

**FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES**

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
2. Plant: Salem Generating Station P.O. Box 236,  
Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit: N/A.
7. Examination Dates: 08-17-92 through 02-02-94.
8. This report covers the examinations that were conducted  
during the 11th Refueling Outage.
9. Components Inspected.

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
#1 Reactor Vessel	Combustion Engineering	66201 Head 66101 Vessel	N/A	20757
#11 Steam Generator	Westinghouse Tampa Div. P.O. Box 19218 Tampa, FL 33616	1003	N/A	68-10
#12 Steam Generator	" "	1022	N/A	68-08
#13 Steam Generator	" "	1023	N/A	68-09
#14 Steam Generator	" "	1203	N/A	68-51
#1 Regen. Ht. Exch.	Sentry Equip. Corp. Oconomowoc, WI	4195-A4 7, 8, 9	N/A	385, 386, 387
#1 Rx. Coolant Filter	Commercial Filters, Corp. Lebanon, IN	101238	N/A	1193

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9. Components Inspected (cont'd):

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
#1 Pressur- izer	Delta Southern	1011	N/A	68-8
Chemical Volume & Control Piping System	United Engineers & Constructors (UE&C) 30 South 17th St., Phila. PA 19101	N/A	N/A	N/A
Component Cooling Sys.	UE&C	N/A	N/A	N/A
Containment Spray Piping System	UE&C	N/A	N/A	N/A
Mainsteam Piping sys.	UE&C	N/A	N/A	N/A
#11 RHR Heat Exchanger	Engineers & Fabricators, Inc.	S15860-C	N/A	122

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9. Components Inspected (cont'd):

COMPONENTS OR APPURTENANCE	MANUFACTURER OR INSTALLER	MANUFACTURER OR INSTALLER SER. NO.	STATION OR PROVINCE NUMBER	NATIONAL BOARD NO.
Pressurizer Relief Piping Sys.	UE&C	N/A	N/A	N/A
Pressurizer Spray Sys.	UE&C	N/A	N/A	N/A
Reactor Coolant Piping Sys	UE&C	N/A	N/A	N/A
Residual Heat Removal System	UE&C	N/A	N/A	N/A
Service Water System	UE&C	N/A	N/A	N/A
Steam Gen. Feed Piping System	UE&C	N/A	N/A	N/A
Safety Injection Piping Sys.	UE&C	N/A	N/A	N/A

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  10. Abstracts of Examinations:
- 1.0 This report also contains augmented examinations required by the Salem Technical Specification, Regulatory Guides, Circulars and Bulletins issued by the United States Nuclear Regulatory Commission.

Examinations were conducted by PSE&G as well as companies under contract to PSE&G. The following is a brief summary with further details found in the attached report and on file at the Salem Generating Station.

Since nearly all inservice examination requirements can be found in Salem's Technical Specifications, this summary lists these requirements first arranged by Technical Specification paragraph number followed by applicable NRC Circulars and Bulletins.

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11. Examination Summary

11.1 Technical Specification 4.0.5 - ASME XI (1983 Edition)

During the Fall 1993 refueling outage, PSE&G, MQS and SwRI personnel performed nondestructive examinations (NDE) of selected Class 1, Class 2 and Class 3 components at the Salem Unit 1 Generating Station. These examinations constituted the fourth inservice examination (ISI) of the second inspection interval of commercial operation.

11.1.1 ISI Examination

Southwest Research Institute (SwRI) under the direction of the Salem ISI Group conducted seventy-six (76) manual ultrasonic, one hundred and four (104) liquid penetrant, and fourteen (14) magnetic particle examinations. Examinations were conducted on the following systems:

Class 1

Vessels

- Pressurizer
- RPV
- Steam Generator #11 through #14

Piping

- Chemical and Volume Control
- Reactor Coolant System
- Pressurizer Relief System
- Pressurizer Spray System
- Residual Heat Removal System
- Safety Injection System

Pumps

- Reactor Coolant Pumps #12 and #14

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11. Examination Summary (Cont'd)

Integrally Welded Attachments

- Reactor Coolant Pumps #12 and #14
- Safety Injection

Class 2

Vessels

- Head Tank #11
- Reactor Coolant Filter
- Regenerative Heat Exchanger

Piping

- Main Steam System
- Steam Generator Feed System
- Containment Spray System
- Residual Heat Removal System
- Pressurizer Relief System
- Chemical Volume Control System
- Safety Injection System

Integrally Welded Attachments

- Residual Heat Removal System
- Safety Injection System
- Main Steam System

The following nonconformance was reported by SWRI:

A PT examination on weld 4-SJ-1172-5 (Summary No. 131200), revealed six (6) linear recordable indications. CNF 1-1 was issued by SWRI and dispositioned by PSE&G as follows:

- After cosmetically blending these indications, a PT re-exam was performed with no linear indications identified.

There were no other unacceptable indications identified.

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11. Examination Summary (cont'd):

11.1.2 Visual Examination on Valves and Bolting

The Salem ISI Group conducted twenty-nine (29) Class 1 bolting exams and one (1) Class 1 valve internal exam.

There were several Class 1 Bolting exam anomalies noted, including boric acid accumulation, nuts seized on the stud or bolt, etc.. Work orders and/or deficiency reports were generated to document and correct all anomalies.

NOTE: No additional exams were required.

Examinations were conducted on the following systems:

Valves

- Pressurizer Relief System
- Safety Injection System
- Residual Heat Removal System
- Chemical Volume and Control System
- Pressurizer Spray System

There were no unacceptable indications identified.

11.1.3 Visual Examination of Supports

The Salem ISI Group conducted and/or coordinated visual examinations on seventeen (17) Class I, five (5) Class II and one hundred forty eight (148) Class III supports.

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11. Examination Summary (cont'd):

Examinations were conducted on the following systems:

Auxiliary Feedwater	Main Steam
Chemical & Volume Control	Pressurizer Relief
Component Cooling	Pressurizing
Containment Spray	Safety Injection
Residual Heat Removal	Service Water

There were no unacceptable indications identified.

11.1.4 System Pressure Tests

System leakage exams were performed in accordance with ASME Section XI on all Class I systems.

A system leakage test was performed with the Reactor Coolant System in mode 3 (normal operating temperature and pressure) with the insulation installed. No pressure boundary leakage was detected. Minor mechanical leakage was identified and work requests were initiated for repairs.

Thirty two (32) System Pressure Tests (SPT) were completed during a System Functional or Inservice Test on Nuclear Class 1, 2 and 3 components in accordance with ASME Section XI. No pressure boundary leakage was detected. Work requests were initiated to correct minor mechanical leakage that was identified.

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11. Examination Summary (cont'd):

- 11.2 Preservice Examinations were performed on the #11 through #14 SJ17 valve replacement welds per DCP 1EO-2309.

The welds that were replaced to facilitate the valve replacement are listed below:

ISIC SUMMARY #	ISIC COMPONENT ID	PSE&G WELD #
174900	1½-SJ-1142-49R	RH-3282-77F-A
175001	1½-SJ-1142-50R	RH-3282-76F-A
175101	1½-SJ-1142-51R	RH-3282-75F-A
175500	1½-SJ-1132-39R	RH-3295-111F-A
179601	1½-SJ-1132-40R	RH-3295-110F-A
179701	1½-SJ-1132-41R	RH-3295-109F-A
184600	1½-SJ-1122-41R	RH-3269-43F-A
184700	1½-SJ-1122-42R	RH-3269-42F-A
184801	1½-SJ-1122-43R	RH-3268-41F-A
190100	1½-SJ-1112-49R	RH-3256-4F-A
190201	1½-SJ-1112-50R	RH-3256-3F-A
190301	1½-SJ-1112-351	RH-3256-2F-A

The PT (PSI Baseline) was performed by MQS with no discrepancies noted.

Preservice Examination was performed on Safety Injection System piping welds, due to the replacement of two (2) orifices per DCP 1EC-3260.

The welds that were replaced to facilitate the replacement of the orifices are listed below:

ISIC SUMMARY #	ISIC COMPONENT ID	PSE&G WELD #
386600	2-SJ-2126-6R	SI-5-256B-A
386620	2-SJ-2126-7R	SI-5-257B-A
387100	2-SJ-2123-6R	SI-1-211C-B
387120	2-SJ-2123-7R	SI-1-212C-A

The PT (PSI Baseline) was performed by MQS with no discrepancies noted.

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11. Examination Summary (cont'd):
- 11.3 Technical Specification 4.4.6.0 - Steam Generator Tube  
Eddy Current Inspection.

11.3.1 Overview

The original scope of Salem 1R11 outage was comprised of 20% bobbin and 20% top of hot leg tubesheet in all four Steam Generators. In addition, a twelve hour window was allotted for RPC testing of dented hot leg tube support plate intersections.

11.3.2 Top of Tubesheet RPC Inspection

The tube expansion in the Steam Generators at Salem Unit 1 were made using the WEXTEx process (explosive expansions). History of Wextex units has shown some cracking in the expansion transition. Cracking of the WEXTEx region can be axial, circumferential or oblique in orientation as detected by RPC. The extent of circumferential cracking and the extent to which an axial indication extends into the transition zone are critical measurements.

Due to indications found, the initial scope was expanded by a total of 612 tubes in Steam Generator #11. One circumferential and seven axial indications were detected in this Steam Generator. None of these indications exceeded the structural integrity limits. Three of these indications were OD initiated. These indications are believed to have been caused by ODSCC rather than residual stress from the WEXTEx expansion process.

There were no expansions of Top of Tubesheet RPC inspections in #12 Steam Generator. 708 tubes were inspected which represents 20.9% of the Steam Generator.

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11. Examination Summary (cont'd):

Steam Generator #13's scope was expanded to 100% due to an indication found in tube R14C39. This tube had a 0.448" axial indication. The entire length of this indication was contained within the WEXTEx transition, therefore the structural integrity limit of 0.41" was exceeded, prompting the 100% expansion of this Steam Generator. This tube also contained two circumferential indications at the top of the tubesheet. Evaluation by Westinghouse determined that this tube required stabilization. One additional tube in this Steam Generator was evaluated for stabilization and determined that no stabilization was required.

Steam Generator #14 had no expansions of the Top of Tubesheet RPC inspection. A total of 708 tubes were inspected which represents 20.9% of the Steam Generator.

11.3.3 Bobbin Inspection

There were five different types of degradation detected during the bobbin inspection. Four of these mechanisms were previously identified to some degree in Unit 1. They are:

AVB WEAR

All four Steam Generators have had some wear at the anti-vibration bars. Steam Generator #11 had two tubes plugged because of this wear.

COLD LEG THINNING

All four Steam Generators have had thinning at peripheral tube cold leg support plate intersections. Steam Generator #14 had one tube plugged because of this mechanism.

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11. Examination Summary (cont'd):

OD INDICATIONS AT NON-DENTED SUPPORT PLATES

These indications are usually distorted on bobbin and require further clarification by RPC testing. Typically, there is a deposit present, which adds to the distortion. This deposit is usually responsible for the corrosion occurring at these intersections. When these indications confirm, the RPC testing reveals a oval-shaped, pit-like signal. Steam Generators #11 and #12 plugged one tube each for this mechanism. Steam Generator #13 had four tubes plugged for this mechanism.

OD INDICATIONS AT THE TOP OF TUBESHEET

These indications are similar to the support plate indications mentioned above, in that the signals are usually distorted by the presence of a deposit. Again, this deposit is most likely responsible for the corrosion occurring. As mentioned, three of these signals were deemed pluggable during the RPC inspection. In addition, several signals in both Steam Generators #11 and #13 were called INR (Indication not Recordable), during the RPC inspection. INR was used since these signals did not fit any of the allowable RPC indications (SAI, MAI, SCI, MCI, SOI and MOI). These INR signals were also bobbin tested. In most cases, these signals were found to be less than a 20% through wall dimension, however, two more tubes in Steam Generator #11 were plugged for this mechanism as well as two tubes in Steam Generator #13.

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11. Examination Summary (cont'd):

One new degradation mechanism was identified during 1R11. This was ID INITIATED INDICATIONS AT DENTED SUPPORT PLATE INTERSECTIONS. These indications can be detected by bobbin when they extend greater than 0.2" beyond the edge of the support plate. These indications can be attributed to residual stress from the dent. The shape rather than the size of the dent appears to be more of a factor in causing these indications. The sizes of the dents associated with these indications are relatively small when compared to the population of dents present in Unit 1 Steam Generators. The first indication of this type was found in Steam Generator #12 tube R33C43 at 1H (1st support plate hot leg side). RPC of this indication revealed that it was 1.59" long, extending from 0.91" above the support plate to 0.68" below. This indication prompted 100% bobbin testing in Steam Generator #12 from 7H to TEH (tube end hot). This expansion revealed two more indications tube R8C4 at 1H and tube R28C85 at 2H. RPC of tube R8C4 showed an MAI (multiple axial indications). One of the axials was 1.102" in length starting at 0.74" above the support plate. The other axial was 1.050" in length starting at 0.66" above the support plate. RPC of tube R28C85 showed and SAI 0.432" long starting at 0.68" above the support plate.

These indications in Steam Generator #12 prompted 100% bobbin testing in the other three Steam Generators from 7H to TEH. No more indications of this type were found in Steam Generators #11 and #14. However, two indications were flagged in Steam Generator #13 for RPC testing. RPC of tube R30C42 at 3H revealed an SAI 1.032" long starting at 0.26" above the support plate and ending at 1.292" below the support plate.

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11. Examination Summary (cont'd):

RPC of tube R3C59 at 1H revealed an MAI. One of the axials was 1.742" long starting at 0.87" above the support plate. The other axial was 0.591" long, starting at 0.74" above the support plate. Both of these indications were determined to be OD indications rather than ID.

RPC OF DENTED SUPPORT PLATE INTERSECTIONS

Twelve hours were allocated for sample testing of dented support plate intersections in each Steam Generator. The 300 largest dents in each Steam Generator were selected for testing. The aim of this testing is to detect the presence of circumferential indications. Circumferential oriented indications can not be seen by bobbin probes. Due to time constraints and restrictions encountered within the tubes, none of the Steam Generators completed the 300 tube program. The number of intersections inspected, ranged from 122 to 170. No indications were detected in this inspection.

11.4 Current Status of the Steam Generators

At this time the total number of tubes plugged in the Salem Unit 1 Steam Generators are as follows:

Generator #	11	12	13	14
Tubes previously plugged	104	121	112	143
New tubes plugged this outage	12	4	10	2
Tubes Plugged	116	125	122	145

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11. Examination Summary (cont'd):

NOTE: The first five and the last five tubes of Row 1 in each of the Steam Generators were mechanically plugged due to a vibration in the tube lane blocking device. The remaining Row 1 tubes were preventively plugged due to a wear problem on the tubes caused by a vibration in the tube lane blocking device. The remaining Row 1 tubes were preventively plugged due to the onset of stress corrosion cracking in the U-bend region as evidenced in two of the four Steam Generators.

11.5 Technical Specification 4.6.1.2d, Containment Type "B" (Penetrations) and Type "C" (Valves) Leak Rate Testing.

11.5.1 Summary

The PSE&G Research Corp. under the field supervision of the Salem ISI Group, conducted local leak rate tests on sixty three (63) Type "B" Electrical Penetrations, seven (7) Type "B" Mechanical Penetrations and fifty five (55) Type "C" Mechanical Penetrations (113 valves).

11.5.2 Results

- A. Several Type "C" valves exceeded the recommended leak rate acceptable criteria. These valves were worked to an acceptable condition and retested satisfactorily.
- B. A total As-Found leakage (min path) of 15,338.5 SCCM was measured through all Type "B" and "C" containment penetrations. After corrective maintenance, a total As-Left (max path) of 26,372.0 SCCM was measured through all Type "B" and "C" penetrations.

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11. Examination Summary (cont'd):

11.6 Technical Specification 4.6.1.3b Elevation 100' and  
130' Airlock Leak Rate Tests

11.6.1 Summary

PSE&G Research Corporation under the field supervision of the Salem ISI Group conducted leak rate tests on the 100' and 130' containment personnel airlocks.

11.6.2 Results

The combined As-Found leakage for these airlocks was 4,925.0 SCCM. After maintenance, the combined As-Left leakage for the airlocks was 5,860.0 SCCM. Both of these measured combined leakage rates satisfies the Technical Specification acceptance criteria of  $\leq 0.05$  La (10,812 SCCM).

11.7 Technical Specification 4.7.9.(a) "Visual Inspection of  
Hydraulic and Mechanical Snubbers"

11.7.1 Summary

The Salem ISI Group conducted visual examinations on twenty four (24) hydraulic and one hundred and eleven (111) mechanical snubbers.

11.7.2 Results

- A. There were no discrepancies noted which would affect snubber operability. There were some minor items noted including missing washers, loose spherical bearings and binding around the bearings and some light corrosion. These conditions were all noted and corrected.
- B. All snubbers which were removed and reinstalled received a satisfactory "As-Left" visual examination.

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11. Examination Summary (cont'd):

11.8 Technical Specification 4.7.9 (c) "Functional Testing  
of Selected Hydraulic and Mechanical Snubbers"

11.8.1 Summary - Hydraulic Snubbers

A. Per Technical Specifications, 10% of the installed hydraulic snubbers as well as all previous outage functional test failures are required to be tested. The 10% sample included one 200 KIP Main Steam snubber and two 1000 KIP Steam Generator snubbers. There were no previous outage test failures.

B. The 10% sample of hydraulic snubber (small size) included one (1) Lisega 3018.

There were no noted discrepancies in the Technical Specification sample.

11.8.2 Summary - Mechanical Snubbers

Per Technical Specifications, 10% of the total installed mechanical snubbers as well as previous outage functional test failures were tested utilizing the WYLE Model 150 Snubber Testing machine. The 10% sample is broken down by snubber category size (small, medium and large) with 10% from each size selected each outage. All selected snubbers are detailed below:

A. The 10% sample of the medium size snubbers included two (2) PSA-1's and nine (9) PSA-3's.

B. The 10% sample of the large size snubbers included one (1) PSA-35.

C. Due to previous outage failures, 100% of the small size snubbers (PSA-1/4's, PSA-1/2") are tested each outage. This resulted in tests being performed on one Model 1/4 and three Model 1/2.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
2. Plant: Salem Generating Station P.O. Box 236,  
Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit N/A.
7. Examination Dates: 08-17-92 through 02-02-94.

11. Examination Summary (cont'd):

11.8.3 Results - Mechanical Snubbers

- A. One medium size Technical Specification sample snubber failed the functional test resulting in a sample expansion of eleven (11) additional tests.
- B. All the small size snubbers were tested satisfactory.

11.9 Technical Specification 4.4.11.1 Reactor Coolant  
Pump Flywheel Examination

11.9.1 Summary

Southwest Research Institute (SwRI) under the direction of the ISI Group performed an in-place ultrasonic examination (UT) on Reactor Coolant Pump (RCP) Flywheels SN 3S-76P917 and SN 2S-76P917. Flywheel examinations are tracked by the RCP Motor serial number and original pump location.

There were no unacceptable indications identified.

11.10 NRC Bulletin 82-02 "Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of Power Plants".

11.10.1 Summary

Visual (VT) and Magnetic Particle (MT) exams were performed on the #11 through #14 Steam Generator (inlet & outlet) and Pressurizer Primary Manway Bolts. Also #11 through #14 Reactor Coolant Pumps main flange bolting was visually inspected.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
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Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit N/A.
7. Examination Dates: 08-17-92 through 02-02-94.

11. Examination Summary (cont'd):

There were several S/G (inlet & outlet) Primary Manway washers replaced, due to deep galling. These washers were replaced in kind. A VT revealed wastage on the shank of one bolt on #14 S/G Outlet manway. This bolt was replaced. There were several PZR Primary Manway bolts replaced, due to non service induced anomalies found during the VT.

NOTE: 100% of all the S/G and PZR manway bolts were examined during this refueling outage.

There were no other unacceptable indications identified.

11.11 NRC Bulletin 88-08 "Thermal Stresses in Piping Connected to Reactor Coolant System".

11.11.1 Summary

Southwest Research Institute (SwRI), and MQS under the direction of the Salem ISI Group, performed augmented ultrasonic and dye penetrant examinations on nine (9) 1.5" Safety Injections welds, two (2) 2" Chemical Volume Control welds and two (2) 3" Chemical Volume Control welds.

NOTE: Seven (7) components had the 88-08 exams waived, due to the replacement of the 11 through 14 SJ17 valves (DCP 1EO-2309). The associated replacement welds were not ultrasonically examined as required by NRC Bulletin 88-08. NRC Bulletin 88-08 is concerned with a service induced condition, which would not exist with these new welds.

There were no unacceptable indications identified.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
2. Plant: Salem Generating Station P.O. Box 236,  
Hancock's Bridge, N.J. 08038
3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit N/A.
7. Examination Dates: 08-17-92 through 02-02-94.

11. Examination Summary (cont'd):

11.12 NRC Bulletin 88-11 "Pressurizer Surge Line Thermal Stratification".

11.12.1 Summary

The Salem ISI group performed visual examinations on all existing surge line pipe supports / HEBA restraints and their associated insulation. A total of ten (10) supports were examined.

There were no unacceptable indications identified.

11.13 NRC Bulletin 87-01, "Thinning of Pipe Walls in Nuclear Power Plants".

11.13.1 Summary

Measurements were taken at one hundred and sixty four (164) locations to identify possible pipe erosion/corrosion. Of the 164 locations, 147 were from the original scope and 17 were the result of significant wall thinning found in 15 components. Fifteen (15) components were cut out and replaced and one (1) component received external weld "build-up". Other than the aforementioned areas, no other conditions were observed.

11.14 Field Directive, S-C-R700-MFD-288, "Leak Rate Test Program for the Steam Generator Blowdown Isolation Valve (GB4)

11.14.1 PSE&G Research and Testing Lab under the field supervision of the Salem ISI Group, conducted leak rate tests on all four Steam Generator Blowdown Valves (GB-4's) in accordance with procedure SC.SS-IS.GB-0004(Q).

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
Newark, N.J. 07101
  2. Plant: Salem Generating Station P.O. Box 236,  
Hancock's Bridge, N.J. 08038
  3. Plant Unit 1
  4. Owner's Certificate of Authorization (if required) N/A.
  5. Commercial Service Date 7/1/77.
  6. National Board Number for Unit N/A.
  7. Examination Dates: 08-17-92 through 02-02-94.
11. Examination Summary (cont'd):

11.14.2 Results

All four valves had an acceptable leak rates  
below the test acceptance criteria of  $\leq 30,000$   
SCCM. No discrepancies were noted.

11.15 Summary NuReg 0578 - TMI Lessons Learned -

- A. Twenty-seven (27) System Pressure Tests were completed per NUREG 0578 commitments.

