PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Salem Generating Station

February 14, 1994

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of January 1994 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Safety Related Maintenance
10CFR50.59 Evaluations
Operating Summary
Refueling Information

Sincerely yours,

General Manager -
Salem Operations

RH:pc

cc: Mr. Thomas T. Martin
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19046

Enclosures

8-1-7.R4

The Energy People

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272
Unit Name: Salem #1
Date: 02/10/94
Telephone: 339-2122

Completed by: Mike Morroni

Month January 1994

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

OPERATING DATA REPORT

Docket No: 50-272
Date: 02/10/94
Completed by: Mike Morroni
Telephone: 339-2122

Operating Status

			Notes
1.	Unit Name	Salem No. 1	
2.	Reporting Period	January 1994	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity(Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
12. Hours in Reporting Period	744	744	145441
12. No. of Hrs. Rx. was Critical	118.2	118.2	95250.17
13. Reactor Reserve Shutdown Hrs.	0	0	0
14. Hours Generator On-Line	0	0	91887.84
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	6895.2	6895.2	290779209.2
17. Gross Elec. Energy Generated (MWH)	0	0	96535970
18. Net Elec. Energy Gen. (MWH)	-17717	-17717	91919836
19. Unit Service Factor	0	0	63.2
20. Unit Availability Factor	0	0	63.2
21. Unit Capacity Factor (using MDC Net)	0	0	57.1
22. Unit Capacity Factor (using DER Net)	0	0	56.7
23. Unit Forced Outage Rate	0	0	21.0

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

None.

25. If shutdown at end of Report Period, Estimated Date of Startup:

Unit returned to service on 2/2/94.

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH JANUARY 1994

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: 02-08-94
COMPLETED BY: Mike Morroni
TELEPHONE: 339-2122

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
0135	01/01/94	S	744	C	4	----------	RC	FUELXX	NUCLEAR NORMAL REFUELING

1
F: Forced
S: Scheduled

2
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Requlatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

SAFETY RELATED MAINTENANCE
MONTH: - JANUARY 1994

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: FEBRUARY 10, 1994
COMPLETED BY: R. HELLER
TELEPHONE: (609)339-5162

WO NO	UNIT	EQUIPMENT IDENTIFICATION
931116130	1	VALVE 12MS15 FAILURE DESCRIPTION: PLUG THREADS INSIDE BODY DAMAGED DURING REMOVAL - REPLACE
931214109	1	SOLID STATE PROTECTION SYSTEM TRAIN "A" FAILURE DESCRIPTION: OUTPUT TEST READS LOW - INVESTIGATE
931214205	1	11 AUXILIARY FEEDWATER PUMP FAILURE DESCRIPTION: MOTOR SLINGER NOT SLINGING - INVESTIGATE
931229159	1	13 REACTOR COOLANT LOOP FAILURE DESCRIPTION: FLOW CHANNEL 2 READS 18% WITH NO LOOP FLOW - INVESTIGATE
940103180	1	12 AUXILIARY FEEDWATER PUMP FAILURE DESCRIPTION: ERRONEOUS INDICATION - INVESTIGATE
940110247	1	VALVE 13SJ54 FAILURE DESCRIPTION: VALVE OFF NORMAL INDICATION DID NOT CLEAR - INVESTIGATE
940117073	1	VALVE 12MS10 FAILURE DESCRIPTION: VALVE FAILED TO FULLY CLOSE - TROUBLESHOOT
940122099	1	VALVE 11BF40 FAILURE DESCRIPTION: VALVE POSITION WILL NOT REACH 100% OPEN, IT WILL ONLY REACH 95% - INVESTIGATE
940122100	1	VALVE 13BF40 FAILURE DESCRIPTION: VALVE DOES NOT REACH THE OPEN LIMIT - REWORK

10CFR50.59 EVALUATIONS
MONTH: - JANUARY 1994

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: FEBRUARY 10, 1994
COMPLETED BY: R. HELLER
TELEPHONE: (609)339-5162

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY

A. Design Change Packages

1EC-3208 Pkg 2

"Salem Fire Damper Upgrade" - The design scope for this package includes the replacement, relocation and/or modification of existing fire dampers of Unit 1's Auxiliary Building Ventilation (ABV) Containment Purge System. The function, basic configuration and operation of the system will not be altered and the codes, standards, qualification and design criteria of the original system will apply. Fire dampers 1ABF-016 and 1ABF-024 will be modified in place in accordance with the results of the manufacturer's (PREFCO) fire test report recommendations to obtain a 1.5 hour fire rating. The existing fire protective coating will not be necessary, and later replaced to facilitate damper modification. Fire dampers 1ABF-222 and 1ABF-228 will be replaced with new Ruskin 1.5 hour rated fire damper and moved into the fire barrier, eliminating the need for existing fire protective coatings. The reason for this change is to abide by the PSE&G commitment to the NRC and compliance with the requirements to 10CFR50, Appendix R. The margin of safety is not reduced because we are merely enhancing the Fire Protection System to meet the criteria of 10CFR50, Appendix "R". These modifications will not reduce the margin of safety for the Auxiliary Building Ventilation System or the Fire Protection System. (SORC 94-008)

