INFORMATION
MONTH: MARCH 1995

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: 04/10/95
COMPLETED BY: R. HELLER
TELEPHONE: 609-339-5162

MONTH: MARCH 1995

1. Refueling information has changed from last month: YES ☐ NO ☒

2. Scheduled date for next refueling: April 6, 1996

Scheduled date for restart following refueling: June 5, 1996

3. a. Will Technical Specification changes or other license amendments be required?

YES ☐ NO ☐

NOT DETERMINED TO DATE ☒

b. Has the reload fuel design been reviewed by the Station Operating Review Committee?

YES ☐ NO ☒

If no, when is it scheduled? April 1996

5. Scheduled date(s) for submitting proposed licensing action: N/A

6. Important licensing considerations associated with refueling:

7. Number of Fuel Assemblies:

a. Incore 193
b. In Spent Fuel Storage 556

8. Present licensed spent fuel storage capacity: 1632

Future spent fuel storage capacity: 1632

9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March 2012

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 2
MARCH 1995

SALEM UNIT NO. 2

The Unit began the period shutdown while No. 21 Reactor Coolant Pump seal replacement was in progress. The repairs were completed and the Unit was synchronized to the grid on 03/09/95. On 03/10/95, the power escalation was held at 81% to perform scheduled post outage testing. In addition, Nos. 22 & 23 Heater Drain Pumps (HDP) were out of service for repairs, delaying power escalation. Repairs to No. 23 HDP were completed and power was increased to 95% on 03/12/95. No. 22 HDP was returned to service and power was increased to 95% on 03/15/95. On 03/16/95, power was reduced to 77% due to the influx of grass and problems with the circulating water pump screens. On 03/18/95, power was administratively limited due to a meadow fire underneath the Keeney Line. The line had been removed from service and required a load reduction due to system loading restrictions. Power was increased to 96% on 03/19/95, and held at 96% to collect state point data. On 03/19/95, power was reduced to investigate problems with No. 22 Steam Generator Feed Pump (SGFP) controls. The Unit was returned to 96% power on 03/21/95, however, No. 21 SGFP speed oscillations occurred and power was reduced to 50% on 03/23/95. The Unit continued to operate at 50% power throughout the remainder of the period, while the investigation into the SGFP control problems continued.