Completed System Pressure Tests on the following systems:

Chemical Volume and Control	Safety Injection
Containment Spray	Sampling
Liquid Radwaste	Waste Gas
Residual Heat Removal	

- B. The PSE&G Research Corp., under the direction of the Salem ISI Group conducted the Waste Gas System Leak Test in accordance with procedure SC.SS-IS.WG-0003(R).

11.15.1 Results

- A. The leakage on systems defined by NUREG 0578 were identified as minor mechanical leakage and would not have any adverse impact on the Auxiliary building access during an accident. Work requests were initiated for repairs.

- B. The Waste Gas System Leak Rate Test showed no adverse findings relative to this Bulletin.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS  
AS REQUIRED BY THE PROVISIONS OF THE ASME CODE RULES

1. Owner: Public Service Electric & Gas Co., 80 Park Plaza  
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2. Plant: Salem Generating Station P.O. Box 236,  
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3. Plant Unit 1
4. Owner's Certificate of Authorization (if required) N/A.
5. Commercial Service Date 7/1/77.
6. National Board Number for Unit N/A.
7. Examination Dates: 08-17-92 through 02-02-94.

11. Examination Summary (cont'd):

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date March 30, 1994 Signed PSE&G Owner By Louis H. Lake

Certificate of Authorization No. (if applicable) N/A

Expiration Date N/A

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of New Jersey and employed by Arkwright Mutual Insurance Company, Norwood, Massachusetts, have inspected the components described in this Owner's Data Report during the period 08-17-92 to 02-02-94 and state that to the best of my knowledge and belief the Owner has performed examinations and taken corrective measures described in this Owner's Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-31, 1994

James E. Carl  
Inspector's Signature

Commissions

NJ 373 "I"  
National Board, State,  
Province and No.

PUBLIC SERVICE ELECTRIC & GAS CO.

SALEM NUCLEAR POWER STATION UNIT 1

OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECTION XI COMPLETE COMPONENTS  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)

March 4, 1994  
REVISION 0

Prepared By: John W. J. [Signature] Date 3/4/94  
Reviewed By: N/A Date \_\_\_\_\_  
Approved By: Louis H. Lake Date 3/4/94

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 1

REACTOR PRESSURE VESSEL

SUMMARY	EXAMINATION AREA	ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	O	G	T
		ITEM NO	METHOD	PROCEDURE	R	E	H
					E	O	E
					C	M	R
							REMARKS
							**CALIBRATION BLOCK**

CIRCUMFERENTIAL WELDS (REF. DWG. NO. A-1)

002001	1-RPV-7042	B-A	UT11.5	VS1SSISRCE0094Q	X	-	-	PERFORM MANUAL UT FROM THE SURFACE OF
	VESSEL TO FLANGE	B1.30	UT5.7		X	-	-	THE VESSEL FLANGE ON 100% VESSEL TO
			UT12.5		X	-	-	FLANGE WELD (#1 THRU 54, STUD HOLES).
								W.O.#931121035 TO PERFORM NDE.
								**53-SAM/109-SAM**

THREADS IN FLANGE (REF. DWG. NO. A-2)

003700	1-RPV-LIG 1 THRU 54	B-G-1	UTO	VS1SSISRCE0009Q	X	-	-	EXAMINE LIGAMENTS #18 THRU #54.
		B6.40						W.O.#931121035 TO PERFORM NDE.
								**7-1.25-8-CS-60-SAM**

REACTOR VESSEL SUPPORT (REF. DWG. NO. A-1)

003751	1-RPV-11CLSUPPORT	F-A	VT-3	S1SSISZZ0020Q	X	-	-	THIS EXAM IS TO BE COORDINATED WITH THE
	SLIDING SUPPORT	F2.10						SAFE END WELD EXAMINATIONS ON THE #11
								COLD LEG. W.O. # 931121035, TO PERFORM
								NDE.
003752	1-RPV-12HLSUPPORT	F-A	VT-3	S1SSISZZ0020Q	X	-	-	THIS EXAM IS TO BE COORDINATED WITH THE
	SLIDING SUPPORT	F2.10						SAFE END WELD EXAMINATIONS OF THE #12
								HOT LEG. W.O.# 931121035, TO PERFORM
								NDE.

INTERIOR (REF. DWG. NO. A-1)

004310	RPV INTERNAL SURFACE	B-N-1	VT-3	VS1SSISZZ0020Q	X	-	-	SWRI TO PERFORM VT-3 INSPECTION.
		B13.10						W.O.#931121035 TO PERFORM NDE. FOR SMD
								TO SUPPORT THE RPV VT, SEE W.O.#
								930601189.

CONTROL ROD DRIVE HOUSINGS (REF. DWG. NO. A-2B)

006050	1-CRDH	B-O	PT (#72)	VS1SSISZZ0003Q	X	-	-	EXAMINED #72 & 73 CRDH, NEAR #50 STUD
	CRD HOUSING BODY-TO-ADAPTER	B14.10	PT (#73)		X	-	-	HOLE. W.O. #931121035 TO PERFORM NDE.
	WELDS							

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 2

**PRESSURIZER**

					N	O	
		ASME			O	G	T
		SEC. XI			R	E	H
SUMMARY EXAMINATION AREA		CATGY	EXAM		E	O	E REMARKS
NUMBER IDENTIFICATION		ITEM NO	METHOD	PROCEDURE	C	M	R **CALIBRATION BLOCK**

BOLTING (REF. DWG. NO. A-3)

007820	PZR BOLTING-MANWAY	B-G-2	VT-1 (ISI)	SCSSISZZ0010Q	-	-	X	A FLORESCENT MT EXAM TO BE PERFORMED PER
		B7.21	VT-1 (PSI)		X	-	-	NRC BULLETIN 82-02. MQS TO PERFORM THE
			MT	VSHSSSPZZ0202Q	X	-	-	MT AND PSE&G TO PERFORM THE VT. W.O. #920401058 & 931121035 TO PERFORM NDE. SEVERAL BOLTS WERE REPLACED PER CJP #S-93-254 & W.O. #930401183.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

**PAGE: 3**

### STEAM GENERATOR 11

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM		O	G T
		ITEM NO	METHOD	PROCEDURE		R	E H
						E	O E
						C	M R
							REMARKS
							**CALIBRATION BLOCK**
<u>BOLTING (REF. DWG. NO. A-4)</u>							
008790	11-STG-OMB BOLTS-OUTLET MANWAY	B-G-2 B7.30	VT-1 MT	SCSSISZZ0010Q VSHSSSPZZ0202Q	X	-	- A FLORESCENT MT EXAM TO BE PERFORMED PER 82-02. MQS TO PERFORM THE MT AND PSE&G TO PERFORM THE VT. W.O. # 931105020 AND 931121035 TO PERFORM ND.
008795	11-STG-IMB BOLTS-INLET MANWAY	B-G-2 B7.30	VT-1 MT	SCSSISZZ0010Q VSHSSSPZZ0202Q	X	-	- A FLORESCENT MT EXAM TO BE PERFORMED PER 82-02. MQS TO PERFORM THE MT AND PSE&G TO PERFORM THE VT. W.O. #931105020 AND 931121035 TO PERFORM NDE.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 4

STEAM GENERATOR 12

SUMMARY EXAMINATION AREA	ASME				N	O	
NUMBER IDENTIFICATION	SEC. XI				O	G	T
	CATGY	EXAM			R	E	H
	ITEM NO	METHOD	PROCEDURE		E	O	E
					C	M	R
-----							
<u>BOLTING (REF. DWG. NO. A-5)</u>							
009650	12-STG-OMB	B-G-2	VT-1	SCSSISZZ0010Q	X	-	-
	BOLTS-OUTLET MANWAY	B7.30	MT	VSHSSSPZZ0202Q	X	-	-
					A FLORESCENT MT EXAM TO BE PERFORMED PER		
					82-02. MQS TO PERFORM THE MT AND PSE&G		
					TO PERFORM THE VT. W.O. #931121035 &		
					931105031 TO PERFORM NDE.		
009675	12-STG-IMB	B-G-2	VT-1	SCSSISZZ0010Q	X	-	-
	BOLTS-INLET MANWAY	B7.30	MT	VSHSSSPZZ0202Q	X	-	-
					A FLORESCENT MT EXAM TO BE PERFORMED PER		
					82-02. MQS TO PERFORM THE MT AND PSE&G		
					TO PERFORM THE VT. W.O. #931121035 &		
					931105031 TO PERFORM NDE.		

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 5

STEAM GENERATOR 13

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	O	G	T
		ITEM NO	METHOD	PROCEDURE	R	E	H
					E	O	E
					C	M	R
<u>BOLTING (REF. DWG. NO. A-6)</u>							
010550	13-STG-OMB	B-G-2	VT-1	SCSSISZZ0010Q	X	-	-
	BOLTS-OUTLET MANWAY	B7.30	MT	VSHSSSPZZ0202Q	X	-	-
							A FLORESCENT MT EXAM TO BE PERFORMED PER 82-02. MQS TO PERFORM THE MT AND PSE&G TO PERFORM THE VT. W.O. #931105032 AND 931121035 TO PERFORM NDE.
010575	13-STG-IMB	B-G-2	VT-1	SCSSISZZ0010Q	X	-	-
	BOLTS-INLET MANWAY	B7.30	MT	VSHSSSPZZ0202Q	X	-	-
							A FLORESCENT MT EXAM TO BE PERFORMED PER 82-02. MQS TO PERFORM THE MT AND PSE&G TO PERFORM THE VT. W.O. # 931105032 AND 931121035 TO PERFORM THE NDE.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 6

STEAM GENERATOR 14

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY EXAM ITEM NO METHOD	PROCEDURE	N O G T R E H E O E C M R	REMARKS
-----				
<u>BOLTING (REF. DWG. NO. A-7)</u>				
011450 14-STG-OMB BOLTS-OUTLET MANWAY	B-G-2 VT-1 B7.30 MT	SCSSISZZ0010Q VSHSSSPZZ0202Q	- - X X - -	A FLORESCENT MT EXAM TO BE PERFORMED PER 82-02. MQS TO PERFORM MT & PSE&G TO PERFORM VT. W.O. #931121035 & 931105033 TO PERFORM NDE. #14CL BOLT #16 WAS FOUND TO HAVE WASTAGE ON BOLT SHANK. CJP # S-93-262 REPLACED BOLT W/ SATISFACTORY PSI VT.
011475 14-STG-IMB BOLTS-INLET MANWAY	B-G-2 VT-1 B7.30 MT	SCSSISZZ0010Q VSHSSSPZZ0202Q	X - - X - -	A FLORESCENT MT EXAM TO BE PERFORMED PER 82-02. MQS TO PERFORM THE MT & PSE&G TO PERFORM VT. W.O. #931105033 & 931121035 TO PERFORM NDE.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 7

CHEMICAL AND VOLUME CONTROL SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E H E O E C M R	REMARKS
<u>12-CV-1143 (REF. DWG. NO. A-8)</u>					
011700 12-CV-1143-3 PIPE TO ELBOW	B-J B9.11	PT UT45 UT45T UT45TAN	VS1SSISZZ0003Q VS1SSISZZ0014Q	X - - - X - X - - X - -	W.O.#931121035 TO PERFORM NDE. THE UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT AND COUNTERBORE.  **12-SS-160-1.283-21-SAM**
<u>3-CV-1141 (REF. DWG. NO. A-9)</u>					
016701 3-CV-1141-14 VALVE TO ELBOW	A-E NB88-08	PT UT45 UT45T UT45TAN	VS1SSISZZ0003Q VS1SSISZZ0014Q	X - - - X - X - - X - -	EXAMINED PER NRC BULLETIN 88-08. W.O.#931121035 TO PERFORM NDE.  **3-SS-160-.451-30-SAM**
016801 3-CV-1141-15 ELBOW TO BRANCH CONNECTION	A-E NB88-08	PT UT45 UT45T UT45TAN	VS1SSISZZ0003Q VS1SSISZZ0014Q	X - - - X - X - - X - -	EXAMINED PER NRC BULLETIN 88-08. W.O.#931121035 TO PERFORM NDE. THE UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT. **3-SS-160-.451-30-SAM**
<u>2-CV-1175 (REF. DWG. NO. A-11, A-12)</u>					
022001 2-CV-1175-35 VALVE TO PIPE	A-E NB88-08	PT UT45 UT45T UT70	VS1SSISZZ0003Q VS1SSISZZ0016Q	X - - X - - X - - X - -	EXAMINED PER NRC BULLETIN 88-08. W.O.#931121035 TO PERFORM NDE.  **1.5-SS-COUP-111-SAM**
022101 2-CV-1175-36 PIPE TO TEE	A-E NB88-08	PT UT45 UT45T UT70	VS1SSISZZ0003Q VS1SSISZZ0016Q	X - - X - - X - - X - -	EXAMINED PER NRC BULLETIN 88-08. W.O.#931121035 TO PERFORM NDE.  **1.5-SS-COUP-111-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 8

PRESSURE RELIEF SYSTEM

SUMMARY EXAMINATION AREA	ASME				N	O	
NUMBER IDENTIFICATION	SEC. XI				O	G	T
	CATGY	EXAM			R	E	H
	ITEM NO	METHOD	PROCEDURE		E	O	E
					C	M	R
					-	-	-

4-PR-1100 (REF. DWG. NO. A-16)

026600	4-PR-1100-8	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	PIPE TO TEE	B9.11	UT45	VS1SSISZZ0014Q	X	-	-	
			UT45T		X	-	-	
			UT45TAN		X	-	-	
								**4-SS-160-.533-28-SAM-R**

3-PR-1107 (REF. DWG. NO. A-16)

027400	3-PR-1107-6	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	PIPE TO ELBOW	B9.21						
027600	3-PR-1107-8	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O. 931121035 TO PERFORM NDE.
	PIPE TO ELBOW	B9.21						

3-PR-1106 (REF. DWG. NO. A-17)

028500	3-PR-1106-3	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	ELBOW TO PIPE	B9.21						

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 9

PRESSURIZING SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							**CALIBRATION BLOCK**
<hr/>							
<u>4-PS-1131 (REF. DWG. NO. A-19,20,21)</u>							
031900	4-PS-1131-13	B-J	PT	VS1SSISZZ0003Q	X	-	W.O.#931121035 TO PERFORM NDE.
	ELBOW TO VALVE	B9.11	UT45	VS1SSISZZ0014Q	X	-	
			UT45T		X	-	
			UT45TAN		X	-	**4-SS-160-.533-28-SAM-R**
032300	4-PS-1131-17	B-J	PT	VS1SSISZZ0003Q	X	-	W.O.#931121035 TO PERFORM NDE. THE UT45
	ELBOW TO PIPE	B9.11	UT45	VS1SSISZZ0014Q	-	X	INDICATIONS ARE GEOMETRIC INDICATIONS
			UT45T		X	-	FROM THE INSIDE SURFACE AND WELD ROOT.
			UT45TAN		X	-	**4-SS-160-.533-28-SAM-R**
032700	4-PS-1131-21	B-J	PT	VS1SSISZZ0003Q	X	-	W.O.#931121035 TO PERFORM NDE.
	PIPE TO TEE	B9.11	UT45	VS1SSISZZ0014Q	X	-	
			UT45T		X	-	
			UT45TAN		X	-	**4-SS-160-.533-28-SAM-R**
<u>4-PS-1111 (REF. DWG. NO. A-22)</u>							
035700	4-PS-1111-15	B-J	PT	VS1SSISZZ0003Q	X	-	W.O.#931121035 TO PERFORM NDE.
	ELBOW TO VALVE	B9.11	UT45	VS1SSISZZ0014Q	X	-	
			UT45T		X	-	
			UT45TAN		X	-	**4-SS-160-.533-28-SAM-R**
036300	4-PS-1111-21	B-J	PT	VS1SSISZZ0003Q	X	-	W.O.#931121035 TO PERFORM NDE. THE UT45
	ELBOW TO PIPE	B9.11	UT45	VS1SSISZZ0014Q	-	X	INDICATION IS A GEOMETRIC REFLECTOR FROM
			UT45T		X	-	THE COUNTERBORE.
			UT45TAN		X	-	**4-SS-160-.533-28-SAM-R**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 10

REACTOR COOLANT SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O G T R E H E O E C M R	REMARKS
<u>29-RC-1140 (REF. DWG. NO. A-27)</u>						
047301	29-RC-1140-1 NOZZLE TO SAFE-END	B-F B5.130	PT	VS1SSISZZ0003Q	X - -	W.O.#931121035 TO PERFORM NDE. FOR SMD TO REMOVE THE SANDBOX COVERS, SEE W.O.# 930601187. **37-SAM/76-SAM**
047401	29-RC-1140-2 SAFE-END TO PIPE	B-J B9.11	PT	VS1SSISZZ0003Q	X - -	W.O.#931121035 TO PERFORM NDE. FOR SMD TO REMOVE THE SANDBOX COVERS, SEE W.O.# 930601187. **2.312-SS-37-SAM**
<u>29-RC-1120 (REF. DWG. NO. A-29)</u>						
050901	29-RC-1120-1 NOZZLE TO SAFE-END	B-F B5.130	PT	VS1SSISZZ0003Q	X - -	W.O.#931121035 TO PERFORM NDE. FOR SMD TO REMOVE THE SANDBOX COVERS, SEE W.O.# 930601187. **37-SAM/76-SAM**
051001	29-RC-1120-2 SAFE-END TO PIPE	B-J B9.11	PT	VS1SSISZZ0003Q	X - -	W.O.#931121035 TO PERFORM NDE. FOR SMD TO REMOVE THE SANDBOX COVERS, SEE W.O.# 930601187. **2.312-SS-37-SAM**
<u>27.5-RC-1130 (REF. DWG. NO. A-32)</u>						
057201	27.5-RC-1130-4 ELBOW TO SAFE-END	B-J B9.11	PT	VS1SSISZZ0003Q	X - -	W.O.#931121035 TO PERFORM NDE. FOR SMD TO REMOVE THE SANDBOX COVERS, SEE W.O.# 930601187. **2.312-SS-37-SAM**
057301	27.5-RC-1130-5 SAFE-END TO NOZZLE	B-F B5.130	PT	VS1SSISZZ0003Q	X - -	W.O.#931121035 TO PERFORM NDE. FOR SMD TO REMOVE THE SANDBOX COVERS, SEE W.O.# 930601187. **37-SAM/76-SAM**
<u>27.5-RC-1110 (REF. DWG. NO. A-34)</u>						
059901	27.5-RC-1110-4 ELBOW TO SAFE-END	B-J B9.11	PT	VS1SSISZZ0003Q	X - -	W.O.#931121035 TO PERFORM NDE. FOR SMD TO REMOVE THE SANDBOX COVERS, SEE W.O.# 930601187. **2.312-SS-37-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 11

## REACTOR COOLANT SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	CATGY	EXAM		O	G	T
		ITEM NO	METHOD	PROCEDURE	E	R	H
					E	O	E
					C	M	R
					-	-	-
<u>27.5-RC-1110 (REF. DWG. NO. A-34)</u>							
060001	27.5-RC-1110-5 SAFE-END TO NOZZLE	B-F B5.130	PT	VS1SSISZ20003Q	X	-	-
					REMARKS		
					**CALIBRATION BLOCK**		
					W.O.#931121035 TO PERFORM NDE. FOR SMD		
					TO REMOVE THE SANDBOX COVERS, SEE W.O.#		
					930601187.		
					**37-SAM/76-SAM**		

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 12

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O		
NUMBER	IDENTIFICATION	SEC. XI	EXAM	METHOD	PROCEDURE	O	G	
		CATGY				R	E	
		ITEM NO				E	O	
						C	M	
								REMARKS
								**CALIBRATION BLOCK**
-----								
<u>12-SJ-1151 (REF. DWG. NO. A-53)</u>								
094700	12-SJ-1151-3 ELBOW TO PIPE	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O. #931121035 TO PERFORM NDE. THIS WELD REPLACES SUMMARY #094600, DUE TO A HANGER OBSTRUCTION. THE UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT. **12-SS-160-1.283-21-SAM**
		B9.11	UT45	VS1SSISZZ0014Q	-	X	-	
			UT45T		X	-	-	
			UT45TAN		X	-	-	
			UT0	VS1SSISZZ0018Q	X	-	-	
<u>10-SJ-1141 (REF. DWG. NO. A-54)</u>								
096700	10-SJ-1141-14 VALVE TO TEE	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. LIMITATION: THE EXAM IS LIMITED TO 0% COVERAGE, DUE TO VALVE \ TEE CONFIGURATION. **10-SS-160-1.119-22-SAM**
		B9.11	UT45	VS1SSISZZ0014Q	-	-	X	
			UT45T		-	-	X	
<u>8-SJ-1162 (REF. DWG. NO. A-58, A-59)</u>								
105000	8-SJ-1162-5 PIPE TO PIPE	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. THE UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT. **8-SS-XX-.860-23-SAM**
		B9.11	UT45	VS1SSISZZ0014Q	-	X	-	
			UT45T		X	-	-	
			UT45TAN		X	-	-	
<u>8-SJ-1152 (REF. DWG. NO. A-60, A-61)</u>								
109600	8-SJ-1152-4PS-2 PENETRATION TO PIRE	B-K-1 B10.10	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
109900	8-SJ-1152-6 PIPE TO ELBOW	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. THE UT45 INDICATIONS ARE GEOMETRIC INDICATIONS FROM THE WELD ROOT AND COUNTERBORE. **8-SS-XX-.860-23-SAM**
		B9.11	UT45	VS1SSISZZ0014Q	-	X	-	
			UT45T		X	-	-	
			UT45TAN		X	-	-	
<u>4-SJ-1182 (REF. DWG. NO. A-74)</u>								
129000	4-SJ-1182-14 PIPE TO ELBOW	B-J	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. THE UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT. **4-SS-160-.533-28-SAM-R**
		B9.11	UT45	VS1SSISZZ0014Q	-	X	-	
			UT45T		X	-	-	
			UT45TAN		X	-	-	