10CFR50.59 EVALUATIONS
MONTH: - JANUARY 1994

DOCKET NO: 50-272
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COMPLETED BY: R. HELLER
TELEPHONE: (609)339-5162

(cont'd)

ITEM	SUMMARY

B. Procedures and Revisions

NC.NA-AP.ZZ-0027(Q) "Inservice Inspection Program" - The revision summary is as follows:

1.) This is a Limited Revision;

2.) The following technical changes was made at Step 5.4: Deleted requirement for SORC review of ISI program submittals to the NRC;

3.) The following minor, editorial changes were made to reflect the recent organization changes: a.) Changed "Nuclear Services" to "Nuclear Support and Services; b.) Changed "Site Services" to "Reliability and Assessment."; c.) Added "Manager Planning and Scheduling (Salem only)", and d.) Step 5.7.2: Changed history file location.

4.) Procedure was reformatted to comply with requirements of NC.NA-AP.ZZ-0032(Q) and NC.NA-AS.ZZ-0001.

5.) Revision bars were not used for typographical errors.

6.) This revision meets the biennial review requirements of NC.NA-AP.ZZ-0032(Q).

The Salem and Hope Creek Technical Specifications were reviewed, including Sections 4.0 and 6.0. The Technical Specification review did not uncover any inconsistencies with this procedure revision. Therefore, the proposal can not change the margin of safety as defined in the basis for any Technical Specification. (SORC 94-008)

(cont'd)

ITEM	SUMMARY

C. Temporary Modifications

T-Mod 94-006

"Installation of Temporary Space Heaters" - This T-Mod will provide additional necessary heating to the Unit 1 Auxiliary Feedwater Storage Tank and Refueling Water Storage Tank level instrumentation lines to prevent these instrument and sample lines from freezing. The T-Mod will install a temporary enclosure heated by space heaters. The temporary enclosure will be constructed of scaffold material enclosed by Herculite. The existing heat trace system is not adequate for the present cold spell at Salem Station. This T-Mod will be monitored hourly by Maintenance when the temporary heaters are in service, and removed when weather conditions permit. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 94-007)

D. Deficiency Report (Use-As-Is)

SMD 94-009

"Operability of the 11, 12, 13 and 14 GB4 Valves" - The 11, 12, 13 and 14 GB4 Valve actuator diaphragms may have reduced reliability due to a manufacturing defect with the actuators and a procedure error. The manufacturing error results in the diaphragm being compressed in the area of the bolt circle which may result in the diaphragm tearing. The procedure error addresses the torque value specified in procedure SC.IC-PM.ZZ-0003(Q) and results in the actuator lid bolting being torqued to a value double that recommended by the vendor. This may aggravate the manufacturing defect in tearing the diaphragms along the bolt circle. The GB-4 valves are air operated valves. The valves provide containment isolation for the Steam Generator Blowdown System. A diaphragm failure of this type will result in a tear along the bolt hole circle where the actuator lids and diaphragm are bolted together. This would result in the valve being unable to maintain an open position and would result in the valve closing. Since the valve is a spring assist to close valve and valve closure is based on the spring assist

(cont'd)

ITEM	SUMMARY

and evacuation of air from under the actuator diaphragm, the safety function of the valve to close is not affected. Loss of blowdown, in itself, is not a concern and is governed by chemistry concerns if blowdown is unavailable for long periods of time. The Technical Specification requirements for the GB-4 valves are to provide containment isolation. The failure of the diaphragm valve will result in the valve failing closed. The ability to satisfy the Technical Specification requirements is not affected during the time that the diaphragm fails since the ability of the valve to close is based on the spring assist and actuator air evacuation from under the diaphragm. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 94-008)

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
JANUARY 1994

SALEM UNIT NO. 1

The Unit began the period shutdown for the eleventh refueling outage. The Unit entered Mode 2 "Hot Standby" on January 24, 1994. Low Power Physics Tests were completed on January 25, 1994 and preparations for increasing power to Mode 1, "Power Operation" were performed. A Reactor Trip occurred on January 27, 1994, due to Lo-Lo Level in #14 Steam Generator. The reactor achieved criticality on January 30, 1994, and preparation for increasing power continued throughout the remainder of the period.

REFUELING INFORMATION
MONTH: - JANUARY 1994

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: FEBRUARY 10, 1994
COMPLETED BY: R. HELLER
TELEPHONE: (609)339-5162

MONTH JANUARY 1994

1. Refueling information has changed from last month:
   YES X NO _____

2. Scheduled date for next refueling: MARCH 4, 1995

3. Scheduled date for restart following refueling: MAY 2, 1995

4. a) Will Technical Specification changes or other license amendments be required?:
   YES _____ NO _____

   NOT DETERMINED TO DATE X

   b) Has the reload fuel design been reviewed by the Station Operating Review Committee?:
   YES _____ NO X

   If no, when is it scheduled?: MARCH 1995

5. Scheduled date(s) for submitting proposed licensing action:
   N/A

6. Important licensing considerations associated with refueling:

7. Number of Fuel Assemblies:

   a. Incore 193
   b. In Spent Fuel Storage 732

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4