

Public Service Electric and Gas Company

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Vice President - Nuclear Engineering

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Gentlemen:

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UPDATE TO SPENT FUEL POOL RERACK LICENSING REPORT SALEM GENERATING STATION UNIT NOS. 1 AND 2 FACILITY OPERATING LICENSE NOS. DPR-70 AND DPR-75 DOCKET NOS. 50-272 AND 50-311

In a letter dated April 28, 1993, Public Service Electric and Gas Company (PSE&G) submitted a request for amendment of Facility Operating Licenses DPR-70 and DPR-75 for Salem Generating Station, Unit Nos. 1 and 2. The proposed changes increased the Spent Fuel Pool capacities for Unit Nos. 1 and 2 from the current 1170 to 1632 fuel assemblies, and extended the decay time for Refueling Operations from 100 to 168 hours.

That letter contained a Licensing Report, as Attachment D, which provided the technical justifications to support the requested changes. Licensing Report page 3-5 indicated that ASME Code Section III (NCA 3800) would be used for spent fuel rack material Quality Assurance (QA) requirements. Our primary contractor (Holtec) has requested that PSE&G allow the use of a 10CFR50 Appendix B QA Program in lieu of the ASME Code. PSE&G has reviewed this request and notified Holtec that spent fuel rack materials purchased per a 10CFR50 Appendix B QA Program, and authorized for construction under the provisions of ASME Section III NF are acceptable. Mr. J. Stone, NRR Project Manager, was notified of this change on July 7, 1993.

The upgraded Fuel Handling Crane design has been finalized. We have selected an Ingersoll-Rand hoist that consists of one hoist. Licensing Report page 2-8 states that crane upgrading calls for installing a lifting system consisting of four hoists in parallel, which together have a rated capacity of over 20 tons. The use of one hoist with a capacity of over 20 tons is an improvement over a four hoist system due to the elimination of load equalization concerns. The revised sentence should read, "upgrading of this crane calls for installing a lifting system consisting of a hoist with a rated capacity of over 20 tons."

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PSE&G completed calculations to support Fuel Handling Crane uprating, as described in our Licensing Report. These calculations complied with the requirements of NUREG-0612. During a subsequent review, we noted that Licensing Report page 2-8, Section (i) stated that a postulated load of 30 tons (1.5 times design load) produced primary stresses less than the crane structural component's material yield strenth. This 30 ton value is incorrect. The correct postulated load value is 60 tons (3.0 times design load). The revised sentence should read, "additionally, a postulated load of 60 tons (3.0 times design load) is shown to produce primary stresses which are less than the material yield strength of the crane structural components."

Please update our previously submitted Licensing Report to reflect these changes. Should you have any questions on this transmittal, please contact us.

Sincerely,

C Mr. T. T. Martin, Administrator - Region I U. S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

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