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 13

SAFETY INJECTION SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
								<b>**CALIBRATION BLOCK**</b>
<u>4-SJ-1172 (REF. DWG. NO. A-75)</u>								
131200	4-SJ-1172-5 ELBOW TO PIPE	B-J B9.11	PT PT UT45 UT45T UT45TAN	VS1SSISZZ0003Q  VS1SSISZZ0014Q	- - X X - - - X - X - - X - -			W.O.#931121035 (NDE). THE PT EXAM REVEALED 6 LINEAR RECORDABLE INDICATIONS. DURING THE RELEVANCY DETERMINATION PROCESS, SURFACE CONDITIONING PROCEEDED A RE-EXAM WITH NO LINEAR INDICATIONS IDENTIFIED. UT45 GEOMETRIC, DUE TO WELD COUNTERBORE. <b>**4-SS-160-.533-28-SAM-R**</b>
133800	4-SJ-1172-28 TEE TO REDUCER	B-J B9.11	PT UTOL UT45 UT45T UT45TAN	VS1SSISZZ0003Q VS1SSISZZ0018Q VS1SSISZZ0014Q	X - - X - - - X - X - - X - -			W.O.#931121035 TO PERFORM NDE. THE UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT. <b>**4-SS-160-.533-28-SAM-R**</b>
<u>3-SJ-1192 (REF. DWG. NO. A-72)</u>								
134700	3-SJ-1192-8A PIPE TO PIPE	B-J B9.21	PT	VS1SSISZZ0003Q	X - -			W.O.#931121035 TO PERFORM NDE.
<u>2-SJ-1147 (REF. DWG. NO. A-78)</u>								
141500	2-SJ-1147-8FB FLANGE BOLTING	B-G-2 B7.50	VT-1	SCSSISZZ0010Q	X - -			W.O.# 920810139 & 931121035.
<u>2-SJ-1137 (REF. DWG. NO. A-80)</u>								
147900	2-SJ-1137-10FB FLANGE BOLTING	B-G-2 B7.50	VT-1	SCSSISZZ0010Q	X - -			W.O.# 920810145 & 931121035.
<u>2-SJ-1128 (REF. DWG. NO. A-83,84,85)</u>								
155500	2-SJ-1128-2FB FLANGE BOLTING	B-G-2 B7.50	VT-1	SCSSISZZ0010Q	X - -			W.O.# 920810149 & 931121035.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 14

SAFETY INJECTION SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
-----								
<u>2-SJ-1128 (REF. DWG. NO. A-83,84,85)</u>								
158200	2-SJ-1128-29 ELBOW TO PIPE	B-J B9.40	PT	VS1SSISZZ0003Q	X	-	-	W.O. #931121035 TO PERFORM NDE. THIS WELD REPLACES SUMMARY #157900, DUE TO HANGER INTERFERENCE.
158600	2-SJ-1128-32 PIPE TO COUPLING	B-J B9.40	PT	VS1SSISZZ0003Q	X	-	-	W.O. #931121035 TO PERFORM NDE. THIS EXAM REPLACES SUMMARY #159100, DUE TO ALARA CONCERNS. SUMMARY #159100 WAS LOCATED IN THE DECAY TUNNEL.
<u>2-SJ-1118 (REF. DWG. NO. A-87, A-88)</u>								
165800	2-SJ-1118-4FB FLANGE BOLTING	B-G-2 B7.50	VT-1	SCSSISZZ0010Q	X	-	-	W.O.# 920810151 & 931121035.
<u>1.5-SJ-1142 (REF. DWG. NO. A-89,90,91)</u>								
174300	1 1/2-SJ-1142-43 ELBOW TO PIPE	B-J B9.40	PT	VS1SSISZZ0003Q	X	-	-	W.O. #931121035 TO PERFORM NDE. THIS WELD REPLACED SUMMARY #174100, DUE TO A HANGER OBSTRUCTION.
174900	1 1/2-SJ-1142-49R PIPE TO VALVE	B-J B9.40	PT	VHSSSPZZ0206Q	X	-	-	PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-184 & W.O# 920508129.
175001	1 1/2-SJ-1142-50R VALVE TO PIPE	A-E NB88-08	PT UT	VHSSSPZZ0206Q	X	-	-	88-08 EXAMS WAIVED, REPLACEMENT. W.O.#931121035 TO PERFORM NDE. PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-184 & W.O# 920508129. **1.5-SS-COUP-111-SAM**
175101	1 1/2-SJ-1142-51R PIPE TO ELBOW	A-E NB88-08	PT UT	VHSSSPZZ0206Q	X	-	-	88-08 EXAMS WAIVED, REPLACEMENT. W.O.#931121035 TO PERFORM NDE. PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-184 & W.O# 920508129. **1.5-SS-COUP-111-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 15

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME	EXAM		N O		REMARKS
NUMBER	IDENTIFICATION	SEC. XI	ITEM NO	METHOD	PROCEDURE	O G T	
		CATGY				R E H	
		EXAM				E O E	
		UT				C M R	
*****							**CALIBRATION BLOCK**
<u>1.5-SJ-1142 (REF. DWG. NO. A-89,90,91)</u>							
175201	1 1/2-SJ-1142-52 ELBOW TO PIPE	A-E NB88-08	PT UT45 UT45T UT70	VHSSSPZZ0206Q VS1SSISZZ0016Q	X - - X - - X - - X - -	EXAMINED PER NRC BULLETIN 88-08. W.O. #931121035 TO PERFORM NDE.  **1.5-SS-COUP-111-SAM**	
175301	1 1/2-SJ-1142-53 PIPE TO BRANCH CONNECTION	A-E NB88-08	PT UT45 UT45T UT70	VS1SSISZZ0003Q VS1SSISZZ0016Q	X - - X - - X - - X - -	EXAMINED PER NRC BULLETIN 88-08. W.O.#931121035 TO PERFORM NDE.  **1.5-SS-COUP-111-SAM**	
<u>1.5-SJ-1132 (REF. DWG. NO. A-92, A-93)</u>							
179500	1 1/2-SJ-1132-39R PIPE TO VALVE	B-J B9.40	PT	VHSSSPZZ0206Q	X - -	PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-183 & W.O# 921117149.	
179601	1 1/2-SJ-1132-40R VALVE TO PIPE	A-E NB88-08	PT UT	VHSSSPZZ0206Q	X - -	88-08 EXAMS WAIVED, REPLACEMENT. W.O.#931121035 TO PERFORM NDE. PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-183 & W.O# 921117149. **1.5-SS-COUP-111-SAM**	
179701	1 1/2-SJ-1132-41R PIPE TO ELBOW	A-E NB88-08	PT UT	VHSSSPZZ0206Q	X - -	88-08 EXAMS WAIVED, REPLACEMENT. W.O.#931121035 TO PERFORM NDE. PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-183 & W.O# 921117149. **1.5-SS-COUP-111-SAM**	
179801	1 1/2-SJ-1132-42 ELBOW TO PIPE	A-E NB88-08	PT UT45 UT45T UT70	VHSSSPZZ0206Q VS1SSISZZ0016Q	X - - X - - X - - X - -	EXAMINED PER NRC BULLETIN 88-08. W.O. #931121035 TO PERFORM NDE.  **1.5-SS-COUP-111-SAM**	



DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 17

SAFETY INJECTION SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
					-	-	-	**CALIBRATION BLOCK**
<u>1.5-SJ-1112 (REF. DWG. NO. A-96, A-97)</u>								
190100	1 1/2-SJ-1112-49R PIPE TO VALVE	B-J B9.40	PT	VHSSSPZZ0206Q	X	-	-	PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-182 & W.O# 921117147.
190201	1 1/2-SJ-1112-50R VALVE TO PIPE	A-E NB88-08	PT UT	VHSSSPZZ0206Q	X	-	-	88-08 EXAMS WAIVED, REPLACEMENT. W.O.#931121035 TO PERFORM NDE. PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-182 & W.O# 921117147. **1.5-SS-COUP-111-SAM**
190301	1 1/2-SJ-1112-51R PIPE TO ELBOW	A-E NB88-08	PT UT	VHSSSPZZ0206Q	X	-	-	88-08 EXAMS WAIVED, REPLACEMENT. W.O.#931121035 TO PERFORM NDE. PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD PER DCP 1E0-2309, CJP# S-93-182 & W.O# 921117147. **1.5-SS-COUP-111-SAM**
190401	1 1/2-SJ-1112-52 ELBOW TO PIPE	A-E NB88-08	PT UT45 UT45T UT70	VHSSSPZZ0206Q VS1SSISZZ0016Q	X X X X	- - - -	-	EXAMINED PER NRC BULLETIN 88-08. W.O. #931121035 TO PERFORM NDE.  **1.5-SS-COUP-111-SAM**
190501	1 1/2-SJ-1112-53 PIPE TO BRANCH CONNECTION	A-E NB88-08	PT UT45 UT45T UT70	VS1SSISZZ0003Q VS1SSISZZ0016Q	X X X X	- - - -	-	EXAMINED PER NRC BULLETIN 88-08. W.O.#931121035 TO PERFORM NDE.  **1.5-SS-COUP-111-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 18

REACTOR COOLANT PUMP 12

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	O	G	T
		ITEM NO	METHOD	PROCEDURE	R	E	H
					E	O	E
					C	M	R
					-	-	-

REMARKS  
\*\*CALIBRATION BLOCK\*\*

INTEGRALLY WELDED SUPPORTS (REF. DWG. NO. A-98)

191800	12-PMP-1LG LUG	B-K-1 B10.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. LIMITATION: THE EXAM IS LIMITED TO 72% COVERAGE, DUE TO THE PUMP BRACING AT THE BOTTOM OF THE LUG.
191850	12-PMP-2LG LUG	B-K-1 B10.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. LIMITATION: THE EXAM IS LIMITED TO 72% COVERAGE, DUE TO THE PUMP BRACING AT THE BOTTOM OF THE LUG.
191900	12-PMP-3LG LUG	B-K-1 B10.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. LIMITATION: THE EXAM IS LIMITED TO 76% COVERAGE, DUE TO THE PUMP BRACING AT THE BOTTOM OF THE LUG.

PUMP MOTOR FLYWHEEL (REF. DWG. NO. A-98)

192101	12-PMP-FLW PUMP MOTOR FLYWHEEL (IN-PLACE)	A-E RG1.14	UTOL UT45	VS1SSISZZ0001Q	X	-	-	PERFORM IN-PLACE UT EXAMINATION PER PER NRC REGULATORY GUIDE 1.14 & W.O. #931121035 ON RCP MOTOR FLYWHEEL S/N 3S-76P917 "TOP SIDE ONLY". LIMITED UT, DUE TO THE BORE AND BOLT HOLES. **42-SAM**
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DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 19

REACTOR COOLANT PUMP 14

SUMMARY	EXAMINATION AREA	ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	ITEM NO	METHOD	PROCEDURE	REMARKS
								**CALIBRATION BLOCK**

INTEGRALLY WELDED SUPPORTS (REF. DWG. NO. A-98)

193400	14-PMP-1LG LUG	B-K-1 B10.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. LIMITATION: THE EXAM IS LIMITED TO 77% COVERAGE, DUE TO THE PUMP BRACING AT THE BOTTOM OF THE LUG.
193450	14-PMP-2LG LUG	B-K-1 B10.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. LIMITATION: THE EXAM IS LIMITED TO 74% COVERAGE, DUE TO THE PUMP BRACING AT THE BOTTOM OF THE LUG.
193500	14-PMP-3LG LUG	B-K-1 B10.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. LIMITATION: THE EXAM IS LIMITED TO 76%, DUE TO THE PUMP BRACING AT THE BOTTOM OF THE LUG.

PUMP MOTOR FLYWHEEL (REF. DWG. NO. A-98)

193701	14-PMP-FLW PUMP MOTOR FLYWHEEL (IN-PLACE)	A-E RG1.14	UTOL UT45	VS1SSISZZ0001Q	X	-	-	PERFORM IN-PLACE UT EXAMINATION PER NRC REGULATORY GUIDE 1.14 & W.O. #931121035 ON THE RCP MOTOR FLYWHEEL S/N 2S-76P917 "TOP SIDE ONLY". LIMITED UT, DUE TO BORE AND BOLT HOLES. **42-SAM**
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DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 20

VALVES

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							**CALIBRATION BLOCK**
<hr/>							
<u>VALVE BODIES</u>							
193870	12 SJ 55 VB ON LINE 10-SJ-1121	B-M-2 B12.50	VT-3	S1SSISZZ0010Q	X	-	- W.O. 931121035 TO PERFORM NDE.
<u>BOLTING (REF. DWG. NO. A-10)</u>							
194100	1 CV 78 ON LINE 3-CV-1133	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	-	-	X W.O.# 920810209 AND 931121035 TO PERFORM NDE. ONE (1) STUD WAS FOUND WITH INADEQUATE THREAD ENGAGEMENT. A DR WAS ISSUED TO SYSTEM ENGINEERING. W.O. #93100204 CORRECTED THE THREAD ENGAGEMENT PROBLEM. A SATISFACTORY RE-EXAM WAS PERFORMED.
<u>BOLTING</u>							
194700	1 PR 1 ON LINE 3-PR-1106	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	-	- PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO DCP 1EC-3219-1, CJP# PS-93-053 & W.O.# 930126150. NEW STUDS AND NUTS WERE INSTALLED.
194800	1 PR 2 ON LINE 3-PR-1107	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	-	- PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO DCP 1EC-3219-1, CJP# PS-93-054 & W.O.# 930126150. NEW STUDS AND NUTS WERE INSTALLED.
195400	1 PS 1 ON LINE 4-PS-1111	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	-	- THE BOLTING WAS REPLACED PER 1EC-3249-1/PS-93-056/930708108 & 931121035.
195500	1 PS 3 ON LINE 4-PS-1131	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	-	- THE BOLTING WAS REPLACED PER 1EC-3249-1/PS-93-057/930708111 & 931121035.
197000	11 SJ 43 ON LINE 6-SJ-1112	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	-	- W.O.# 920810155 AND 931121035 TO PERFORM NDE.

## VALVES

SUMMARY EXAMINATION AREA		ASME SEC. XI	EXAM			N O G T R E H E O E C M R	REMARKS
NUMBER	IDENTIFICATION	ITEM NO	METHOD	PROCEDURE			**CALIBRATION BLOCK**
<u>BOLTING (REF. DWG. NO. A-68)</u>							
197900	12 SJ 43 ON LINE 6-SJ-1122	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	- -	W.O.# 920810158 AND 931121035 TO PERFORM NDE.
<u>BOLTING (REF. DWG. NO. A-56)</u>							
198200	12 SJ 55 ON LINE 10-SJ-1121	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	-	- X	W.O.# 920810195 & 931121035 TO PERFORM NDE. THE VT-1 EXAM REVEALED BORON RESIDUE BETWEEN THE VALVE BODY TO BONNET. THE VALVE WAS DISASSEMBLED & INSPECTED & BOLTING MATERIAL WAS REPLACED UNDER W.O. # 931002074 & CJP #S-93-250.
198300	12 SJ 56 ON LINE 10-SJ-1121	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	- -	W.O.# 920810203
<u>BOLTING (REF. DWG. NO. A-66)</u>							
198800	13 SJ 43 ON LINE 6-SJ-1132	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	-	- X	W.O.# 920810161 & 931121035 TO PERFORM NDE. BORON RESIDUE WAS FOUND ON SEVERAL STUDS. W.O. #931026136 REMOVED THE BORON AND A SATISFACTORY VT RE-EXAM WAS PERFORMED.
<u>BOLTING (REF. DWG. NO. A-67)</u>							
199200	13 SJ 156 ON LINE 6-SJ-1131	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	- -	W.O.# 920810173
<u>BOLTING</u>							
199500	14 RH 27 ON LINE 8-SJ-1145	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	- -	THE BOLTING MATERIAL WAS REPLACED IN ACCORDANCE WITH CJP #S-93-249 & W.O. #940326008.
199600	14 SJ 43 ON LINE 6-SJ-1142	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	- -	W.O.# 920810162 AND 931121035 TO PERFORM NDE.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 22

VALVES

SUMMARY EXAMINATION AREA		ASME			N	O	REMARKS	
NUMBER	IDENTIFICATION	SEC. XI	EXAM	PROCEDURE	O	G		
		CATGY			R	E		
		ITEM NO			E	O		
		METHOD			C	M		
					-	-	**CALIBRATION BLOCK**	
<hr/>								
<u>BOLTING (REF. DWG. NO. A-54)</u>								
199700	14 SJ 54 ON LINE 10-SJ-1141	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	-	-	X W.O.# 920810207 & 931121035 TO PERFORM NDE. THE VT REVEALED BORON RESIDUE ON THE STUDS. W.O. #931013197 WAS INITIATED TO CLEAN OFF THE BORON. AFTER THE BORON WAS REMOVED A SATISFACTORY VT-1 EXAM WAS PERFORMED.	
199800	14 SJ 55 ON LINE 10-SJ-1141	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	X	-	- W.O.# 920810201	
199900	14 SJ 56 ON LINE 10-SJ-1141	B-G-2 B7.70	VT-1	SCSSISZZ0010Q	-	-	X W.O.# 920810206 & 931121035 TO PERFORM NDE. THE VISUAL EXAMINATION REVEALED SEVERAL STUDS WITH A LACK OF THREAD ENGAGEMENT. W.O. #930730004 DISSASSEMBLED & INSPECTED THIS VALVE. DURING RE-ASSEMBLY THE LACK OF THREAD ENGAGEMENT WAS CORRECTED.	

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 23

HEAD TANKS, VESSEL 11

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	O	G	T
		ITEM NO	METHOD	PROCEDURE	R	E	H
					E	O	E
					C	M	R
							REMARKS
							**CALIBRATION BLOCK**

CIRCUMFERENTIAL WELDS (REF. DWG. NO. B-4)

204900	11-HT-1	C-A	UT0L	VS1SSISZZ0018Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	UPPER HEAD TO SHELL	C1.20	UT0W		-	-	X	LIMITATION: EXAM LIMITED TO 38%, DUE TO
			UT45	VS1SSISZZ0012Q	X	-	-	WELD CONFIGURATION. UT ON THE WELD NOT
			UT45T		X	-	-	PERFORMED, DUE TO THE WELD
			UT45TAN		X	-	-	CONFIGURATION.
								**6-SS-10-.140-45-SAM**

DATE: 03/04/94

REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
 OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
 SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
 CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 24

REACTOR COOLANT FILTER

SUMMARY EXAMINATION AREA		ASME			N	O		
		SEC. XI			O	G	T	
		CATGY	EXAM		R	E	H	
		ITEM NO	METHOD	PROCEDURE	E	O	E	REMARKS
NUMBER	IDENTIFICATION				C	M	R	**CALIBRATION BLOCK**
<hr/>								
<u>CIRCUMFERENTIAL WELDS (REF. DWG. NO. B-6)</u>								
205020	1-RCF-1 UPPER HEAD TO FLANGE	C-A C1.20	UTOL	VS1SSISZZ0018Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
			UTOW		X	-	-	LIMITATION: EXAM LIMITED TO 74%, DUE TO
			UT45	VS1SSISZZ0012Q	X	-	-	CONFIGURATION.
			UT45T		X	-	-	
			UT45TAN		X	-	-	**14-SS-10-.250-96-SAM**
205030	1-RCF-2 FLANGE TO SHELL	C-A C1.10	UTOL	VS1SSISZZ0018Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
			UTOW		X	-	-	LIMITATION: EXAM LIMITED TO 86%, DUE TO
			UT45	VS1SSISZZ0012Q	X	-	-	CONFIGURATION AND WELDED SUPPORT.
			UT45T		X	-	-	
			UT45TAN		X	-	-	**14-SS-10-.250-96-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 25

REGENERATIVE HEAT EXCHANGER

SUMMARY EXAMINATION AREA		ASME			N	O	
		SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
NUMBER	IDENTIFICATION				C	M	R
							REMARKS
							**CALIBRATION BLOCK**
<u>CIRCUMFERENTIAL WELDS (REF. DWG. NO. B-7)</u>							
205105	1-RHE-5 CAP TO SHELL	C-A C1.10	UTOL	VS1SSISZZ0018Q	X	-	W.O.#931121035 TO PERFORM NDE. THE UT45
			UTOW		X	-	INDICATION IS A GEOMETRIC REFLECTOR FROM
			UT45	VS1SSISZZ0013Q	-	X	THE INSIDE SURFACE.
			UT45T		X	-	
			UT60		X	-	**9.5-SS-X-.750-86-SAM**
			UT60T		X	-	
205120	1-RHE-8 SHELL TO CAP	C-A C1.10	UTOL	VS1SSISZZ0018Q	X	-	W.O.#931121035 TO PERFORM NDE. THE UT45
			UTOW		X	-	AND UT60 INDICATIONS ARE GEOMETRIC
			UT45	VS1SSISZZ0013Q	-	X	REFLECTORS FROM THE WELD ROOT.
			UT45T		X	-	
			UT60		-	X	**9.5-SS-X-.750-86-SAM**
			UT60T		X	-	

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 26

RESIDUAL HEAT REMOVAL HEAT EXCHANGER 11

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM		O	G
		ITEM NO	METHOD	PROCEDURE		R	E
						H	
						E	O
						E	REMARKS
						C	M
						R	**CALIBRATION BLOCK**
						-	-

INTEGRALLY WELDED SUPPORTS (REF. DWG. NO. B-8)

205170	11-RHRHEX-1VS VESSEL SUPPORT	C-C C3.10	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
205180	11-RHRHEX-2VS VESSEL SUPPORT	C-C C3.10	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 27

FEEDWATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							**CALIBRATION BLOCK**
-----							
<u>16-BF-2141 (REF. DWG. NO. B-10)</u>							
210601	16-BF-2141-1R	A-E	RT	VSHSSPZZ0210Q	X	-	PERFORM NDE UNDER W.O. # 9304216193. 45
	REDUCER TO NOZZLE	ATS	UT45	VS1SSISZZ0017Q	-	X	AND 60 DEGREE INDICATIONS ARE GEOMETRIC
			UT45T		X	-	REFLECTORS FROM THE INSIDE SURFACE.
			UT60		-	X	
			UT60T		X	-	**116-SAM**
-----							
<u>16-BF-2131 (REF. DWG. NO. B-11)</u>							
213401	16-BF-2131-1R	A-E	RT	VSHSSPZZ0210Q	X	-	PERFORM NDE UNDER W.O. 930426192. THE
	ELBOW TO NOZZLE	ATS	UT45	VS1SSISZZ0017Q	X	-	60 DEGREE INDICATION IS A GEOMETRIC
			UT45T		X	-	REFLECTOR FROM THE WELD ROOT.
			UT60		-	X	
			UT60T		X	-	**35-SAM/36-SAM**
-----							
<u>16-BF-2121 (REF. DWG. NO. B-12)</u>							
217791	16-BF-2121-1R	A-E	RT	VSHSSPZZ0210Q	X	-	PERFORM NDE UNDER W.O. # 930426190. THE
	REDUCER TO NOZZLE	ATS	UT45	VS1SSISZZ0017Q	-	X	45 AND 60 DEGREE INDICATIONS ARE
			UT45T		X	-	GEOMETRIC REFLECTORS FROM THE WELD ROOT.
			UT60		-	X	
			UT60T		X	-	**116-SAM**
-----							
<u>16-BF-2111 (REF. DWG. NO. B-13)</u>							
220573	16-BF-2111-1R	A-E	RT	VSHSSPZZ0210Q	X	-	PERFORM NDE UNDER W.O. 930426188. THE
	REDUCER TO NOZZLE	ATS	UT45	VS1SSISZZ0017Q	-	X	45 AND 60 DEGREE INDICATIONS ARE
			UT45T		X	-	GEOMETRIC REFLECTORS FROM THE WELD ROOT.
			UT60		-	X	
			UT60T		X	-	**116-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 28

