PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Salem Generating Station

July 14, 1994

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of June 1994 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
10CFR50.59 Evaluations
PORV or Safety Valve Challenges
Operating Summary
Refueling Information

Sincerely yours,

General Manager -
Salem Operations

RH:pc

cc: Mr. Thomas T. Martin
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19046

Enclosures

8-1-7.R4

The Energy People

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272
Unit Name: Salem #1
Date: 07/10/94
Telephone: 339-2122

Completed by: Mike Morroni

Month June 1994

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	1009
2	0	18	997
3	0	19	1048
4	272	20	1035
5	672	21	892
6	945	22	826
7	1010	23	486
8	985	24	403
9	943	25	0
10	604	26	0
11	0	27	162
12	0	28	809
13	0	29	993
14	0	30	1069
15	129	31	
16	957		

OPERATING DATA REPORT

Docket No: 50-272
Date: 07/10/94
Completed by: Mike Morroni
Telephone: 339-2122

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	June 1994	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity(Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

		This Month	Year to Date	Cumulative
12.	Hours in Reporting Period	720	4343	145441
12.	No. of Hrs. Rx. was Critical	608.15	2236.23	97368.25
13.	Reactor Reserve Shutdown Hrs.	0	0	0
14.	Hours Generator On-Line	472.85	1726.55	93614.39
15.	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	1289928	5828832	296601146
17.	Gross Elec. Energy Generated (MWH)	411540	1578850	98114820
18.	Net Elec. Energy Gen. (MWH)	382073	1443303	93380856
19.	Unit Service Factor	65.7	39.8	62.8
20.	Unit Availability Factor	65.7	39.8	62.8
21.	Unit Capacity Factor (using MDC Net)	48	30	56.6
22.	Unit Capacity Factor (using DER Net)	47.6	29.8	56.2
23.	Unit Forced Outage Rate	29	50.7	21.9

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

None.

25. If shutdown at end of Report Period, Estimated Date of Startup:

N/A.

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH JUNE 1994

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: 07-10-94
COMPLETED BY: Mike Morroni
TELEPHONE: 339-2122

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
0850	6/01/94	F	76.65	B	4	----------	CC	VALVEX	NUCLEAR OTHER STEAM VALVES
1058	6/10/94	F	116.36	A	3	----------	HA	TRANSF	GENERATOR CURRENT AND POTENTIAL TRANSFORMERS
1209	6/22/94	F	49.03	A	5	----------	HF	PUMPXX	INTAKE GRATING FOULING
1250	6/24/94	S	54.15	B	1	----------	HF	PUMPXX	INTAKE GRATING FOULING

1
F: Forced
S: Scheduled

2
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Requlatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

10CFR50.59 EVALUATIONS
MONTH: - JUNE 1994

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: JULY 10, 1994
COMPLETED BY: R. HELLER
TELEPHONE: (609)339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY

A. Design Change Packages

5EC-3033 Pkg 1

"Nuclear Services Building" - This DCP provides utility tie ins for the Nuclear Services Building being constructed as a "commercial building" in the yard area between Salem and Hope Creek Generating Stations. The building will provide additional office space and equipment storage and lab facilities. This DCP addresses engineering and design issues associated with utility tie ins to plant systems, including domestic water, sanitary sewer, fire protection water, fire alarm, electric power, paging, and telecommunications, as they relate to Hope Creek and Salem plant structures, systems and components. The modified systems are not discussed in any Technical Specification and do not interface with any safety related equipment. Therefore, the margin of safety is not reduced. (SORC 94-049)

1EC-3278 Pkg 1

"Fuel Handling Area Ventilation Modifications" Rev. 2 - This DCP entails modifications to the Unit 1 Fuel Handling Area Ventilation System which are summarized as follows: 1.) Replace backdraft dampers 2.) Remove the inlet guide vane controls for the non safety related dampers. The vanes themselves will be retained for use in airflow balancing and will be fixed in place after balancing 3.) Replacement of the safety related exhaust fan's inlet guide vanes and pneumatic operators as well as controls. 4.) Removal of the non safety related truck bay wall mounted exhaust fan, wiring and controls. In it's place, install a non safety related, seismically supported fixed louvre and a safety related gravity pressure relief damper which will allow outdoor makeup air to enter the building when the supply fan is off or when building negative pressure exceeds the building 2" pressure control band. However, at no

ITEM	SUMMARY
	time will airflow exit the building through this opening. These modifications will not reduce the margin of safety as defined in the basis for any Technical Specification. (SORC 94-050)
1EC-1019 Pkg 1	"Abandonment Of No. 12 CVCS Hold-Up Tank" - The No. 12 CVCS Hold-Up Tank experienced buckling in 1979 as the result of a partial vacuum condition. These are three CVCS Hold-Up Tanks for each of the two Salem units. Vacuum breakers were subsequently installed on all of these tanks. The Design Analysis which forms part of this DCP has determined the acceptability of the abandonment in place of the No. 12 CVCS Hold-Up Tank. DCP 1EA-1019 formally documents abandonment of the tank. Upon implementation of this DCP, TRIS will be revised such that the normal position of the associated valves will all be "Locked Closed". The existing level indication equipment will remain connected and active so that if liquid does seep past any of the boundary valves, alarm notification will be available at the No. 109 Panel. The abandonment of 12 CVCS Hold Up Tank does not reduce the margin of safety as defined in the basis for any Technical Specification. (SORC 94-051)
1EC-3320 Pkg 1	"Condensate & Feedwater Systems Overpressurization Protection Installation" - The purpose of this change is as follows: 1.) To install 3/4" x 1" flanged safety relief valves (11 & 12BF82) on the valve bonnet of check valves 11 and 12BF2. This will involve drilling the flat plate valve bonnets and welding on a flanged pipet connection to mount the relief valves. Route the 1" relief valve outlet piping from relief valves 11 & 12BF82 (Elv. 114'-0") to Columns HJ-13 and G-11 respectively, then down to Elv. 100'-6". Six (6) pipe supports will be required for these piping runs; and 2.) To install 3/4" x 1" flanges safety relief valves (11, 12 & 13CN141 & 142) on the 18" 90 degree elbows on the outlet of 12/15A, B and C low