MAIN STEAM SYSTEM

SUMMARY EXAMINATION AREA	ASME				N	O	
NUMBER IDENTIFICATION	SEC. XI				O	G	T
	CATGY	EXAM			R	E	H
	ITEM NO	METHOD	PROCEDURE		E	O	E
					C	M	R
					-	-	-
<u>32-MS-2141 (REF. DWG. NO. B-25)</u>							
227600	32-MS-2141-1	C-F-2	MT	VS1SSISZZ0002Q	X	-	-
	NOZZLE TO REDUCER	C5.51	UT45	VS1SSISZZ0017Q	X	-	-
			UT45T		X	-	-
							W.O.#931121035 TO PERFORM NDE.
							**32-CS-XX-1.618-47-SAM**
<u>32-MS-2121 (REF. DWG. NO. B-27)</u>							
229900	32-MS-2121-2PS-1	C-C	MT	VS1SSISZZ0002Q	X	-	-
	PIPE SUPPORT	C3.20					W.O.#931121035 TO PERFORM NDE.
<u>30-MS-2141 (REF. DWG. NO. B-25)</u>							
232000	30-MS-2141-1	C-F-2	MT	VS1SSISZZ0002Q	X	-	-
	REDUCER TO ELBOW	C5.51	UT45	VS1SSISZZ0017Q	-	X	-
			UT45T		X	-	-
							W.O.#931121035 TO PERFORM NDE. THE UT45
							INDICATIONS ARE GEOMETRIC REFLECTORS
							FROM THE WELD ROOT.
							**PL-1.5-CS-65-SAM**
232040	30-MS-2141-1LD-I	C-F-2	MT	VS1SSISZZ0002Q	X	-	-
	LONGITUDINAL	C5.52	UT45	VS1SSISZZ0017Q	X	-	-
			UT45T		X	-	-
							W.O.#931121035 TO PERFORM NDE. EXAMINE
							2.5T OF WELD LENGTH.
							**PL-1.5-CS-65-SAM**
232080	30-MS-2141-1LD-O	C-F-2	MT	VS1SSISZZ0002Q	X	-	-
	LONGITUDINAL	C5.52	UT45	VS1SSISZZ0017Q	X	-	-
			UT45T		X	-	-
							W.O.#931121035 TO PERFORM NDE. EXAMINE
							2.5T OF WELD LENGTH.
							**PL-1.5-CS-65-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 29

PRESSURE RELIEF SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E C M R	O G T E O E	REMARKS
-----						
<u>12-PR-2101 (REF. DWG. NO. B-37, B-38)</u>						
254900 12-PR-2101-2 PIPE TO PIPE	C-F-1 C5.11	PT UT45 UT45T UT45TAN	VS1SSISZZ0003Q VS1SSISZZ0014Q	X - - X - - X - - X - -		W.O. #931121035 TO PERFORM NDE. THIS WELD REPLACED SUMMARY #254400, DUE TO HANGER INTERFERENCE. EXAMINED 2.5T OF THE INTERSECTING LONGITUDINAL SEAM ON THE UP STREAM AND DOWN STREAM SIDE. **31-SAM/66-SAM**

PAGE: 30

## RESIDUAL HEAT REMOVAL SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O G T R E H E O E C M R			REMARKS
<b>14-RH-2114 (REF. DWG. NO. B-45)</b>								
263290	14-RH-2114-16 FLANGE TO PIPE	C-F-1 C5.11	PT UT45 UT45T UT45TAN	VS1SSISZZ0003Q VS1SSISZZ0014Q	X X X X	- - - -	- - - -	W.O.#931121035 TO PERFORM NDE. EXAMINED 2.5T OF THE ASSOCIATED LONG SEAM WELD.  **14-SS-40-.438-79-SAM**
263292	14-RH-2114-16PS-1 PIPE SUPPORT	C-C C3.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
263294	14-RH-2114-16PS-2 PIPE SUPPORT	C-C C3.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
263296	14-RH-2114-16PS-3 PIPE SUPPORT	C-C C3.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
263298	14-RH-2114-16PS-4 PIPE SUPPORT	C-C C3.20	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
263304	14-RH-2114-18 FLANGE TO PUMP	C-F-1 C5.11	PT UT45T	VS1SSISZZ0003Q VS1SSISZZ0014Q	X X	- -	- -	W.O.#931121035 TO PERFORM NDE. LIMITATION: THE UT EXAM WAS LIMITED TO 50% COVERAGE. THE TRANVERSE SCAN WAS THE ONLY UT PERFORMED, DUE TO THE FLANGE AND PUMP CONFIGURATION. **14-SS-40-.438-79-SAM**
<b>14-RH-2112 (REF. DWG. NO. B-46)</b>								
263384	14-RH-2112-29 TEE TO PIPE	C-F-1 C5.11	PT UT45 UT45T UT45TAN	VS1SSISZZ0003Q VS1SSISZZ0014Q	X X X X	- - - -	- - - -	W.O.#931121035 TO PERFORM NDE. EXAMINATION INCLUDES 2.5T OF THE INTERSECTING LONGITUDINAL SEAM ON THE DOWN STREAM SIDE. **14-SS-40-.438-79-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 31

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O		
NUMBER	IDENTIFICATION	SEC. XI	EXAM	METHOD	PROCEDURE	O	G	T
		CATGY				R	E	H
		ITEM NO				E	O	E
						C	M	R
								REMARKS
								**CALIBRATION BLOCK**
-----								
<u>14-SJ-2124 (REF. DWG. NO. B-52)</u>								
263716	14-SJ-2124-1	C-F-1	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	VALVE TO ELBOW	C5.11	UT45	VS1SSISZZ0014Q	X	-	-	LIMITATION: EXAM LIMITED TO 95%, DUE TO
			UT45T		X	-	-	CONFIGURATION OF VALVE.
			UT45TAN		X	-	-	
								**14-SS-40-.438-79-SAM**
263718	14-SJ-2124-1LD-I	C-F-1	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. EXAMINE
	LONGITUDINAL	C5.12	UT45	VS1SSISZZ0014Q	X	-	-	2.5T OF WELD LENGTH.
			UT45T		X	-	-	
			UT45TAN		X	-	-	
								**14-SS-40-.438-79-SAM**
263720	14-SJ-2124-1LD-O	C-F-1	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. EXAMINE
	LONGITUDINAL	C5.12	UT45	VS1SSISZZ0014Q	X	-	-	2.5T OF WELD LENGTH.
			UT45T		X	-	-	
			UT45TAN		X	-	-	
								**14-SS-40-.438-79-SAM**
<u>12-SJ-2152 (REF. DWG. NO. B-54)</u>								
263830	12-SJ-2152-5	C-F-1	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	ELBOW TO PIPE	C5.11	UT45	VS1SSISZZ0014Q	X	-	-	EXAMINATION INCLUDED 2.5T OF THE
			UT45T		X	-	-	INTERSECTING LONGITUDINAL SEAM DOWN
			UT45TAN		X	-	-	STREAM.
								**12-SS-40-.377-31-SAM**
263896	12-SJ-2152-36	C-F-1	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE. EXAM
	PIPE TO PIPE	C5.11	UT45	VS1SSISZZ0014Q	-	X	-	2.5T OF LONG SEAM ON BOTH SIDES. THE
			UT45T		X	-	-	UT45 INDICATIONS ARE GEOMETRIC
			UT45TAN		X	-	-	REFLECTORS FROM THE INSIDE SURFACE.
								**12-SS-40-.377-31-SAM**
263898	12-SJ-2152-36PS-1	C-C	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	PIPE SUPPORT	C3.20						
263900	12-SJ-2152-36PS-2	C-C	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	PIPE SUPPORT	C3.20						

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 32

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	EXAM		O	G	T
		CATGY			R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
							REMARKS
							**CALIBRATION BLOCK**

12-SJ-2152 (REF. DWG. NO. B-54)

263904	12-SJ-2152-36PS-4	C-C	PT	VS1SSISZZ0003q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	PENETRATION TO PIPE	C3.20						

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 33

CONTAINMENT SPRAY

SUMMARY EXAMINATION AREA		ASME			N	O	REMARKS
NUMBER	IDENTIFICATION	SEC. XI			O	G	
		CATGY	EXAM		R	E	
		ITEM NO	METHOD	PROCEDURE	E	O	
					C	M	
						R	
*****							
8-CS-2114 (REF. DWG. NO. B-66)							
355480	8-CS-2114-4 PIPE TO ELBOW	C-F-1	PT	VS1SSISZZ0003Q	X	-	W.O.# 931121035 TO PERFORM NDE. THIS EXAM REPLACES SUMMARY #355400, DUE TO A SUPPORT INTERFERENCE.  **8-SS-10-.140-24-SAM** °
		C5.11	UT45	VS1SSISZZ0014Q	X	-	
			UT45T		X	-	
			UTO	VS1SSISZZ0018Q	X	-	
355500	8-CS-2114-5 ELBOW TO PIPE	C-F-1	PT	VS1SSISZZ0003Q	X	-	W.O. #931121035 TO PERFORM NDE. THIS EXAM REPLACES SUMMARY #355420, DUE TO A SUPPORT INTERFERENCE.  **8-SS-10-.140-24-SAM**
		C5.11	UT45	VS1SSISZZ0014Q	X	-	
			UT45T		X	-	
			UT45TAN		X	-	
			UTO	VS1SSISZZ0018Q	X	-	

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 34

CHEMICAL & VOLUME CONTROL

SUMMARY EXAMINATION AREA		ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI				O	G	T
		CATGY	EXAM			R	E	H
		ITEM NO	METHOD	PROCEDURE		E	O	E
								REMARKS
								**CALIBRATION BLOCK**
-----								
<u>3-CV-2163 (REF. DWG. NO. B-70)</u>								
358300	3-CV-2163-6 PIPE TO ELBOW	C-F-1 C5.21	PT UT45 UT45T UT45TAN UTOL	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X X X X X	- - - - -	- - - - -	W.O. 931121035 TO PERFORM NDE. THIS WELD REPLACES SUMMARY #365500, DUE TO ALARA CONCERNS. THE WELD WAS LOCATED IN THE SEAL WATER INJECTION FILTER ROOM. **3-SS-160-.451-30-SAM**
358320	3-CV-2163-7 ELBOW TO PIPE	C-F-1 C5.21	PT UT45 UT45T UT45TAN UTOL	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X - X X X	- X - - -	- - - - -	W.O. 931121035 TO PERFORM NDE. THIS WELD REPLACES SUMMARY #365520, DUE TO ALARA CONCERNS. THE WELD WAS LOCATED IN THE SEAL WATER INJECTION FILTER ROOM. THE UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT. **3-SS-160-.451-30-SAM**
358340	3-CV-2163-8 PIPE TO ELBOW	C-F-1 C5.21	PT UT45 UT45T UT45TAN UTOL	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X X X X X	- - - - -	- - - - -	W.O. 931121035 TO PERFORM NDE. THIS WELD REPLACES SUMMARY #368720, DUE TO ALARA CONCERNS. THE WELD WAS LOCATED IN THE SEAL WATER INJECTION FILTER ROOM. **3-SS-160-.451-30-SAM**
<u>3-CV-2160 (REF. DWG. NO. B-71)</u>								
359100	3-CV-2160-5 ELBOW TO PIPE	C-F-1 C5.21	PT UT45 UT45T UT45TAN UT60 UTOL	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X - X X - X	- X - - X -	- - - - -	W.O. #931121035 TO PERFORM NDE. THE UT45 AND UT60 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT. **3-SS-160-.451-30-SAM**
359120	3-CV-2160-6 PIPE TO PIPE	C-F-1 C5.21	PT UT45 UT45T UT45TAN UT60 UTOL	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X - X X - X	- X - - X -	- - - - -	W.O. #931121035 TO PERFORM NDE. THIS EXAM REPLACED SUMMARY #359000, DUE TO A HIGH NUMBER OF EXAM LIMITATIONS PRESENT. THE UT45 AND UT60 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT. **3-SS-160-.451-30-SAM**
359140	3-CV-2160-7 PIPE TO ELBOW	C-F-1 C5.21	PT UT45 UT45T UT45TAN UT60 UTOL	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X X X X X X	- - - - - -	- - - - - -	W.O. #931121035 TO PERFORM NDE. THIS EXAM REPLACED SUMMARY #359080, DUE TO A SUPPORT INTERFERENCE. **3-SS-160-.451-30-SAM**

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 35

CHEMICAL & VOLUME CONTROL

SUMMARY EXAMINATION AREA		ASME			N	O				
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	O	G			
		ITEM NO				R	E			
						E	O			
						C	M			
						-	-	**CALIBRATION BLOCK**		
<u>3-CV-2160 (REF. DWG. NO. B-72)</u>										
359840	3-CV-2160-42 PIPE TO ELBOW	C-F-1 C5.21	PT	VS1SSISZZ0003Q	X	-	-	W.O. 931121035 TO PERFORM NDE. THIS WELD REPLACED SUMMARY #359800, DUE TO ALARA CONCERNS. THE WELD WAS LOCATED INSIDE THE SEAL WATER INJECTION FILTER ROOM. THE UT45 INDICATION IS A GEOMETRIC INDICATION FROM THE WELD ROOT. **3-SS-160-.451-30-SAM**		
			UT45	VS1SSISZZ0014Q	-	X	-			
			UT45T		X	-	-			
			UT45TAN		X	-	-			
			UT0L	VS1SSISZZ0018Q	X	-	-			
359860	3-CV-2160-43 ELBOW TO PIPE	C-F-1 C5.21	PT	VS1SSISZZ0003Q	X	-	-	W.O. 931121035 TO PERFORM NDE. THIS WELD REPLACED SUMMARY #359820, DUE TO ALARA CONCERNS. THE WELD WAS LOCATED INSIDE THE SEAL WATER INJECTION FILTER ROOM. THE UT45 INDICATION IS A GEOMETRIC INDICATION FROM THE COUNTERBORE. **3-SS-160-.451-30-SAM**		
			UT45	VS1SSISZZ0014Q	-	X	-			
			UT45T		X	-	-			
			UT45TAN		X	-	-			
			UT0L	VS1SSISZZ0018Q	X	-	-			
359880	3-CV-2160-44 PIPE TO ELBOW	C-F-1 C5.21	PT	VS1SSISZZ0003Q	X	-	-	W.O. 931121035 TO PERFORM NDE. THIS WELD REPLACES SUMMARY #360000, DUE TO ALARA CONCERNS. THE WELD WAS LOCATED INSIDE THE SEAL WATER INJECTION FILTER ROOM. THE UT45 INDICATION IS A GEOMETRIC INDICATION FROM THE COUNTERBORE. **3-SS-160-.451-30-SAM**		
			UT45	VS1SSISZZ0014Q	-	X	-			
			UT45T		X	-	-			
			UT45TAN		X	-	-			
			UT0L	VS1SSISZZ0018Q	X	-	-			
<u>2-CV-2170 (REF. DWG. NO. B-81)</u>										
366000	2-CV-2170-1 REDUCER TO ELBOW	C-F-1 C5.30	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.		
<u>2-CV-2167 (REF. DWG. NO. B-90)</u>										
366620	2-CV-2167-7 PIPE TO REDUCER	C-F-1 C5.30	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.		
<u>2-CV-2164 (REF. DWG. NO. B-90)</u>										
368300	2-CV-2164-1 REDUCER TO PIPE	C-F-1 C5.30	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.		

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 36

CHEMICAL & VOLUME CONTROL

SUMMARY EXAMINATION AREA		ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	O	G	T
		ITEM NO	METHOD			R	E	H
						E	O	E
						C	M	R
								REMARKS
								**CALIBRATION BLOCK**

2-CV-2158 (REF. DWG. NO. B-90)

369620	2-CV-2158-7	C-F-1	PT	VS1SSISZZ0003Q	X	-	-	W.O.#931121035 TO PERFORM NDE.
	PIPE TO REDUCER	C5.30						

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 37

SAFETY INJECTION

SUMMARY	EXAMINATION AREA	ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI	EXAM	PROCEDURE		O	G	T
		CATGY	METHOD			R	E	H
		ITEM NO				E	O	E
						C	M	R
<b>4-SJ-2113 (REF. DWG. NO. B-98)</b>								
383820	4-SJ-2113-3 FLANGE TO PIPE	C-F-1 C5.21	PT UT45 UT45T UT45TAN UT0	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X - - - X - X - - X - - X - -			W.O.#931121035 TO PERFORM NDE. THE UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT.  **4-SS-160-.533-28-SAM-R**
383940	4-SJ-2113-15 ELBOW TO PIPE	C-F-1 C5.21	PT UT45 UT45T UT45TAN UT0	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X - - - X - X - - X - - X - -			W.O.#931121035 TO PERFORM NDE. LIMITATION: EXAM LIMITED TO 73%, DUE TO THE PROXIMITY OF THE CONCRETE STRUCTURE. THE UT45 INDICATION IS A GEOMETRIC INDICATION FROM THE WELD ROOT. **4-SS-160-.533-28-SAM-R**
<b>4-SJ-2113 (REF. DWG. NO. B-100)</b>								
384200	4-SJ-2113-29 PIPE TO ELBOW	C-F-1 C5.21	PT UT45 UT45T UT45TAN UT0L	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X - - - X - X - - X - - X - -			W.O. 931121035 TO PERFORM NDE. THIS WELD REPLACED SUMMARY #384280, DUE TO HANGER INTERFERENCE. THE UT45 INDICATIONS ARE GEOMETRIC REFLECTORS FROM THE WELD ROOT. **4-SS-160-.533-28-SAM-R**
384220	4-SJ-2113-30 ELBOW TO PIPE	C-F-1 C5.21	PT UT45 UT45T UT45TAN UT0	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X - - X - - X - - X - - X - -			W.O. 931121035 TO PERFORM NDE. THIS WELD REPLACES SUMMARY #384300, DUE TO HANGER INTERFERENCE.  **4-SS-160-.533-28-SAM-R**
384260	4-SJ-2113-32 PIPE TO FLANGE	C-F-1 C5.21	PT UT45 UT45T UT45TAN UT0L	VS1SSISZZ0003Q VS1SSISZZ0014Q  VS1SSISZZ0018Q	X - - - X - X - - X - - X - -			W.O.#931121035 TO PERFORM NDE. THE UT45 INDICATION IS A GEOMETRIC REFLECTOR FROM THE WELD ROOT.  **4-SS-160-.533-28-SAM-R**
<b>2-SJ-2126 (REF. DWG. NO. B-104)</b>								
386600	2-SJ-2126-6R REDUCER TO ORIFICE	C-F-1 C5.30	PT	VHSSSPZZ0204Q	X - -			PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD, PER DCP 1EC-3260. (REF. W.O. #930628119, ACT #5)

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 38

SAFETY INJECTION

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E O C	O G T E H M R	REMARKS
<u>2-SJ-2126 (REF. DWG. NO. B-104)</u>							
386620	2-SJ-2126-7R ORIFICE TO REDUCER	C-F-1 C5.30	PT	VHSSSPZZ0204Q	X	- -	PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD, PER DCP 1EC-3260. (REF. W.O. #930628119, ACT #5)
<u>2-SJ-2123 (REF. DWG. NO. B-105)</u>							
387100	2-SJ-2123-6R REDUCER TO ORIFICE	C-F-1 C5.30	PT	VHSSSPZZ0204Q	X	- -	PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD, PER DCP 1EC-3260. (REF. W.O. #930630065, ACT #5)
387120	2-SJ-2123-7R ORIFICE TO REDUCER	C-F-1 C5.30	PT	VHSSSPZZ0204Q	X	- -	PSI BASELINE EXAMINATION WAS PERFORMED, DUE TO THE REPLACEMENT OF THIS WELD, PER DCP 1EC-3260. (REF. W.O. #930630065, ACT #5)

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 39

AUXILIARY FEEDWATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O		
NUMBER	IDENTIFICATION	SEC. XI	EXAM	PROCEDURE	O	G	T	
		CATGY	METHOD		R	E	H	
		ITEM NO			E	O	E	REMARKS
					C	M	R	**CALIBRATION BLOCK**
-----								
<u>AF1053 (REF. DWG. NO. AF-1-3A 01)</u>								
500145	14AFWG-002 GUIDE	F-C F3.10	VT-3	S1SSISZZ0020Q	X	-	-	PRESERVICE PER 1EC-3226-01, W.O.# 930203123, CJP# PS-93-004.
500150	14AFWG-003 GUIDE	F-C F3.10	VT-3	S1SSISZZ0020Q	X	-	-	PRESERVICE PER 1EC-3226-01, W.O.# 930203123, CJP# PS-93-004.
<u>AF1063 (REF. DWG. NO. AF-1-3A 01)</u>								
500390	13AFWA-046 ANCHOR	D-A D1.20	VT-3	S1SSISZZ0020Q	X	-	-	PRESERVICE PER 1EC-3226-01, W.O.# 930203123, CJP# PS-93-004.
<u>AF1075 (REF. DWG. NO. AF-1-3A 01)</u>								
500630	12AFWG-002 GUIDE	F-C F3.10	VT-3	S1SSISZZ0020Q	X	-	-	PRESERVICE PER 1EC-3226-01, W.O.# 930203123, CJP# PS-93-004.
<u>AF1089 (REF. DWG. NO. AF-1-3A 01)</u>								
500770	11AFWS-050 SUPPORT	F-C F3.10	VT-3	S1SSISZZ0020Q	X	-	-	PRESERVICE PER 1EC-3226-01, W.O.# 930203123, CJP# PS-93-004.
500785	11AFWG-050A GUIDE	F-C F3.10	VT-3	S1SSISZZ0020Q	X	-	-	PRESERVICE PER 1EC-3226-01, W.O.# 930203123, CJP# PS-93-004.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 40

COMPONENT COOLING SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E C M R	O G T E O E	REMARKS **CALIBRATION BLOCK**
<u>CC1010 (REF. DWG. NO. CC-1-2A 02)</u>						
501255 1A-CCH-404 HANGER		VT-3				THIS SUPPORT (SUM.# 501255) WAS DELETED PER EC-3255/PS-93-050/930610082.
<u>CC1022 (REF. DWG. NO. CC-1-2A 02)</u>						
501355 1A-CCH-405 HANGER		VT-3				THIS SUPPORT (SUM.# 501355) WAS DELETED PER 1EC-3255/PS-93-050/930610082.
<u>CC1051 (REF. DWG. NO. CC-1-2A 02)</u>						
501380 1A-CCH-402 HANGER		VT-3				THIS SUPPORT (SUM.# 501380) WAS DELETED PER 1EC-3255/PS-93-050/930610082.
501390 1A-CCH-403 HANGER	F-C F3.10	VT-3				THIS SUPPORT (SUM.# 501390) WAS DELETED PER 1EC-3255/PS-93-050/930610082.
<u>CC1066 (REF. DWG. NO. CC-1-2A 02)</u>						
501425 1A-CCH-406 HANGER		VT-3				THIS SUPPORT (SUM.# 501425) WAS DELETED PER 1EC-3255/PS-93-050/930610082.
501430 1A-CCH-082 HANGER		VT-3	S1SSISZZ0020Q	X - -		PRESERVICE PER 1EC-3255-1, CJP# PS-93-50 & W.O.# 930610082.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 41

CONTAINMENT SPRAY SYSTEM

SUMMARY EXAMINATION AREA	ASME				N	O	
NUMBER IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	O	G	T
	ITEM NO	METHOD			R	E	H
					E	O	E
					C	M	R
							REMARKS
							**CALIBRATION BLOCK**

SJ1094 (REF. DWG. NO. CS-1-1A 01)

502910	1A-CSS-158	F-C	VT-3	S1SSISZZ0020Q	X	-	-	W.O.#931121035 TO PERFORM NDE. RE-EXAM
	SUPPORT	F3.10						FROM 1RFO#10.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 42

CHEMICAL AND VOLUME CONTROL SYSTEM

SUMMARY EXAMINATION AREA	ASME				N	O	
NUMBER IDENTIFICATION	SEC. XI				O	G	T
	CATGY	EXAM			R	E	H
	ITEM NO	METHOD	PROCEDURE		E	O	E
					C	M	R
					-	-	-

REMARKS  
\*\*CALIBRATION BLOCK\*\*

CV1161 (REF. DWG. NO. CV-1-2B 02)

503235	1A-CVCS-383 SUPPORT	F-C F3.10	VT-3	S1SSISZZ0020Q	-	-	X	W.O.#931121035 TO PERFORM NDE. RESCHEDULED FROM 1RFO#10. NEED TO REMOVE CONCRETE BLOCK FOR EXAM. THE INSPECTION FOUND BORON ON THE SUPPORT. W.O. #931027158 CLEANED THE BORON OFF. A RE-EXAM WAS SCHED. FOR S1RFO#12 (ACCESSIBILITY PROBLEMS).
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RC1077 (REF. DWG. NO. CV-1-4B 01)

503945	1C-CVCH-326 HANGER		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3167/PS-92-40/920805116.
503970	1C-CVCG-276 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3167/PS-92-40/920805116.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 43

MAIN STEAM SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E C M R	O G T E O M R	REMARKS
<u>MS1003 (REF. DWG. NO. MS-1-2 04)</u>							
504315	1P-MS-VS006 1 LUG	F-C F3.10	VT-3	S1SSISZZ0020Q	X - -		W.O.#931121035 TO PERFORM NDE. RE-EXAM FROM 1RFO#10.
<u>MS1112 (REF. DWG. NO. MS-1-2 04)</u>							
504535	1P-MSH-122T HANGER		VT-3/4	S1SSISZZ0020Q	X - -		PRE-SERVICE INSPECTION WAS PERFORMED UNDER W.O. #931121035 & 931002088, DUE TO THE DISASSEMBLY OF THE 12MS167 VALVE ACTUATOR. ADDITIONAL STEEL PLATE AND GOUGE MARKS IN THE SUPPORT CHANNELS WERE FOUND TO BE ACCEPTABLE PER NON DR.
<u>MS1269 (REF. DWG. NO. MS-1-2 04)</u>							
504710	1P-MSH-124T HANGER		VT-3/4	S1SSISZZ0020Q	X - -		PRE-SERVICE INSPECTION WAS PERFORMED UNDER W.O. #931121035 & 931002089, DUE TO THE DISASSEMBLY OF THE 14MS167 VALVE ACTUATOR. NO MODIFICATIONS WERE PERFORMED ON THIS HANGER. (SEE NON DR, FOR ADDITIONAL INFORMATION)

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 44

PRESSURIZING SYSTEM

SUMMARY EXAMINATION AREA NUMBER IDENTIFICATION	ASME SEC. XI CATGY EXAM ITEM NO METHOD	PROCEDURE	N O G T R E H E O E C M R	REMARKS **CALIBRATION BLOCK**
<u>RC1276 (REF. DWG. NO. RC-1-1A 01)</u>				
505065 1C-PSH-001 HANGER	F-C F3.10	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.
505066 R-239776-1	A-E NB88-11	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.
505067 R-239776-2	A-E NB88-11	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.
505068 R-239776-3	A-E NB88-11	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.
505069 R-239776-4	A-E NB88-11	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.
505070 1C-PSH-002 HANGER	F-C F3.10	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.
505071 R-239776-5	A-E NB88-11	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.
505072 R-239776-6	A-E NB88-11	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.
505073 R-239776-7	A-E NB88-11	VT-3	S1SSISZZ0020Q	X - - W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 45

PRESSURIZING SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
-----							
RC1276 (REF. DWG. NO. RC-1-1A 01)							
505075	1C-PSH-003 HANGER	F-C F3.10	VT-3	S1SSISZZ0020Q	X	-	-
					W.O.#931121035 TO PERFORM NDE. EXAMINED PER NRC BULLETIN 88-11.		

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 46

PRESSURIZER RELIEF SYSTEM

SUMMARY	EXAMINATION AREA	ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE	O	G	T
		ITEM NO	METHOD			R	E	H
						E	O	E
						C	M	R
								REMARKS
								**CALIBRATION BLOCK**

RC1287 (REF. DWG. NO. RC-1-2B 01)

505300	1C-PRH-177	VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER
	HANGER						1EC-3168/PS-92-41/920805117.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 2 SECTION XI COMPLETE COMPONENTS

PAGE: 47

RESIDUAL HEAT REMOVAL SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM		O	G T
		ITEM NO	METHOD	PROCEDURE		R	E H
						E	O E
						C	M R
							REMARKS
							**CALIBRATION BLOCK**

RH1013 (REF. DWG. NO. RH-1-1A 02)

505755	1A-RHRA-PS30 ANCHOR	F-C F3.10	VT-3	S1SSISZZ0020Q	X - -	W.O.#931121035 TO PERFORM NDE.
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DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 1 SECTION XI COMPLETE COMPONENTS

PAGE: 48

SAFETY INJECTION SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	O	G	T
		ITEM NO	METHOD	PROCEDURE	R	E	H
					E	O	E
					C	M	R
							REMARKS
							**CALIBRATION BLOCK**

RC1023 (REF. DWG. NO. RH-1-3F 01)

506195	1C-11SIS-133	F-C	VT-3	S1SSISZZ0020Q	X	-	-	W.O.#931121035 TO PERFORM NDE. RE-EXAM
	SUPPORT	F3.10						FROM 1RFO#10.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 49

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI			O	G	T
		CATGY	EXAM		R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
REMARKS							
**CALIBRATION BLOCK**							
<hr/>							
<u>SW1136 (REF. DWG. NO. SW-1-7K 01)</u>							
508920	1MSWPG-049A GUIDE	D-B D2.20	VT-3	S1SSISZZ0020Q	X	-	- THIS GUIDE WAS DISSASSEMBLED AND A NEW WELD WAS ADDED TO FACILITATE THE REMOVAL OF PIPE SPOOL RD-1-SW-75, UNDER CJP #S-92-332 AND W.O. #920512075.
<u>SW1169 (REF. DWG. NO. SW-1-3B 01)</u>							
508990	A1-SWG-364 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
508995	A1-SWA-354 ANCHOR		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509000	A1-SWG-366 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509005	A1-SWG-367 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1176 (REF. DWG. NO. SW-1-3B 01)</u>							
509010	A1-SWA-345 ANCHOR		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509015	A1-SWG-344 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509020	A1-SWG-316 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 50

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O		
NUMBER	IDENTIFICATION	SEC. XI	EXAM	PROCEDURE	O	G	T	REMARKS
		CATGY			E	H	E	
		ITEM NO	METHOD		C	M	R	**CALIBRATION BLOCK**
-----								
<u>SW1176 (REF. DWG. NO. SW-1-3B 01)</u>								
509025	A1-SWA-315 ANCHOR	D-B D2.20	VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509030	A1-SWH-314 HANGER		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509035	A1-SWG-313 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509040	A1-SWG-324 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509045	A1-SWH-323 HANGER		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509050	A1-SWA-WS08 ANCHOR	D-B D2.20	VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1181 (REF. DWG. NO. SW-1-3B 01)</u>								
509055	A1-SWG-378 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509060	A1-SWA-358 ANCHOR		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509065	A1-SWG-377 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 51

SERVICE WATER SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E C M R	O G T E H O E R	REMARKS
							**CALIBRATION BLOCK**
<u>SW1181 (REF. DWG. NO. SW-1-3B 01)</u>							
509070	A1-SWG-376 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509075	A1-SWG-348 GUIDE		VT-3				THIS SUPPORT (SUM.# 509075) WAS DELETED PER 1EC-3149/PS-93-030/930422073.
509080	A1-SWG-347 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509085	A1-SWG-346 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1185 (REF. DWG. NO. SW-1-3B 01)</u>							
509090	A1-SWS-369 SUPPORT		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509095	A1-SWG-368 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509100	A1-SWA-356 ANCHOR		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509105	A1-SWG-370 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1191 (REF. DWG. NO. SW-1-1B 02)</u>							
509110	P1-SWA-WS12 ANCHOR	D-B D2.20	VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE PER 1EC-3150-1/PS-93-006/930615162.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 52

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N O		REMARKS
NUMBER	IDENTIFICATION	SEC. XI	EXAM	PROCEDURE	O	G	
		CATGY			R	E	
		ITEM NO			E	O	
					C	M	
-----							**CALIBRATION BLOCK**
<u>SW1197 (REF. DWG. NO. SW-1-1A 02)</u>							
509160	C-13SWH-2 HANGER		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
509165	C-13SWG-3C GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1220 (REF. DWG. NO. SW-1-1A 02)</u>							
509270	C-12SWH-1 HANGER		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
509275	C-11SWG-3B GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
509280	C-11SWG-4B GUIDE		VT-3				THIS SUPPORT (SUM.# 509280) WAS DELETED PER 1EC-3054-2/PS-93-021/920115113.
509285	C-11SWG-5B GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
509290	C-12SWG-3A GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
509295	C-12SWG-4 ANCHOR		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
509300	C-11SWG-9B GUIDE		VT-3				THIS SUPPORT (SUM.# 509300) WAS DELETED PER 1EC-3054-2/PS-93-021/920115113.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 53

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N O		REMARKS				
NUMBER	IDENTIFICATION	SEC. XI	EXAM	METHOD	PROCEDURE	G T					
		CATGY				R E H					
		ITEM NO				E O E					
						C M R					
-----							**CALIBRATION BLOCK**				
-----											
<u>SW1220 (REF. DWG. NO. SW-1-1A 02)</u>											
509305	C-11SWG-11A GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.				
509310	C-11SWG-6 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.				
509315	C-12SWG-7A GUIDE		VT-3		S1SSISZZ0020Q	X - -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.				
509320	C-12SWH-9 HANGER		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.				
<u>SW1220 (REF. DWG. NO. SW-1-5B 02)</u>											
509345	C-12SWG-12 GUIDE	F-C F3.10	VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.				
<u>SW1238 (REF. DWG. NO. SW-1-1A 02)</u>											
509370	C-11SWH-1 HANGER		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.				
509375	C-11SWG-3A GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.				
509380	C-11SWG-4A GUIDE		VT-3				THIS SUPPORT (SUM.# 501390) WAS DELETED PER 1EC-3054-2/PS-93-021/920115113.				
509385	C-11SWG-5A GUIDE		VT-3		S1SSISZZ0020Q	X - -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.				

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 54

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	REMARKS
NUMBER	IDENTIFICATION	SEC. XI	EXAM	METHOD	PROCEDURE	O	
		CATGY				G	
		ITEM NO				T	
						R	
						E	
						O	**CALIBRATION BLOCK**
						E	
						M	
						R	
<u>SW1238 (REF. DWG. NO. SW-1-1A 02)</u>							
509395	C-11SWG-8A GUIDE	D-B D2	VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
509400	C-11SWG-9A GUIDE		VT-3		S1SSISZZ0020Q	X - -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
509405	C-11SWH-10 HANGER		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1238 (REF. DWG. NO. SW-1-5A 02)</u>							
509425	C-11SWG-13 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1365 (REF. DWG. NO. SW-1-3B 01)</u>							
509525	A1-SWG-343 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509530	A1-SWG-325 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1366 (REF. DWG. NO. SW-1-3B 01)</u>							
509535	A1-SWG-326 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509540	A1-SWG-327 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 55

SERVICE WATER SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E C M R	O G T E H E R	REMARKS **CALIBRATION BLOCK**
<u>SW1366 (REF. DWG. NO. SW-1-38 01)</u>							
509545	A1-SWG-304 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509550	1A-SWA-303A GUIDE	D-B D2.20	VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509555	A1-SWG-303 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509560	A1-SWH-302 HANGER		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509565	A1-SWG-301 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509570	A1-SWG-300 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509575	A1-SWG-318 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509580	A1-SWH-317 HANGER		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509585	A1-SWA-WS05 ANCHOR	D-B D2.20	VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RFP)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 56

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	R	E	H
		ITEM NO	METHOD	PROCEDURE	E	O	E
					C	M	R
					-	-	-
REMARKS							
**CALIBRATION BLOCK**							
-----							
<u>SW1373 (REF. DWG. NO. SW-1-3B 01)</u>							
509590	A1-SWG-328 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-
					THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.		
509595	A1-SWG-329 GUIDE		VT-3				
					THIS SUPPORT (SUM.# 509595) WAS DELETED PER 1EC-3149/PS-93-030/930422073.		
<u>SW1377 (REF. DWG. NO. SW-1-3B 01)</u>							
509600	A1-SWS-351 SUPPORT		VT-3				
					THIS SUPPORT (SUM.# 509600) WAS DELETED PER 1EC-3149/PS-93-030/930422073.		
509605	A1-SWG-352 GUIDE		VT-3				
					THIS SUPPORT (SUM.# 509605) WAS DELETED PER 1EC-3149/PS-93-030/930422073.		
<u>SW1384 (REF. DWG. NO. SW-1-1A 02)</u>							
509635	C-14SWH-1 HANGER		VT-3	S1SSISZZ0020Q	X	-	-
					THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.		
509640	C-13SWG-3D GUIDE		VT-3	S1SSISZZ0020Q	X	-	-
					PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.		
<u>SW1496 (REF. DWG. NO. SW-1-9H 01)</u>							
509805	1A-SWS-091 SUPPORT		VT-3	S1SSISZZ0020Q	X	-	-
					THIS SUPPORTS VERTICAL MEMBERS WERE REPLACED UNDER CJP #S-93-141 AND W.O. #920530148.		
<u>SW1498 (REF. DWG. NO. SW-1-3B 01)</u>							
509815	A1-SWS-340 SUPPORT		VT-3	S1SSISZZ0020Q	X	-	-
					THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.		

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 57

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N	O	
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	O	G	T
		ITEM NO	METHOD	PROCEDURE	R	E	H
					E	O	E
					C	M	R
							REMARKS
							***CALIBRATION BLOCK**
-----							
<u>SW1541 (REF. DWG. NO. SW-1-3B 01)</u>							
509840	A1-SWG-349 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509845	A1-SWG-330 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509850	A1-SWS-350 SUPPORT		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509855	A1-SWG-382 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509860	A1-SWA-357 ANCHOR		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509865	A1-SWG-385 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1542 (REF. DWG. NO. SW-1-3B 01)</u>							
509870	A1-SWS-332 SUPPORT		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509875	A1-SWG-312 GUIDE		VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509880	A1-SWA-311 ANCHOR	D-B D2.20	VT-3	S1SSISZZ0020Q	X	-	- THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 58

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N O G T O G T R E H E O E C M R			REMARKS
NUMBER	IDENTIFICATION	SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE				**CALIBRATION BLOCK**
-----								
<u>SW1542 (REF. DWG. NO. SW-1-3B 01)</u>								
509885	A1-SWG-310 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509890	A1-SWG-309 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1545 (REF. DWG. NO. SW-1-3B 01)</u>								
509895	A1-SWG-341 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509900	A1-SWG-342 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1546 (REF. DWG. NO. SW-1-3B 01)</u>								
509905	A1-SWS-331 SUPPORT		VT-3					THIS SUPPORT (SUM.# 509905) WAS DELETED PER 1EC-3149/PS-93-030/930422073.
509910	A1-SWS-339 SUPPORT		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509915	A1-SWG-375 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509920	A1-SWS-374 SUPPORT		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509925	A1-SWA-355 ANCHOR		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 59

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME			N O		REMARKS
NUMBER	IDENTIFICATION	SEC. XI	EXAM	METHOD	PROCEDURE	O G T	
		CATGY				R E H	
		ITEM NO				E O E	
						C M R	
-----							-----
<u>SW1546 (REF. DWG. NO. SW-1-3B 01)</u>							
509930	A1-SWG-384 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1550 (REF. DWG. NO. SW-1-3B 01)</u>							
509935	A1-SWA-353 ANCHOR		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509940	A1-SWG-359 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509945	A1-SWG-360 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509950	A1-SWG-361 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509955	A1-SWS-334 SUPPORT		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509960	A1-SWG-333 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509965	A1-SWG-383 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>(REF. DWG. NO. SW-1-3B 01)</u>							
509966	A1-SWG-2433 GUIDE		VT-3		S1SSISZZ0020Q	X - -	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073. ANCHOR A1-SWA-383 WAS REPLACED WITH GUIDE A1-SWG-2433.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 60

SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME				N	O	
NUMBER	IDENTIFICATION	SEC. XI	EXAM	METHOD	PROCEDURE	O	G	T
		CATGY				R	E	H
		ITEM NO				E	O	E
						C	M	R
						REMARKS		
						**CALIBRATION BLOCK**		
-----								
<u>SW1551 (REF. DWG. NO. SW-1-3B 01)</u>								
509970	A1-SWG-322 GUIDE		VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509975	A1-SWH-321 HANGER		VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509980	A1-SWA-WS07 ANCHOR	D-B D2.20	VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1556 (REF. DWG. NO. SW-1-3B 01)</u>								
509985	A1-SWS-338 SUPPORT		VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509990	A1-SWS-336 SUPPORT		VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
509995	A1-SWS-335 SUPPORT		VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
510000	A1-SWG-337 GUIDE		VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1557 (REF. DWG. NO. SW-1-3B 01)</u>								
510005	A1-SWG-308 GUIDE		VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
510010	1A-SWA-307A GUIDE	D-B D2.20	VT-3		S1SSISZZ0020Q	X	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 61

SERVICE WATER SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O O G T R E H E O E C M R			REMARKS
								**CALIBRATION BLOCK**
<u>SW1557 (REF. DWG. NO. SW-1-3B 01)</u>								
510015	A1-SWG-307 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
510020	A1-SWG-306 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
510025	A1-SWG-305 GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
510030	A1-SWA-WS06 ANCHOR	D-B D2.20	VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
<u>SW1619 (REF. DWG. NO. SW-1-1A 02)</u>								
510135	C-14SWH-1 HANGER		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510225	C-13SWG-3B GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1640 (REF. DWG. NO. SW-1-1A 02)</u>								
510250	C-13SWH-1 HANGER		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510255	C-13SWG-3A GUIDE		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1664 (REF. DWG. NO. SW-1-1A 02)</u>								
510395	C-12SWH-2 HANGER		VT-3	S1SSISZZ0020Q	X	-	-	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 62

SERVICE WATER SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E O E C M R	O G T R E H O E R	REMARKS **CALIBRATION BLOCK**
<u>SW1664 (REF. DWG. NO. SW-1-1A 02)</u>							
510400	C-11SWG-3D GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510405	C-11SWG-4D GUIDE		VT-3				THIS SUPPORT (SUM.# 501405) WAS DELETED PER 1EC-3054-2/PS-93-021/920115113.
510410	C-11SWG-5D GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
510415	C-12SWG-3B GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
510420	C-12SWG-5 ANCHOR		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510425	C-11SWG-9D GUIDE		VT-3				THIS SUPPORT (SUM.# 510425) WAS DELETED PER 1EC-3054-2/PS-93-021/920115113.
510430	C-11SWG-11C GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510433	C-12SWG-6A GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
510435	C-12SWG-5B GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (SIRFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

PAGE: 63

SERVICE WATER SYSTEM

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	ASME SEC. XI CATGY ITEM NO	EXAM METHOD	PROCEDURE	N O R E H E O E C M R	O G T R E H E O E C M R	REMARKS  **CALIBRATION BLOCK**
-----							
<u>SW1664 (REF. DWG. NO. SW-1-1A 02)</u>							
510440	C-12SWG-7B GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
510445	C-12SWG-8 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
510450	C-12SWG-10 GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510455	C-12SWH-11 HANGER		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1664 (REF. DWG. NO. SW-1-5B 02)</u>							
510485	C-12SWG-16 GUIDE	F-C F3.10	VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1684 (REF. DWG. NO. SW-1-5A 02)</u>							
510520	C-11SWG-17 GUIDE	F-C F3.10	VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1684 (REF. DWG. NO. SW-1-1A 02)</u>							
510555	C-11SWH-2 HANGER		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510560	C-11SWG-3C GUIDE		VT-3	S1SSISZZ0020Q	X	- -	THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.

DATE: 03/04/94  
REVISION: 0

SALEM NUCLEAR POWER STATION UNIT 1  
OUTAGE ISI EXAMINATION RESULTS (S1RFO#11, REV.0)  
SECOND INTERVAL, SECOND PERIOD, SECOND OUTAGE (93RF)  
CLASS 3 SECTION XI COMPLETE COMPONENTS

**PAGE: 64**

## SERVICE WATER SYSTEM

SUMMARY EXAMINATION AREA		ASME					
NUMBER	IDENTIFICATION	SEC. XI	CATGY	EXAM	PROCEDURE		REMARKS
		ITEM NO	METHOD				**CALIBRATION BLOCK**
<u>SW1684 (REF. DWG. NO. SW-1-1A 02)</u>							
510565	C-11SWG-4C GUIDE		VT-3				THIS SUPPORT (SUM.# 510565) WAS DELETED PER 1EC-3054-2/PS-93-021/920115113.
510570	C-11SWG-5C GUIDE		VT-3	S1SSISZZ0020Q	X - -		PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
510575	C-11SWG-7 ANCHOR		VT-3	S1SSISZZ0020Q	X - -		THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510580	C-11SWG-8B GUIDE		VT-3	S1SSISZZ0020Q	X - -		PRESERVICE (NO MOD.) PER 1EC-3054-2/PS-93-021/920115113.
510585	C-11SWG-9C GUIDE		VT-3				THIS SUPPORT (SUM.# 501585) WAS DELETED PER 1EC-3054-2/PS-93-021/920115113.
510590	C-11SWG-11B GUIDE		VT-3	S1SSISZZ0020Q	X - -		THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
510595	C-11SWH-12 HANGER		VT-3	S1SSISZZ0020Q	X - -		THIS SUPPORT WAS MODIFIED PER 1EC-3054-2/PS-93-021/920115113.
<u>SW1986 (REF. DWG. NO. SW-1-3B 01)</u>							
510785	A1-SWG-320 GUIDE		VT-3	S1SSISZZ0020Q	X - -		THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.
510790	A1-SWH-319 HANGER		VT-3	S1SSISZZ0020Q	X - -		THIS SUPPORT WAS MODIFIED PER 1EC-3149/PS-93-030/930422073.