ITEM	SUMMARY
	pressure feedwater heaters between valves 11, 12 & 13CN24/25 and 29/30. This will involve drilling the 18" elbows and welding on a flanged pipet connection to mount the relief valves. The outlet piping on relief valves 11, 12 and 13CN141 & 142 will not require any pipe supports. These modifications address concerns identified in INPO Operations and Maintenance Reminder (O&MR-302) and INPO Significant Event Report (SER) 02-92. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 94-051)
1EA-1065 Pkg 1	"Moving of Pipe Schedule Break On Drawings" - The purpose of this change is to revise applicable plant drawings (e.g. P&IDs, Arrangement Dwgs.,....) to move the pipe schedule breaks from the outlet of the CV-124 and CV253 valves the inlet of these valves. CV124 and CV253 act as barrier valves between safety related and non-safety related portions of the CVCS system. To move the pipe schedule breaks on applicable drawings to include these valves in the safety related portions would be to conform to ANS/ANSI Standards (See N10.2a-1975/ANS-51.8 Section 2.3.3). These valves are presently being purchased as safety related/seismic class 1 (valve mark number AA-28). Also, a work history review for these valves shows that maintenance has, and continues to be, performed to safety related /seismic class 1 standards. Shifting pipe schedule breaks to include CV124 and CV253 in safety related sections of piping will only move plant requirements in a more conservative direction. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 94-051)

10CFR50.59 EVALUATIONS
MONTH: - JUNE 1994

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: JULY 10, 1994
COMPLETED BY: R. HELLER
TELEPHONE: (609)339-5162

(cont'd)

ITEM	SUMMARY

B. Procedures and Revisions

NC.NA-AP.ZZ-0024(Q) "Radiation Protection Program" - This is a limited revision to clarify the scope of certain monitoring requirements, to incorporate recent NRC guidance concerning training and monitoring requirements, and to clarify certain administrative requirements. The changes to this procedure do not relate to design criteria, specifications or operation of the fuel cladding, RCS boundary, or containment and does not address any margin of safety as defined in the bases for the Technical Specifications. Therefore, there is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 94-051)

C. Temporary Modifications

TMR 93-109 "Upgrade Hoisting Capacity of the Fuel Handling Crane From 5 tons to 20 Tons" - This T-Mod is in support of DCP 1EC-3252, and will install a new trolley and hoist for the removal and installation of racks. The existing trolley and hoist will be used for moving the spent fuel. This temporary modification to the fuel handling crane is to be done at the start of the rerack operation. No welding or permanent attachments are made to the crane. No existing crane components are to be removed. This modification will not reduce the margin of safety as defined in the basis for any Technical Specification. (SORC 94-049)

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
JUNE 1994

SALEM UNIT NO. 1

The Unit began the period shutdown for repairs to a pressurizer safety valve and replacement of an Individual Rod Position Indicating (IRPI) system step counter. The Unit entered Mode 2 "Startup" on June 2, 1994, and was synchronized on June 4, 1994. Power was increased to 100% at 02:14 on June 7, 1994, and was later reduced to 90% at 23:00 the same day to repair a heater drain pump. The repairs were completed on June 10, 1994, and power was being increased when a generator/reactor trip occurred due to a failed generator voltage regulator potential transformer. The transformer was replaced and the Unit was synchronized on June 15, 1994. The Unit achieved full power on June 17, 1994. On June 17, 1994, power was reduced to 90% to repair a heater drain tank level control problem. The repairs were completed and the Unit returned to full power on June 19, 1994. Power was briefly reduced to 84% on June 21, 1994 due to high condenser back pressure. Power was further reduced to 55% on June 23, 1994, and the Unit was taken off line on June 24, 1994, for circulating water inlet area dredging. The dredging was completed in front of Unit 1 and the Unit was synchronized on June 27, 1994. The Unit continued to operate at essentially full power throughout the remainder of the period.

REFUELING INFORMATION
MONTH: - JUNE 1994

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: JULY 10, 1994
COMPLETED BY: R. HELLER
TELEPHONE: (609)339-5162

MONTH JUNE 1994

1. Refueling information has changed from last month:
   YES X NO _____

2. Scheduled date for next refueling: APRIL 8, 1995

3. Scheduled date for restart following refueling: JUNE 6, 1995

4. a) Will Technical Specification changes or other license amendments be required?:
   YES _____ NO _____

   NOT DETERMINED TO DATE X

   b) Has the reload fuel design been reviewed by the Station Operating Review Committee?:
   YES _____ NO X

   If no, when is it scheduled?: MARCH 1995

5. Scheduled date(s) for submitting proposed licensing action:
   N/A

6. Important licensing considerations associated with refueling:

7. Number of Fuel Assemblies:

   a. Incore 193

   b. In Spent Fuel Storage 732

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4