# Index of NIS-2 and NR-1 Forms

## I. NIS-2 Forms

<u>Component</u>	<u>Work Order</u>	<u>Brief Description</u>
1CVE43	920514012	Replaced charging pump oil cooler
11CS5		Replaced disk & nozzle
1SJ32		Replaced disk
1PR3		Installed modified flexi-disc
1PR4		Installed modified flexi-disc
1PR5		Installed modified flexi-disc
2PR3 (spare)		Installed modified flexi-disc
1CCE5		Modified head with manway

## II. NR-1 Forms

<u>Code Job #</u>	<u>Brief Description</u>
S92-379	Replaced studs, nuts and bonnets in kind
S92-392	Replace valve bolting
S93-008	Replaced valve bolting
S93-036	Replaced stem and valve bolting
S93-082	Repaired vane holes and taper pins
S93-108	Replaced bolting with upgraded material
S93-114	Replaced valve disc
S93-124	Replaced valve disc
S93-149	Replaced modified miniflow orifice
S93-150	Replaced modified miniflow orifice
S93-171	Replaced valve stem, bolting and weld repair on leak-off
S93-181	Replaced valve with valve of new design
S93-182	Replaced valve with valve of new design
S93-183	Replaced valve with valve of new design
S93-184	Replaced valve with valve of new design
S93-190	Replaced blind flange on fuel transfer canal in kind
S93-192	Replaced stem
S93-193	Replaced stem
S93-207	Replaced bonnet stud
S93-212	Weld repairs on valve body
S93-218	Replaced snubber serial # 330 with snubber serial # 23687
S93-220	Replaced transition tubes assemblies
S93-221	Replace bonnet bolting
S93-225	Replaced flange and spool, elbow and bolting in kind
S93-226	Replaced plug and assembly in kind
S93-227	Replaced snubber serial # 25784 with snubber serial # 27164
S93-228	Replaced snubber serial # 445 with snubber serial # 27629

S93-229	Replaced snubber serial # 11 with snubber serial # 3903
S93-233	Re-weld support after inspection and re-assembly of valve
S93-238	Replaced flange to valve bolting
S93-240	Replaced valve bolting
S93-241	Replaced bolting and nuts in kind
S93-249	Replace (16) studs in kind
S93-250	Replaced valve bolting
S93-254	Replaced pressurizer manway cover and bolting
S93-260	Replaced pressurizer relief tank rupture disc flanges
S93-262	Replace (1) bolt in kind on #14 Steam Generator Primary Manway Cold Leg
S93-266	Replace valve disc
S93-270	Replace valve with spare valve
S93-276	Replace (4) cap screws and valve disc
S93-277	Replaced bolting
S93-278	Installed new stop pin for air lock equalizing ball valve
S93-279	Replaced valve in kind
S93-287	Replaced snubber serial # 25784 with snubber serial # 27164
PS92-040	Modified existing support
PS92-041	Modified pipe support
PS93-012	Replaced stem assembly and bolting with upgraded material and replaced valve bolting
PS93-045	Replaced GB4 valve internals
PS93-053	Replaced valve bolting and valve internals
PS93-054	Replaced valve bolting and valve internal
PS93-052	Modified pressurizer piping supports
PS93-055	Modified pressurizer relief tank small bore piping
PS93-070	Added break flanges to pipe spool piece 1CA2310
PS93-073	Removal of boric acid blender
PS93-074	Replace valve internals, bolting and nuts
PS93-075	Install new condensate chambers for #11 main steam header
PS93-076	Install new condensate chambers for #12 main steam header
PS93-077	Install new condensate chambers for #13 main steam header
PS93-078	Install new condensate chambers for #14 main steam header
PS93-079	Install supports for condensate pots on #11 main steam header
PS93-080	Install supports for condensate pots on #12 main steam header
PS93-081	Install supports for condensate pots on #13 main steam header
PS93-082	Install supports for condensate pots on #14 main steam header
PS93-083	Modify pipe support C-RVVS-5
PS93-088	Replace 2 nuts on pipe support #P-CTAG-660

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Co. Date September 21, 1993  
Name  
End of Buttonwood Rd. Hancocks Bridge, NJ  
Address
2. Plant Salem Nuclear Generating Station Sheet 1 of 1  
Name  
End of Buttonwood Rd., Hancocks Bridge, NJ Unit 1  
Address WO# 920514012 CJP # S91-324  
Repair Organization P.O. No., Job No., etc.
3. Work Performed by PSE&G Type Code Symbol Stamp NR  
Name  
End of Buttonwood Rd., Hancocks Bridge, NJ Authorization No. 36  
Address Expiration Date 10/3/94
4. Identification of System Chemical Volume Control
5. (a) Applicable Construction Code N/A 19 Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 83 Summer 83

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
1CVE43	ITT Standard	N/A	N/A	N/A	N/A	Replaced	No
1CVE43	ITT Standard	5-142-05 014042	N/A	N/A	N/A	Replacement	No

7. Description of Work Replaced Charging Pump Oil Cooler

8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐  
\* Other ☒ Pressure NOP psi Test Temp. NOT °F  
\* System Inservice Test

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks \_\_\_\_\_

Applicable Manufacturer's Data Reports to be attached

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp NR

Certificate of Authorization No. 36

Expiration Date 10/3/94

Signed *J. Schutler Principal Engr*  
Owner or Owner's Designee, Title

Date 9-28, 19 93

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the components described in this Owner's Report during the period 7-92 to 6-93, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Factory Mutual Engr. Assn.

*James E. Loh*  
Inspector's Signature

Commissions NJ 373

National Board, State, Province, and Endorsements

Date 10-12, 19 93

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Date Nov. 2, 1993  
Name  
End of Alloway Creek Neck Rd. Hancocks  
Address Bridge, NJ 08038 Sheet One of One
2. Plant Salem Unit One  
Name  
End of Alloway Creek Neck Rd. Hancocks PO# P1-426636  
Address Bridge, NJ 08038 Crane-Aloyco Inc. NR Travel# 366  
Repair Organization P.O. No., Job No., etc.
3. Work Performed by Crane-Aloyco Inc. Type Code Symbol Stamp VR NR  
Name  
104 N. Chicago St; Joliet, IL 60434 Authorization No. 50,138,261,262 24,63  
Address Expiration Date July 2, 1996 Feb. 17, 1994
4. Identification of System Containment Spray (11CS5)
5. (a) Applicable Construction Code N/A 19 N/A Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 83 Summer '83 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
11CS5							
Disk	Crosby Valve	84616-39-0040	N/A	N/A	N/A	Replaced	No
Nozzle	Crosby Valve	088027-16-0024	N/A	N/A	N/A	Replaced	No

7. Description of Work Replaced Disk & Nozzle, Lapped & Set Tested

8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐  
Other ☒ Pressure 260 psi Test Temp. Ambient°F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

# FORM NIS-2 (Back)

9. Remarks

Applicable Manufacturer's Data Reports to be attached

## CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp VR NR  
50, 138, 261, 262 24, 62 VR NR  
 Certificate of Authorization No. \_\_\_\_\_ Expiration Date July 2, 1996 Feb. 17, 1994  
 Signed *[Signature]* Date November 22<sup>nd</sup>, 19 93  
 Owner or Owner's Designee, Title

## CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Insurance Co. of Norwood, MA have inspected the components described in this Owner's Report during the period 10/24/93 to 11-9-93, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*[Signature]* Commissions PA 2056 NIBS JSWJ 431  
 Inspector's Signature National Board, State, Province, and Endorsements

Date 11/24, 19 93

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Date Nov. 2, 1993  
Name  
 End of Alloway Creek Neck Rd. Hancocks  
Address Bridge, NJ 08038 Sheet One of One

2. Plant Salem Unit One  
Name PO# P1-426636  
 End of Alloway Creek Neck Rd. Hancocks Crane-Aloyco Inc. NR Travel# 363  
Address Bridge, NJ 08038 Repair Organization P.O. No., Job No., etc.  
 3. Work Performed by Crane-Aloyco Inc. Type Code Symbol Stamp VR NR  
Name Authorization No. 50,138,261,262 24,63  
104 N. Chicago St. Joliet, IL 60434 Expiration Date July 2, 1996 Feb. 17, 1994  
Address

4. Identification of System Safety Injection

5. (a) Applicable Construction Code N/A 19 N/A Edition, N/A Addenda, N/A Code Case  
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 83 Summer 83 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
1SJ32							
Disc	Crosby Valve	N95137-0035	N/A	N/A	N/A	Replaced	No

7. Description of Work Replaced Disk, Cleaned, Lapped and Set Tested

8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐  
 Other ☒ Pressure 220 psi Test Temp. Ambient °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks \_\_\_\_\_

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp VR NR  
50, 138, 261, 262 24, 63 VR NR  
 Certificate of Authorization No. \_\_\_\_\_ Expiration Date July 2, 1996-Feb. 17, 1994  
 Signed Albino Principal Eng. Date November 22, 19 93  
 Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Insurance Co. of Norwood, MA have inspected the components described in this Owner's Report during the period 10-26-93 to 11-9-93, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Factory Mutual Engr. Assn.

MS Lewis Commissions PA 2056 NIBS in NJ 431  
 Inspector's Signature National Board, State, Province, and Endorsements

Date 11/24/ 19 93

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Co. Date 1/25/94  
Name  
End of Buttonwood Rd., Hancocks Bridge, NJ  
Address
2. Plant Salem Generating Station Unit 1  
Name  
End of Buttonwood Rd., Hancocks Bridge, NJ  
Address Pl-931602  
Repair Organization P.O. No., Job No., etc.
3. Work Performed by Wyle/Crosby Type Code Symbol Stamp NR  
Name  
7800 Governors Dr., Hunstville ALA 35807  
Address Authorization No. 63  
Expiration Date 2/17/94
4. Identification of System Reactor Coolant  
ASME Sect.
5. (a) Applicable Construction Code III 19 68 Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 83 S83

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
1PR3	Crosby	N69878-01-0001	N/A	N/A	1968	Repaired	No
1PR4	Crosby	RV-466	N/A	N/A	1968	Repaired	No
1PR5	Crosby	RV-2-8010A	N/A	N/A	1968	Repaired	No
2PR3 Spare	Crosby	RV-464	N/A	N/A	1968	Repaired	No

7. Description of Work Installed modified flexi-disc - 2 and refurbished all valves.

8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐  
Other ☒ Pressure 1PR3-2485psi Test Temp. 1PR3-654 °F 2PR3 (Spare) 2482PSI 635°F  
1PR4-2496 1PR4-656 °F  
1PR5-2492 1PR5-664 °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks \_\_\_\_\_

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Repair conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp NR

Certificate of Authorization No. 63 Expiration Date 2/17/94

Signed [Signature] Date 1-28, 19 94  
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Ins. Co. of Norwood, MA have inspected the components described in this Owner's Report during the period Sept. 1993 to Jan. 1994, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection. Factory Mutual Engr. Assn.

[Signature] Commissions NJ373 I  
Inspector's Signature National Board, State, Province, and Endorsements

Date 1-28, 19 94

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Public Service Electric & Gas Co. Date February 7, 1994  
Name  
End of Buttonwood Rd., Hancocks Bridge, NJ  
Address
2. Plant Salem Nuclear Generating Station Unit 1  
Name  
End of Buttonwood Rd., Hancocks Bridge, NJ WO #931023024 CJP PS-93-086  
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by Joseph Oat Corporation Type Code Symbol Stamp U1  
Name  
2500 Broadway, Camden, NJ 08104 Authorization No. U-184  
Address Expiration Date 7/9/94
4. Identification of System Component Cooling
5. (a) Applicable Construction Code ASME VIII 19 68 Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 83

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
1CCE5	Jos-Oats Corp.	S-15763A	1148	N/A	1970	Replaced	Y
1CCE5	Jos-Oats Corp.	2529	N/A	N/A	1993	Replacement	N

7. Description of Work Modified existing flat head by installing Manway S/N 2529.

8. Tests Conducted: Hydrostatic ☒ Pneumatic ☐ Nominal Operating Pressure ☐  
Other ☐ Pressure 167 psi Test Temp. 61.2 °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Attached: U-2A, Manufacturer's Partial Data Report from Jos. Oat Corp.  
Applicable Manufacturer's Data Reports to be attached  
for Manway and Cover.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code, Section XI. repair or replacement

Type Code Symbol Stamp NR

Certificate of Authorization No. 36 Expiration Date 10/3/94

Signed J. Shubert Principal Eng Date Feb 7, 19 94  
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New Jersey and employed by Arkwright Mutual Ins. Co. of Norwood, MA have inspected the components described in this Owner's Report during the period Sept. 1993 to Feb., 1994, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection. Factory Mutual Engr. Assn.

J. Shubert Commissions NB 11207 INA, NJ 1020  
Inspector's Signature National Board, State, Province, and Endorsements

Date 28 MARCH 19 94

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS Page 1 of 1

1. Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-379  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical Volume Control Nuclear Class II

5. a: Component repaired, modified or replaced 11CV155  
b: Name of manufacturer ITT Grinnell  
c: Identifying nos. SD-C-112277 N/A N/A N/A 1988  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code B31.7 1969 N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 N/A N/A  
(edition) (addenda) (Code Case(s))  
\*B31.7

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Replaced studs, nuts and bonnet in kind.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920921216 Deficiency Report #: SMD 92-736  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*Inservice Leak Exam Temp. NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 10-26 19 92 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 10-7, 19 92 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 10-28, 19 92 Signed [Signature] Commissions Factory Mutual System  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S92-392  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Containment Spray Nuclear Class II

5. a: Component repaired, modified or replaced 12CS36 Valve

b: Name of manufacturer Aloyco

c: Identifying nos. 12024/S70WDD N/A N/A N/A 1987  
(mfr's. serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code ASME DP&V 1968 N/A N/A II  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1968 N/A N/A  
\*ASME DP&V/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

Description of work Replaced valve bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920921177 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Report Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 11-9 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-8, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11-22, 1993 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 5844 INA NJ 373  
(Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-008  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical Volume Control Nuclear Class II

5. a: Component repaired, modified or replaced 1CV284 Valve  
b: Name of manufacturer Anchor Darling  
c: Identifying nos. 9847-70 N/A N/A N/A 1970  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 4GM42FBH N/A N/A II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 4GM42FBH (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 921028084 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-18 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-18 1994 Signed [Signature] Factory Mutual Engr. Assn. Commissions NB5844 INA NJ373  
(Inspector) (Nat'l.Bd.No.(including endorsements)state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-036  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced 1PS28 Valve  
b: Name of manufacturer Velan  
c: Identifying nos. 8117 N/A N/A N/A 1977  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME DP&V 1968 N/A N/A I  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1968 N/A N/A  
\*ASME DP&V/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replace stem and valve bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920804179 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-14 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-17 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB S894 IN/A NJ 373  
(Nat'l.Bd.No.(including endorsements)state or province and number)

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-082  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Service Water Nuclear Class II

5. a: Component repaired, modified or replaced 13SW58  
b: Name of manufacturer Masonellian International Inc.  
c: Identifying nos. N-00113-40-3 N/A N/A N/A 1974  
ASME Draft P&V 1968 Ed. Mar 1970 Add. (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 N/A N/A  
(edition) (addenda) (Code Case(s))  
\*ANSI B31.7 / ASME Draft P&V 1968 Ed. Mar 1970 Add.

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Repaired vane holes and taper pins by welding.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930331103 Deficiency Report #: SMD 93-072  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*Inservice Leak Exam Temp. NOT

### CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 4-30 19 93 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

### CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 4/1, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date May 20 19 93 Signed [Signature] Commissions NB 7666 NJ 1028  
(Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-108  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical Volume Control Nuclear Class II

5. a: Component repaired, modified or replaced 1CVC6 Tank  
b: Name of manufacturer Jos. Oat & Sons, Inc.  
c: Identifying nos. 1781-2A 374 N/A N/A 1969  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME III Class C 1968 N/A N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/ASME III Class C 1968 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replaced bolting with upgraded material.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930409094 Deficiency Report #: N/A  
Design Change Package #: 1EO-2337 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 11-30 19 93 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11/8, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-2 19 93 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 6344 U/RSW NJ 451  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-114  
(name) (repair organization's P.O. no., job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. a: Component repaired, modified or replaced 12SJ156 Valve

b: Name of manufacturer Velan

c: Identifying nos. 78704 N/A N/A N/A 1968  
(mfr's use E-Spec) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code 676241 N/A N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*676241 N/A N/A  
(edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G \*W E-Spec \*\* NOP N/A

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve disc.

(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920713003 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 94

PSE&G Co.-Nuclear Dept.

Date 6/7 19 93 Signed Allen J. Thomas Roberts Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on June 7, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 6-7 19 93 Signed William R. Rogers Commissions NB7980 NIBS ES PA2204  
(Inspector) (Nat'l Bd. No. including endorsement(s) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-124  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. a: Component repaired, modified or replaced 13SJ156 Valve  
b: Name of manufacturer Velan  
c: Identifying nos. 78704 N/A N/A N/A 1968  
(mfr's use only) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code 676241 N/A N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*76241 N/A N/A  
(edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G \*W E-Spec

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve disc.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 900715077 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994  
Date 6/7 19 93 Signed PSE&G Co. - Nuclear Dept. Allen J. Thomas Robert III Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co  
of MA have inspected the repair, modification or replacement described in this report on JUNE 7, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 6-7 19 93 Signed William R. Rogers II Commissions NB7980 NIBSIS PA2204  
(Inspector) (Nat'l. Bd. No. including endorsements, state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CIP #S93-149  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Chemical Volume Control (CVC) Nuclear Class II
5. a: Component repaired, modified or replaced No. 11 Charging Pump - Miniflow Orifice (Comp ID 1CVE20)  
b: Name of manufacturer Pacific Pump Inc.  
c: Identifying nos. 45603 N/A N/A N/A 1970  
(mfr's. serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 677125 N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A N/A N/A  
(edition) (addenda) (Code Case(s))  
\*W E-Spec 677125
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☒ pneumatic ☐ design pressure ☐ \*\* pressure 3550 psi Code Case(s) N/A
11. Description of work The modified miniflow orifice was replaced in accordance with the  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
DCP 1EC-3260 Pkg. 1 Instructions.
12. Remarks: Work Order #: 930628119 Deficiency Report #: N/A  
Design Change Package #: 1EC-3260 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test @ NOP/NOT Temp. 81°F

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 11-23 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 9/9, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11/29 1993 Signed MS Hamer Factory Mutual Engr. Assn.  
(Inspector) (Nat'l Bd. No. (including endorsements) state or province and number) A9 2056 N15315 NJ 431

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. GTP #S93-150  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical Volume Control (CVC) Nuclear Class II

5. a: Component repaired, modified or replaced No. 12 Charging Pump - Miniflow Orifice (Comp ID 1CVC21)  
b: Name of manufacturer Pacific Pump Inc.  
c: Identifying nos. 45604 N/A N/A N/A 1970  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 677125 N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A N/A N/A  
(edition) (addenda) (Code Case(s))  
\*W E-Spec 677125

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☒ pneumatic ☐ design pressure ☐ \*\* pressure 3550 psi Code Case(s) N/A

11. Description of work The modified miniflow orifice was replaced in accordance with the  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
DCP 1EC-3260 Pkg. 1 Instructions.

12. Remarks: Work Order #: 930630065 Deficiency Report #: N/A  
Design Change Package #: 1EC-3260 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test @ NOP/NOT Temp. 81°F

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 11-23 19 93 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 9/9, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11/27, 19 93 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions A2056 NIBS13 NS 431  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CTP #S93-171  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class II

5. a: Component repaired, modified or replaced 11SJ134 Valve  
b: Name of manufacturer Velan  
c: Identifying nos. 1540 N/A N/A N/A 1988  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 676258 N/A N/A 2  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 676258 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve stem, bolting and performed weld repair on leak-off  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

line.

12. Remarks: Work Order #: 930708104 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 11-30 19 93. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11/30, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11/30, 19 93. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions PA 2056 NIBS 15 NJ 431  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CTP #S93-181  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Safety Injection Nuclear Class I
5. a: Component repaired, modified or replaced 11SJ17 Valve  
b: Name of manufacturer Rockwell - Edwards  
c: Identifying nos. N/A N/A N/A N/A 1968  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code WE-Spec 677264 N/A N/A 1  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 677264 (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A
11. Description of work Replaced valve with valve of new design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 930709175 Deficiency Report #: N/A  
Design Change Package #: 1EO-2309 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-26 1994 Signed James E. Gosh Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Page 2 of 2Code Job Package No.: S93-181

1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
4. System: Safety Injection Nuclear Class: I

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 11SJ17 ValveName of Manufacturer: Edward Valve Inc.

AFX-55 N/A N/A N/A 1993 Yes  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \*ASME III 1989 Edition N/A Addenda, Code Class 1  
Code Cases: N/A  
\*ANSI B31.7 1969 Ed. 1970 Add.

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. GTP #S93-182  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. a: Component repaired, modified or replaced 12SJ17 Valve  
b: Name of manufacturer Rockwell Edwards  
c: Identifying nos. N/A N/A N/A N/A 1968  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code WE-Spec 677264 N/A N/A 1  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 677264 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve with valve of new design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 921117147 Deficiency Report #: N/A  
Design Change Package #: 1EO-2309 Pkg. 2  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 19 94 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-26 19 94 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NT373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: S93-182

1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System: Safety Injection Nuclear Class: I

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 12SJ17 ValveName of Manufacturer: Edward Valve Inc.

AFX-54 N/A N/A N/A 1993 Yes  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \*ASME III 1989 Edition N/A Addenda, Code Class 1  
Code Cases: N/A  
\*ANSI B31.7 1969 Ed. 1970 Add.

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CTP #S93-183  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Safety Injection Nuclear Class I
5. a: Component repaired, modified or replaced 13SI17 Valve  
b: Name of manufacturer Rockwell - Edwards  
c: Identifying nos. N/A N/A N/A N/A 1968  
(mfr's. serial no.) (Nat'l Bd No.) (jurisdictional no.) (other) (year built)  
d: Construction Code WE-Spec 677264 N/A N/A 1  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 677264 (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A
11. Description of work Replaced valve with valve of new design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 921117149 Deficiency Report #: N/A  
Design Change Package #: 1EO-2309 Pkg. 3  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-14 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 19 94 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-26 19 94 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ373  
(Nat'l Bd No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Page 2 of 2Code Job Package No.: S93-183

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System: Safety Injection Nuclear Class: I

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: 13SJ17 ValveName of Manufacturer: Edward Valve Inc.

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
AFX-53	N/A	N/A	N/A	1993	Yes
Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

\*ANSI B31.7 1969 Ed./1970 Add.

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-184  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. a: Component repaired, modified or replaced 14SI17 Valve  
b: Name of manufacturer Rockwell - Edwards  
c: Identifying nos. N/A N/A N/A N/A 1968  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 677264 N/A N/A 1  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 677264 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve with valve of new design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920508129 Deficiency Report #: N/A  
Design Change Package #: 1EO-2309 Pkg. 4  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-26 1994 Signed James E. Gurl Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Page 2 of 2Code Job Package No.: S93-184

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System: Safety Injection Nuclear Class: I

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: 14SJ17 ValveName of Manufacturer: Edward Valve Inc.

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
AFX-52	N/A	N/A	N/A	1993	Yes
Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

\*ANSI B31.7 1969 Ed./1970 Add.

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-190  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Fuel Transfer System Nuclear Class II
5. a: Component repaired, modified or replaced Fuel Transfer System Blind Flange  
b: Name of manufacturer Westinghouse  
c: Identifying nos. N/A N/A N/A N/A 1970  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 677020 N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A N/A N/A  
(edition) (addenda) (Code Case(s))  
\*W E-Spec 677020
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A
11. Description of work Replaced blind flange on fuel transfer canal in kind from folio.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 930503126 Deficiency Report #: N/A  
Design Change Package #: 1EC-3229 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*Type "B" Mechanical Penetration Leak Rate Test

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 12-9 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 12-8, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-10 1993 Signed James E. Losh Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 5894 IWA NJ 373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-192  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced 1PR7 Valve  
b: Name of manufacturer Velan  
c: Identifying nos. PSE-54040 N/A N/A N/A 1967  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 676258 N/A N/A I  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 676258 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced stem.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930708100 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-18 1994 Signed James E. Cook Factory Mutual Engr. Assn.  
(Inspector) (Nat'l.Bd.No.(including endorsements)state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-193  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced 1PR6 Valve  
b: Name of manufacturer Velan  
c: Identifying nos. 6381782 N/A N/A N/A 1967  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 676258 N/A N/A I  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 676258 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replaced stem.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930708098 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-18 1994 Signed [Signature] Factory Mutual Engr. Assn. Commissions NB5894 INA NJ373  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-207  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical Volume Control Nuclear Class II

5. a: Component repaired, modified or replaced 11CV148 Valve Bolting  
b: Name of manufacturer ITT Grinnell  
c: Identifying nos. 085936 N/A N/A N/A 1988  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec E676281 1970 N/A II  
ANSI B31.7 1969 (edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 676281 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced bonnet stud.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 910918004 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

#### CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 9-29 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

#### CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 9-21, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9-24, 1993 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 5894 INA NY 373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☒ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-212  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Steam Generator Feedwater Nuclear Class II

5. a: Component repaired, modified or replaced 14BF22 Valve  
b: Name of manufacturer Rockwell  
c: Identifying nos. PSJC-70 N/A N/A N/A 1969  
(mfr's designation) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
PS Spec (addenda)  
d: Construction Code 71-6164/70-6497 N/A N/A II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*PS Spec 71-6164/70-6497 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Performed weld repairs on valve body.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 921006123 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 2-4 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 2-3, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-1 1994 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ 373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-218  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Residual Heat Removal Nuclear Class I

5. a: Component repaired, modified or replaced 1P-SISN-502-A Snubber  
b: Name of manufacturer Pacific Scientific  
c: Identifying nos. 330 N/A N/A N/A 1976  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code PS Spec 73-6402 N/A N/A I  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1974 S76 N/A  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A

11. Description of work Replaced snubber serial # 330 with snubber serial # 23687.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931102641 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*VT 3 Pre-Service Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 12-6 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-16, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-8 1993 Signed James E. Lork Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒**  
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-220  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. a: Component repaired, modified or replaced 1PS1SN-502 A&B, 1PS1SN503 A&B - Snubbers  
 b: Name of manufacturer Pacific Scientific  
 c: Identifying nos. N/A N/A N/A N/A 1988  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code PS Spec 73-6402 N/A N-71 thru N71-9  
ANSI B31.7 1969 Edition 1970 Add. (addenda) N249 1/2, N247 (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N-71-9  
\*ANSI B31.7/PS Spec 7402 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Replaced transition tubes assemblies.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931006115 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*VT-3 Pre-Service Examination Temp. N/A

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 2-9 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 10-27, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 7-25 1994 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

**FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒**  
**TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS**

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-221  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Demin Water Nuclear Class II

5. a: Component repaired, modified or replaced 1DR30 Check Valve  
 b: Name of manufacturer Velan  
 c: Identifying nos. 0252 N/A N/A N/A 1988  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
 d: Construction Code ASME Sec. III 1971 1971 None 2  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1971 1971 N/A  
\*ASME Sec. III/B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replace bonnet bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931101126 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 10-25 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

**CERTIFICATE OF INSPECTION**

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 10-13, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 10-26, 1993 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-225  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam/Turbine Bypass Nuclear Class II

5. a: Component repaired, modified or replaced Piping 12MS200  
b: Name of manufacturer M.J. Doyle  
c: Identifying nos. N/A N/A N/A N/A 1988  
(mfr's. serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 1970 N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N-416  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N-416  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N-416  
(edition) (addenda) (Code Case(s))  
\*ANSI B31.7

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced flange and spool, elbow and bolting in kind.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 921216155 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Leak Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 3-1, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 2-3, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-2, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 2/14 NJ373  
(Nat'l Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-226  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced Valve 14MS171  
b: Name of manufacturer Fisher  
c: Identifying nos. 5725449 N/A N/A N/A 1975  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME Sec. III 1971 S/1972 N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1971 S/72 N/A  
(edition) (addenda) (Code Case(s))  
\*ASME Sec. III

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A  
Description of work Replaced plug and stem assembly in kind.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931116004 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 19 94 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-19 19 94 Signed [Signature] Commissions NB 5894 IN 4 NJ 373  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-227  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced 1C-PR-SN010 Snubber  
b: Name of manufacturer Pacific Scientific  
c: Identifying nos. 25784 N/A N/A N/A 1976  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME III NF 1974 W76 1644-1,1644-5 1  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1974 S70 1644-4  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Replaced snubber serial #25784 with snubber serial #27164.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931102041 Deficiency Report #: SMD 93-311  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*VT3 Pre-Service Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-6, 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-16, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-14, 1994 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 3894 INA NJ373  
(Nat'l.Bd.No.(including endorsements)state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-228  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Steam Generator Feedwater Nuclear Class I
5. a: Component repaired, modified or replaced 1S-13FW-SN017B Snubber  
b: Name of manufacturer Pacific Scientific  
c: Identifying nos. 445 N/A N/A N/A 1976  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code PS Spec 73-6402 N/A 1644-4 I  
ASME III 74 W75, ANSI B31.7 1969 Ed. 1970 Add. (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1974 S76 1644-4  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A
11. Description of work Replaced snubber serial # 445 with snubber serial # 27629.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 931102041 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*VT3 Preservice Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 12-8 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-16, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-8 1993 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 IN A NJ 373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-229  
(name) (repair organization's P.O. no., job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced 1C-PRA-162 Snubber

b: Name of manufacturer Pacific Scientific

c: Identifying nos. 111 N/A N/A N/A 1968  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code PS Spec 73-6402 N/A 1644-4 1979  
ASME III 1974 W75/ANSI B31.7 1969 Add. 1970 Add. (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1974 W75 1644-4  
(edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Replaced snubber serial #11 with snubber serial # 3903.

(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931102041 Deficiency Report #: N/A

Design Change Package #: N/A Rev. N/A

See attached pages for Description of Replacement. N/A

\*\*VT3 Pre-service Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 12-6 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-16, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-8 1993 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☒ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-233  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 14MS169 Support  
b: Name of manufacturer Fisher Valve - PSE&G Support  
c: Identifying nos. None None N/A N/A 1972  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code PSE&G Spec 69-6445 N/A None II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/PSE&G Spec 69-6445 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) None

11. Description of work Removal of support for valve disassembly and inspection - rewelded  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
of support after inspection and re-assembly of valve.

12. Remarks: Work Order #: 931104008 Deficiency Report #: None  
Design Change Package #: None Rev. None  
See attached pages for Description of Replacement. N/A  
\*\*N/A Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 11-22 19 93 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 10-13, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11/23 19 93 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions P4 2056 NJ BS 13 NJ 43  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-238  
(name) (repair organization's P.O. no., job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.

(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Residual Heat Removal Nuclear Class II

5. a: Component repaired, modified or replaced 12RH18 Valve

b: Name of manufacturer Fisher

c: Identifying nos. 11319 N/A N/A N/A 1988  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code WE-Spec 676368 N/A N/A II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 676368 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced flange to valve bolting.

(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 921005117 Deficiency Report #: N/A

Design Change Package #: N/A Rev. N/A

See attached pages for Description of Replacement. N/A

\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 11-26 1993. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-23, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11-27, 1993. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-240  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Component Cooling Nuclear Class II

5. a: Component repaired, modified or replaced 1CC187 Valve  
b: Name of manufacturer Anchor Darling  
c: Identifying nos. U56669 N/A N/A N/A 1970  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 676258 N/A N/A 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec 676258 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replace valve bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930628060 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 12-13 19 93 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 12-7, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-14 19 93 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions P42056 NIBSIS NJ 431  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-241  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class II

5. a: Component repaired, modified or replaced 1WR81  
b: Name of manufacturer Aloyco Div./Calworth  
c: Identifying nos. 70/A627-3 N/A N/A N/A 1987  
(mfr's. serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANST B31.7 1969 N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 N/A N/A  
(edition) (addenda) (Code Case(s))  
\*ANSI B31.7

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Replaced bolting and nuts in kind.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931115009 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*Inservice Leak Exam Temp. NOT/NOP

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-18 1994 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 IWA NJ373  
(Nat'l Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-249  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Residual Heat Removal Nuclear Class I

5. a: Component repaired, modified or replaced 14RH27  
b: Name of manufacturer Anchor Darling Valve  
c: Identifying nos. None Available N/A N/A N/A 1968  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec 676241 N/A N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A N/A N/A  
(edition) (addenda) (Code Case(s))  
\*W E-Spec 676241

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Replace (16) studs in kind from folio.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 940326008 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-14 19 94. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-18, 19 94. Signed James E. Cook Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-250  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Safety Injection Nuclear Class I

5. a: Component repaired, modified or replaced 12SJ55 Valve  
b: Name of manufacturer Anchor Darling  
c: Identifying nos. 11542-10-61 N/A N/A N/A 1971  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec G676241 N/A N/A I  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec G676241 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931002074 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-14, 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-18, 1994 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 5894 INA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-254  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced Pressurizer - Manway Cover - 1RCE10  
b: Name of manufacturer Westinghouse  
c: Identifying nos. 1011 68-7 N/A N/A 1970  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME III 1965 W1965 N/A 1  
(edition) (addenda) (Code Case(s)) (Code Class)  
ANSI B31.7 1969 Ed. 1970 Add.

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*See Attachment  
(edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replaced pressurizer manway cover and bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930401183 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-14 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co. of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24 1994 Signed James E. Cash Factory Mutual Engr. Assn.  
(Inspector) (Natl. Bd. No. (including endorsements) state or province and number) NB5894 INA NJ373

## FORM NR-1 ATTACHMENT

Page 2 of 2Code Job Package No.: S93-254

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
4. System: Reactor Coolant Nuclear Class: 1

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: Pressurizer Manway CoverName of Manufacturer: Westinghouse

C42744 N/A N/A N/A 1987 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1983 Edition S83 Addenda, Code Class 1  
Code Cases: N/A  
ANSI B31.7 1969 Ed. 1970 Add.

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-260  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class II

5. a: Component repaired, modified or replaced Pressurizer Relief Tank-Flanges-1RCE11  
b: Name of manufacturer Westinghouse  
c: Identifying nos. L-10662 61420 N/A N/A 1969  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME III 1968 N/A N/A (C) 2  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1989 89 N/A  
(edition) (addenda) (Code Case(s))  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add.

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replace pressurizer relief tank rupture disc flanges with upgraded  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
material.

12. Remarks: Work Order #: 930107096 Deficiency Report #: N/A  
Design Change Package #: 1EZ-4770 Rev. 0  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-14, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-17, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB3894 IMA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-262  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced #14 Steam Generator  
b: Name of manufacturer Westinghouse  
c: Identifying nos. 1203 68-51 N/A N/A 1971  
(mfr's. serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME Sec. III 1965 S/1966 N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1965 S66 N/A  
(edition) (addenda) (Code Case(s))  
\*ASME Sec. III

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*Pressure N/A psi Code Case(s) N/A

11. Description of work Replace (1) bolt in kind from folio on #14 Steam Generator  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
Primary Manway Cold Leg.

12. Remarks: Work Order #: 931105033 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test @ NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 19 94

Date 1-14 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 19 94 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-28 19 94 Signed James E. Lord Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ373  
(Nat'l Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-266  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class II

5. a: Component repaired, modified or replaced 1WR81 - Valve  
b: Name of manufacturer Aloyco  
c: Identifying nos. 70/A627-3 N/A N/A N/A 1970  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec G-676241 Rev. 1 N/A II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec G-67624 Rev. 1 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*pressure NOP psi Code Case(s) N/A

11. Description of work Replace valve disc.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931115185 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-21 1994 Signed James E. Cook Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-270  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 12MS15 Valve  
b: Name of manufacturer Crosby  
c: Identifying nos. N55100-03-0045 N/A N/A N/A 1989  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME III 1968 W 1977 1440 1  
(edition) (addenda) (Code Case(s)) (Code Class)  
ANSI B31.7 1969 Ed. 1970 Add.

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1968 W70 1440  
(edition) (addenda) (Code Case(s))  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add.

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve with spare valve from stores.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931116130 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-14, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-17, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) (Natl. Bd. No. (including endorsements) state or province and number)  
NB 3894 INIA NJ 373

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CTP #S93-276  
(name) (repair organization's P.O. no., job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.

(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Control Air Nuclear Class II

5. a: Component repaired, modified or replaced 11CA360

b: Name of manufacturer Rockwell International

c: Identifying nos. Not Available N/A N/A N/A 1987  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code ASME Sec. III 1977 1979 N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1977 W79 N/A  
(edition) (addenda) (Code Case(s))

**\*ASME Sec. III**

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Replace (4) cap screws and valve disc in kind from folio.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931004200 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOP/NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 12-8 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 12-3, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-17 1993 Signed James E. Cook Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-277  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical & Volume Control (CVC) Nuclear Class II

5. a: Component repaired, modified or replaced 1CVE17 #1 Let Down Heat Exchanger  
b: Name of manufacturer Atlas  
c: Identifying nos. 843 694 N/A N/A 1967  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME Sect. VIII 1967 N/A II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1967 N/A N/A  
\*ASME Sec. VIII/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replaced bolting.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930924134 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-14, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-21, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NBS894 INA NJ373  
(Nat'l.Bd.No.(including endorsements)state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-278  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Reactor Coolant Nuclear Class II
5. a: Component repaired, modified or replaced 1RCE35 El. 100 Personnel Hatch  
b: Name of manufacturer Chicago Budge and Iron  
c: Identifying nos. 2668 N/A N/A N/A 1970  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME III 1965 W67 N/A (B) 2  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1965 W67 (B) 2  
(edition) (addenda) (Code Case(s))  
\*ASME III
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A
11. Description of work Install new stop pin for air lock equalizing ball valve.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 930422060 Deficiency Report #: 93-475  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*Passed Leak Rate Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 12-22 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-29, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-22 1993 Signed James E. Cook Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-279  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Service Water Nuclear Class II

5. a: Component repaired, modified or replaced Butterfly Valve 15SW72  
b: Name of manufacturer Masoneilan International Inc.  
c: Identifying nos. N-00113-39-5 N/A N/A N/A 1977  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME Draft P&V 1968 M/70 N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1968 M/70 N/A  
(edition) (addenda) (Code Case(s))  
\*ASME Draft P&V

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work The valve was replaced in kind (S/N N-00203-1-8).  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 9310202208 Deficiency Report #: N/A  
Design Change Package #: N/A Rev. N/A  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NO NCT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 3-1, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-1, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NBS894 INA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #S93-287  
(name) (repair organization's P.O. no., job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.

(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant

Nuclear Class I

5. a: Component repaired, modified or replaced 1C-PR-SN010 Snubber

b: Name of manufacturer Pacific Scientific

c: Identifying nos. 25784 N/A N/A N/A 1976  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code ASME III NF 1974 W76 1644-1, 1644-5 1  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1974 S70 1644-4  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A

11. Description of work Replaced snubber serial # 25784 with snubber serial # 27164.

(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931102041 Deficiency Report #: SMD 93-311

Design Change Package #: N/A Rev. N/A

See attached pages for Description of Replacement. N/A

\*\*VT3 Pre-Service Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 12-6 1993 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-16, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-8, 1993 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 1894 INH NJ 377  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS92-040  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical Volume Control Nuclear Class I

5. a: Component repaired, modified or replaced 1C-CVCS-276  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1975  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code Spec. 69-6445, Rev. 9 N/A N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A N/A N/A  
(edition) (addenda) (Code Case(s))  
\*S-C-MBOO-MDS-043 Rev. 0

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A

11. Description of work Modification of existing support.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920805116 Deficiency Report #: N/A  
Design Change Package #: 1EC-3167 Rev. 1  
See attached pages for Description of Replacement.  
\*\*N/A Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 12-7 19 93 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-18, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-17 19 93 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Page 2 of 2Code Job Package No.: PS92040

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
System: Chemical Volume Control Nuclear Class: I

ID of Component Replaced: 1C-CVCH-326  
Name of Manufacturer: PSE&G  
N/A N/A N/A N/A 1976 N/A  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: Spec.69-6445 Rev.9 Edition N/A Addenda, Code Class I  
Code Cases: N/A

ID of Replacement: 1C-CVCH-326  
Name of Manufacturer: PSE&G  
N/A N/A N/A N/A 1993 N/A  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: S-C-MBOO-MDS-043 Edition N/A Addenda, Code Class I  
Code Cases: N/A Rev. 0

ID of Component Replaced: 1A-CVCG-902  
Name of Manufacturer: PSE&G  
N/A N/A N/A N/A 1972 N/A  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: Spec.69-6445 Rev.9 Edition N/A Addenda, Code Class II  
Code Cases: N/A

ID of Replacement: 1A-CVCG-902  
Name of Manufacturer: PSE&G  
N/A N/A N/A N/A 1993 N/A  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: S-C-MBOO-MDS-043 Edition N/A Addenda, Code Class II  
Code Cases: N/A Rev. 0

ID of Component Replaced: \_\_\_\_\_  
Name of Manufacturer: \_\_\_\_\_  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_  
Name of Manufacturer: \_\_\_\_\_  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CTP #PS92-041  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced IC-PRH-177  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1974  
(mfr's. serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code Spec 69-6430 Rev. 5 N/A N/A I  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A N/A N/A  
(edition) (addenda) (Code Case(s))  
\*S-C-MBOO-MDS-043

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A

11. Description of work Modification of pipe support.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 920805117 Deficiency Report #: N/A  
Design Change Package #: 1EC-3168 Rev. 0  
See attached pages for Description of Replacement.  
\*\*N/A Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 12-3 19 93 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11/18, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12/14, 19 93 Signed [Signature] Commissions Factory Mutual Engr. Assn. P4 2056 NIBS/JS WT 43/  
(Inspector) (Nat'l Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: PS92041

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
4. System: Chemical Volume Control Nuclear Class: II

ID of Component Replaced: 1-CVCA-653Name of Manufacturer: PSE&G

N/A N/A N/A N/A 1980 N/A  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.? ASME Code Stamp'd.?  
Original Const. Code: Spec 69-6445 Rev9 Edition N/A Addenda, Code Class II  
Code Cases: N/A

ID of Replacement: 1-CVCA-653Name of Manufacturer: PSE&G

N/A N/A N/A N/A 1993 N/A  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.? ASME Code Stamp'd.?  
Replacement Const. Code: Spec S-C-MB00-MDS Edition N/A Addenda, Code Class II  
Code Cases: N/A -043

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Cases: \_\_\_\_\_

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-012  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Control Air Nuclear Class II
5. a: Component repaired, modified or replaced 11CA330 and 123CA330 Valves  
b: Name of manufacturer Masoneilan  
c: Identifying nos. N-00133-1-1&2 N/A N/A N/A 1992  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ASME III 1971 S/1971 N/A II  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1971 S71 N/A  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A
11. Description of work Replaced stem assembly and bolting on 11CA330. Replace stem assembly  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
on 12CA330. Stem assemblies install to match new soft seats.
12. Remarks: Work Order #: 920421124 Deficiency Report #: N/A  
Design Change Package #: 1EC-3170 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 12-22, 1993. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-24, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 12-27, 1993. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions PA 205621 BS 15 NS 431  
(Nat'l.Bd.No.(including endorsements)state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-045  
(name) (repair organization's P.O. no., job no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Generator Blowdown Nuclear Class II

5. a: Component repaired, modified or replaced 11GB4, 12GB4, 13GB4, 14GB4 Valve Internals  
b: Name of manufacturer Masoneillan  
c: Identifying nos. See Attch. N/A N/A N/A 1974  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 None 2  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A 1969 N/A  
\*ANSI B31.7/ASME Sec. III 1971 - Summer 71 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ pressure \*\* psi Code Case(s) N/A

11. Description of work Replace GB4 valve internals IAW DCP 1EC-3226-01.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930203123 Deficiency Report #: None  
Design Change Package #: 1EC-3226 Rev. 01  
See attached pages for Description of Replacement.  
\*\* NOP Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 2-23 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 2-22, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 2-28 19 94 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ373  
(Nat'l.Bd.No.(including endorsements)state or province and number)

## FORM NR-1 ATTACHMENT

Page 2 of 2Code Job Package No.: PS93045

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
System: Generator Blowdown Nuclear Class: 2

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ID of Component Replaced: 11GB4 Valve Internals  
Name of Manufacturer: Masoneillan  
N2/N001301-1 N/A N/A N/A 1974  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: ANSI B31.7 Edition 1969 Addenda, Code Class 2  
Code Cases: N/A ASME Sec. III 1971 - Summer 71

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ID of Replacement: 12GB4 Valve Internals  
Name of Manufacturer: Masoneillan  
N2/N001301-2 N/A N/A N/A 1974  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: ANSI B31.7 Edition 1969 Addenda, Code Class 2  
Code Cases: N/A ASME Sec. III 1971 - Summer 71

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ID of Component Replaced: 11GB4 Valve Internals  
Name of Manufacturer: Masoneillan  
N2/N001301-3 N/A N/A N/A 1974  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: ANSI B31.7 Edition 1969 Addenda, Code Class 2  
Code Cases: N/A ASME Sec. III 1971 - Summer 71

=====

ID of Replacement: 12GB4 Valve Internals  
Name of Manufacturer: Masoneillan  
N2/N001301-4 N/A N/A N/A 1974  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: ANSI B31.7 Edition 1969 Addenda, Code Class 2  
Code Cases: N/A ASME Sec. III 1971 - Summer 71

=====

ID of Component Replaced: \_\_\_\_\_  
Name of Manufacturer: \_\_\_\_\_  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

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ID of Replacement: \_\_\_\_\_  
Name of Manufacturer: \_\_\_\_\_  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

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FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-053  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Reactor Coolant Nuclear Class I
5. a: Component repaired, modified or replaced 1PRI Valve  
b: Name of manufacturer Copes Vulcan  
c: Identifying nos. 6910-95001-164-1 N/A N/A 1969  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec E676270 N/A N/A I  
ANSI B31.7 1969 Ed. 1970 Add. (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E-Spec E-676270 (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A
11. Description of work Replaced valve bolting and valve internal with materials of new design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 930126150 Deficiency Report #: N/A  
Design Change Package #: 1EC-3219 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-17 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-19 1994 Signed James E. Gohl Factory Mutual Engr. Assn.  
(Inspector) (Natl. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-054  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced 1PR2 Valve  
b: Name of manufacturer Copes Vulcan  
c: Identifying nos. 6910-95001-164-2 N/A N/A N/A 1969  
(mfr's. serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code W E-Spec E676270 N/A N/A I  
ANSI B31.7 1969 Edition 1970 Add. (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/W E Spec E676270 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Replaced valve bolting and valve internal with materials of new design.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930126150 Deficiency Report #: N/A  
Design Change Package #: 1EC-3219 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test NOP Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-17 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-19 1994 Signed [Signature] Factory Mutual Engr. Assn. Commissions NB 5544 INA NJ 373  
(Inspector) (Nat'l Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-052  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class II

5. a: Component repaired, modified or replaced 1C-PRIS-404 Support  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1976  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code PS Spec 69-6445 N/A N/A 2  
SGS/M-FD-1 (edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A N/A N/A  
\*PS Spec S-C-MBOO-MDS-043 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Modified pressurizer piping supports.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930126150 Deficiency Report #: N/A  
Design Change Package #: 1EC-3219 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*No Pre-Service Exam Required Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 12-8 19 93 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 20 DEC, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 20 DEC, 19 93 Signed [Signature] Factory Mutual Engr. Assn. Commissions NB11207 INA NJ1040  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. GTP #PS93-055  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Reactor Coolant Nuclear Class I

5. a: Component repaired, modified or replaced PRT Pipe Spools S1-RC-3220 and S1-RC-3222  
b: Name of manufacturer UE&C/PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1974  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 1970 N/A I  
ANSI B31.1 1967 (edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/ANSI B31.1 1967 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure NOP psi Code Case(s) N/A

11. Description of work Modified pressurizer relief tank small bore piping.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930126150 Deficiency Report #: N/A  
Design Change Package #: 1EC-3219 Pkg. 1  
See attached pages for Description of Replacement. N/A  
\*\*System Inservice Test Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-17 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-11, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-21 19 94 Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INH NJ 373  
(Nat'l.Bd.No.(including endorsements)state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-070 Rev.1  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Control Air Nuclear Class II

5. a: Component repaired, modified or replaced Pipe Spool 1CA2310  
b: Name of manufacturer M.J. Doyle  
c: Identifying nos. N/A N/A N/A N/A 1974  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code B31.1 1967/B31.7 1969 None None II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1967 N/A None  
(edition) (addenda) (Code Case(s))  
\*ANSI B31.1/B31.7 1969

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☒ design pressure ☐ \*\*pressure 142 psi Code Case(s) None

11. Description of work Added break flanges to pipe spool piece 1CA2310.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930615135 Deficiency Report #: None  
Design Change Package #: 1EC-3150-1 Rev. 0  
See attached pages for Description of Replacement. N/A  
\*\* Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 2-25 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-24, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 2-25 19 94 Signed James E. Carl Factory Mutual Engr. Assn.  
(Inspector) (Nat'l.Bd.No.(including endorsements)state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-073  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical & Volume Control System Nuclear Class II

5. a: Component repaired, modified or replaced Boric Acid Blender  
b: Name of manufacturer Westinghouse  
c: Identifying nos. None N/A N/A N/A 1972  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 1970 None II  
ANSI B31.1 1967 (edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/ANSI B31.1 1967 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☒ pneumatic ☐ design pressure ☐ \*\* pressure 170 psi Code Case(s) N/A  
Description of work Removal of boric acid blender no longer needed in chemical and  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
volume control system and re-installation of piping.

12. Remarks: Work Order #: 930618133 Deficiency Report #: N/A  
Design Change Package #: 1EC-3254 Rev. 0  
See attached pages for Description of Replacement. N/A  
\*\*N/A Temp. 90°F

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-14 1993 Signed PSE&G Co.-Nuclear Dept. S. Schutler Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-2, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-25, 1994 Signed James E. Gosh Commissions NB5894 INA NJ373  
(Inspector) (Factory Mutual Engr. Assn.)  
(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-074  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Chemical & Volume Control System Nuclear Class II

5. a: Component repaired, modified or replaced 1CV172 Control Valve  
b: Name of manufacturer Copes Vulcan  
c: Identifying nos. 6910-95001-187-1 N/A N/A N/A 1971  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code ANSI B31.7 1969 1970 N/A II  
W E-Spec 676270 (edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 None  
\*ANSI B31.7/W E-Spec 676270 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☒ pneumatic ☐ design pressure ☐ \*\*pressure 170 psi Code Case(s) N/A

11. Description of work Replace existing 1CV172 valve internals in accordance with DCP 1EC-3254  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)  
Package 1 and replacement of bolting and nuts per revision 1 of DCP 1EC-3254.

12. Remarks: Work Order #: 930618133 Deficiency Report #: None  
Design Change Package #: 1EC-3254 Rev. 0  
See attached pages for Description of Replacement.  
\*\*N/A Temp. 90°F

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-13, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-2, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-25, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-075  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 1MSE57, 1MSE58, 1MSE59, 1MSE60 Condensate Chambers  
b: Name of manufacturer Connex Pipe Systems  
c: Identifying nos. See Attachment N/A N/A N/A 1993  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code See Attachment N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1989 1989 N/A  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☒ pneumatic ☐ design pressure ☐ \*\*pressure 1400psi Code Case(s) N/A

11. Description of work Installed new condensate chambers for #11 main steam header.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930810099 Deficiency Report #: N/A  
Design Change Package #: 1EC-3256 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. N/A / NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-24 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-19, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24 19 94 Signed James E. Lark Factory Mutual Engr. Assn.  
(Inspector) (Nat'l. Bd. No. (including endorsements) state or province and number)

Code Job Package No.: PS93075

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
4. System: Main Steam Nuclear Class: II

=====

ID of Component Replacement: 1MSE57 Condensate Chamber  
Name of Manufacturer: Connex Pipe System  
43471 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

=====

ID of Replacement: 1MSE58 Condensate Chamber  
Name of Manufacturer: Connex Pipe System  
43472 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

=====

ID of Component Replaced: N/A  
Name of Manufacturer: \_\_\_\_\_  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

=====

ID of Replacement: 1MSE59 Condensate Chamber  
Name of Manufacturer: Connex Pipe System  
43473 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

=====

ID of Component Replaced: N/A  
Name of Manufacturer: \_\_\_\_\_  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

=====

ID of Replacement: 1MSE60 Condensate Chamber  
Name of Manufacturer: Connex Pipe System  
43474 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

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FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-076  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 1MSE61, 1MSE62, 1MSE63, 1MSE64 Condensate Chambers  
b: Name of manufacturer Connex Pipe Systems  
c: Identifying nos. See Attachment N/A N/A N/A 1993  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code See Attachment N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1989 1989 N/A  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☒ pneumatic ☐ design pressure ☐ \*\*pressure 1400psi Code Case(s) N/A

11. Description of work Installed new condensate chambers for #12 main steam header.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930810099 Deficiency Report #: N/A  
Design Change Package #: 1EC-3256 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. N/A / NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-24, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-19, 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 5894 INA W5373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: PS93076

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System: Main Steam Nuclear Class: II

=====

ID of Component Replacement: 1MSE61 Condensate Chamber

Name of Manufacturer: Connex Pipe System

43471 N/A N/A N/A 1993 No

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

=====

ID of Replacement: 1MSE62 Condensate Chamber

Name of Manufacturer: Connex Pipe System

43472 N/A N/A N/A 1993 No

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

=====

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

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ID of Replacement: 1MSE63 Condensate Chamber

Name of Manufacturer: Connex Pipe System

43473 N/A N/A N/A 1993 No

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

=====

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_

Code Cases: \_\_\_\_\_

=====

ID of Replacement: 1MSE64 Condensate Chamber

Name of Manufacturer: Connex Pipe System

43474 N/A N/A N/A 1993 No

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?

Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2

Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

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FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-077  
(name) (repair organization's P.O. no., job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 1MSE65, 1MSE66, 1MSE67, 1MSE68 Condensate Chambers

b: Name of manufacturer Connex Pipe Systems

c: Identifying nos. See Attachment N/A N/A N/A 1993  
(mfr's. serial no.) (Nat'l Bd. No.) (jurisdictional no.) (other) (year built)

d: Construction Code See Attachment N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1989 1989 N/A  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☒ pneumatic ☐ design pressure ☐ \*\* pressure 1400 psi Code Case(s) N/A

11. Description of work Installed new condensate chambers for #13 main steam header.

(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930810099 Deficiency Report #: N/A  
Design Change Package #: 1EC-3256 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. N/A / NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-24 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-19, 19 94 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24 19 94 Signed James E. Corl Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA N5373  
(Nat'l Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: PS93077

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
2. Owner: Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Public Service Electric & Gas Co.  
3. Location: Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Salem Nuclear Generating Station  
4. System: Main Steam Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Nuclear Class: II

ID of Component Replacement: 1MSE65 Condensate ChamberName of Manufacturer: Connex Pipe System

43471 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

ID of Replacement: 1MSE66 Condensate ChamberName of Manufacturer: Connex Pipe System

43472 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 1MSE67 Condensate ChamberName of Manufacturer: Connex Pipe System

43473 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 1MSE68 Condensate ChamberName of Manufacturer: Connex Pipe System

43474 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-078  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 1MSE69, 1MSE70, 1MSE71, 1MSE72 Condensate Chambers  
b: Name of manufacturer Connex Pipe Systems  
c: Identifying nos. See Attachment N/A N/A N/A 1993  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code See Attachment N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1989 1989 N/A  
\*ASME III/ANSI B31.7 1969 Ed. 1970 Add. (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☒ pneumatic ☐ design pressure ☐ \*\* pressure 1400 psi Code Case(s) N/A  
Description of work Installed new condensate chambers for #14 main steam header.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930810099 Deficiency Report #: N/A  
Design Change Package #: 1EC-3256 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*System Inservice Test Temp. N/A / NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-24 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 1-19 1994 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24 1994 Signed James E. Carl Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 I NA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: PS93078

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
4. System: Main Steam Nuclear Class: II

ID of Component Replacement: 1MSE69 Condensate ChamberName of Manufacturer: Connex Pipe System

43471 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

ID of Replacement: 1MSE70 Condensate ChamberName of Manufacturer: Connex Pipe System

43472 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 1MSE71 Condensate ChamberName of Manufacturer: Connex Pipe System

43473 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 1MSE72 Condensate ChamberName of Manufacturer: Connex Pipe System

43474 N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ASME III 1989 Edition 1989 Addenda, Code Class 2  
Code Cases: N/A ANSI B31.7 1969 Ed. 1970 Add

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-079  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Main Steam Nuclear Class II
5. a: Component repaired, modified or replaced 1C-MSG-2532A&B, 1C-MSG-2533A&B Supports  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1993  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code See Attachment N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/PS Spec S-C-MBOO-MDS-043, 1985 (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A
11. Description of work Installed supports for condensate pots on #11 main steam header.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 930810091 Deficiency Report #: N/A  
Design Change Package #: 1EC-3256 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*No Preservice Required Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-3 1994 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-11, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24 1994 Signed James E. Good Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA 10373  
(Nat'l.Bd.No.(including endorsements)state or province and number)

Code Job Package No.: PS93079

Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
System: Main Steam Nuclear Class: II

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: 1C-MSG-2532 A&B SupportsName of Manufacturer: PSE&G

N/A	N/A	N/A	N/A	1993	No
Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: 1C-MSG-2533 A&B SupportsName of Manufacturer: PSE&G

N/A	N/A	N/A	N/A	1993	No
Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:				Edition	Addenda, Code Class
Code Cases:					

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:				Edition	Addenda, Code Class
Code Cases:					

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-080  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 1C-MSG-2534A&B, 1C-MSG-2533A&B Supports  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1993  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code See Attachment N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/PS Spec S-C-MBOO-MDS-043, 1985 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\*pressure N/A psi Code Case(s) N/A

11. Description of work Installed supports for condensate pots on #12 main steam header.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930810097 Deficiency Report #: N/A  
Design Change Package #: 1EC-3256 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*No Preservice Required Temp. NOT

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-3 19, 94. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-11, 19 93 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24, 19 94. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

Code Job Package No.: PS93080

1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System: Main Steam Nuclear Class: II

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:			Edition		Addenda, Code Class
Code Cases:					

ID of Replacement: 1C-MSG-2534 A&B SupportsName of Manufacturer: PSE&G

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
N/A	N/A	N/A	N/A	1993	No
Replacement Const. Code:			Edition		Addenda, Code Class
Code Cases:					

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:			Edition		Addenda, Code Class
Code Cases:					

ID of Replacement: 1C-MSG-2535 A&B SupportsName of Manufacturer: PSE&G

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
N/A	N/A	N/A	N/A	1993	No
Replacement Const. Code:			Edition		Addenda, Code Class
Code Cases:					

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Original Const. Code:			Edition		Addenda, Code Class
Code Cases:					

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No.	Nat'l. Bd.	Jurist No.	Other	Year Built	ASME Code Stamp'd.?
Replacement Const. Code:			Edition		Addenda, Code Class
Code Cases:					

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS Page 1 of 2

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-081  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Main Steam Nuclear Class II

5. a: Component repaired, modified or replaced 1C-MSG-2562A&B, 1C-MSG-2563A&B Supports  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1993  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code See Attachment N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/PS Spec S-C-MBOO-MDS-043, 1985 (edition) (addenda) (Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A

11. Description of work Installed supports for condensate pots on #13 main steam header.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930810099 Deficiency Report #: N/A  
Design Change Package #: 1EC-3256 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*No Preservice Required Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 1-3, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-11, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB5894 INA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Code Job Package No.: PS93081

1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System: Main Steam Nuclear Class: II

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 1C-MSG-2562 A&B SupportsName of Manufacturer: PSE&G

N/A N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ANSI B31.7 1969 Edition 1970 Addenda, Code Class II  
Code Cases: N/A PS Spec S-C-MBOO-MDS-043 1985

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 1C-MSG-2563 A&B SupportsName of Manufacturer: PSE&G

N/A N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ANSI B31.7 1969 Edition 1970 Addenda, Code Class II  
Code Cases: N/A PS Spec S-C-MBOO-MDS-043 1985

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 2

- Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-082  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)
3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System Main Steam Nuclear Class II
5. a: Component repaired, modified or replaced 1C-MSG-2564A&B, 1C-MSG-2565A&B Supports  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1993  
(mfr's. serial no.) (Nat'l.Bd.No.) (jurisdictional no.) (other) (year built)  
d: Construction Code See Attachment N/A N/A II  
(edition) (addenda) (Code Case(s)) (Code Class)
6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))
7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))
8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*1969 1970 N/A  
\*ANSI B31.7/PS Spec S-C-MBOO-MDS-043, 1985 (edition) (addenda) (Code Case(s))
9. Design responsibilities PSE&G
10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) N/A
11. Description of work Installed supports for condensate pots on #14 main steam header.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)
12. Remarks: Work Order #: 930816100 Deficiency Report #: N/A  
Design Change Package #: 1EC-3256 Pkg. 1  
See attached pages for Description of Replacement.  
\*\*No Preservice Required Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994.

Date 1-3, 1994. Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 11-11, 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24, 1994. Signed [Signature] Factory Mutual Engr. Assn.  
(Inspector) Commissions NB 5894 INA NJ 323  
(Nat'l.Bd.No.(including endorsements) state or province and number)

## FORM NR-1 ATTACHMENT

Page 2 of 2Code Job Package No.: PS93082

1. Work performed by: Public Service Electric & Gas Co., Nuclear Dept.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
2. Owner: Public Service Electric & Gas Co.  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
3. Location: Salem Nuclear Generating Station  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038
4. System: Main Steam Nuclear Class: II

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 1C-MSG-2564 A&B SupportsName of Manufacturer: PSE&G

N/A N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ANSI B31.7 1969 Edition 1970 Addenda, Code Class II  
Code Cases: N/A PS Spec S-C-MBOO-MDS-043 1985

ID of Component Replaced: N/A

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: 1C-MSG-2565 A&B SupportsName of Manufacturer: PSE&G

N/A N/A N/A N/A 1993 No  
Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: ANSI B31.7 1969 Edition 1970 Addenda, Code Class II  
Code Cases: N/A PS Spec S-C-MBOO-MDS-043 1985

ID of Component Replaced: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Original Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

ID of Replacement: \_\_\_\_\_

Name of Manufacturer: \_\_\_\_\_

Serial No. Nat'l. Bd. Jurist No. Other Year Built ASME Code Stamp'd.?  
Replacement Const. Code: \_\_\_\_\_ Edition \_\_\_\_\_ Addenda, Code Class \_\_\_\_\_  
Code Cases: \_\_\_\_\_

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☒ OR INSTALLATION OF REPLACEMENT(S) ☐  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-083 Rev.0

(name)

(repair organization's P.O. no., job. no., etc.)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

(address)

2. Owner Public Service Electric and Gas Co.

(name)

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1

Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Pressurizer Relief Tank Line Support Nuclear Class II

5. a: Component repaired, modified or replaced Pipe Support #C-RVVS-5

b: Name of manufacturer Installed by PSE&G

c: Identifying nos. N/A N/A N/A N/A Installed 1975

(mfr's. serial no.)

(Nat'l. Bd. No.)

(jurisdictional no.)

(other)

(year built)

d: Construction Code Specification 69-6445-1974

None

II

(edition)

(addenda)

(Code Case(s))

(Code Class)

6. Section XI

1983

Summer 83

N/A

(edition)

(addenda)

(Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A

(edition)

(addenda)

(Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*See Below None

\*Spec 696445 1974

(edition)

(addenda)

(Code Case(s))

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) None

11. Description of work Modify pipe support C-RVVS-5

(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 931007147 Deficiency Report #: None

Design Change Package #: 1EC-3054-2 Rev. 0

See attached pages for Description of Replacement. N/A

\*\*N/A

Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 3-25 1994 Signed PSE&G Co.-Nuclear Dept.

(repair organization)

[Signature]

(authorized representative)

Principal Engr.

(title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 12-28 1993 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-25 1994 Signed [Signature]

(Inspector)

Commissions Factory Mutual Engr. Assn.  
NB5894 INA NJ 373

(Nat'l. Bd. No. (including endorsements) state or province and number)

FORM NR-1 REPORT OF REPAIR ☐ MODIFICATION ☐ OR INSTALLATION OF REPLACEMENT(S) ☒  
TO NUCLEAR COMPONENTS AND SYSTEMS IN NUCLEAR POWER PLANTS

Page 1 of 1

Work performed by Public Service Electric & Gas Co. - Nuclear Dept. CJP #PS93-088 Rev.0  
(name) (repair organization's P.O. no., job. no., etc.)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

2. Owner Public Service Electric and Gas Co.  
(name)  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038  
(address)

3. Name, address and identification of nuclear power plant Salem Nuclear Generating Station Unit #1  
Foot of Buttonwood Road, Hancocks Bridge, NJ 08038

4. System Control Air Nuclear Class II

5. a: Component repaired, modified or replaced Pipe Support P-CTAG-660  
b: Name of manufacturer PSE&G  
c: Identifying nos. N/A N/A N/A N/A 1974  
(mfr's. serial no.) (Nat'l. Bd. No.) (jurisdictional no.) (other) (year built)  
d: Construction Code 69-6445/SG-SMFD-1 N/A None II  
(edition) (addenda) (Code Case(s)) (Code Class)

6. Section XI 1983 Summer 83 N/A  
(edition) (addenda) (Code Case(s))

7. Applicable edition of ASME Code Section XI under which repairs, modifications, or replacements were made: 1983 S83 N/A  
(edition) (addenda) (Code Case(s))

8. Applicable edition of Construction Code under which repairs, modifications, or replacements were made: \*N/A N/A None  
(edition) (addenda) (Code Case(s))  
\*S-C-MBOO-MDS-043

9. Design responsibilities PSE&G

10. Tests conducted: hydrostatic ☐ pneumatic ☐ design pressure ☐ \*\* pressure N/A psi Code Case(s) None

11. Description of work Replace 2 nuts on pipe support #P-CTAG-660.  
(use of additional sheet(s) or sketch(es) is acceptable if properly identified)

12. Remarks: Work Order #: 930615135 Deficiency Report #: None  
Design Change Package #: 1EC-3150-1 Rev. 0  
See attached pages for Description of Replacement. N/A  
\*\* Temp. N/A

CERTIFICATE OF COMPLIANCE

We certify that the statements made in this report are correct and the repair, modification or replacement of the items described above conforms to Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition.

Certificate of Authorization no. 36 to use the "NR" stamp expires Oct. 3, 1994

Date 2-25 19 94 Signed PSE&G Co.-Nuclear Dept. [Signature] Principal Engr.  
(repair organization) (authorized representative) (title)

CERTIFICATE OF INSPECTION

I, the undersigned, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors, and certificate of competency issued by the state or province of New Jersey and employed by Arkwright Mutual Insurance Co of MA have inspected the repair, modification or replacement described in this report on 2-25, 19 94 and state that to the best of my knowledge and belief, this repair, modification or replacement has been made in accordance with Section XI of the ASME Code and the National Board rules as defined in the publication NB-102, current edition. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the Inspector nor his employer shall be held in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 2-25 19 94 Signed James E. Cook Factory Mutual Engr. Assn.  
(Inspector) Commissions NBS894 IWA NJ373  
(Nat'l. Bd. No. (including endorsements) state or province and number)