ATTACHMENT TO LICENSE AMENDMENT NO. 125

TO FACILITY COMBINED LICENSE NO. NPF-92

DOCKET NO. 52-026

Replace the following pages of the Facility Combined License No. NPF-92 with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Combined License No. NPF-92

REMOVE	INSERT
7	7

Appendix A to Facility Combined License Nos. NPF-91 and NPF-92

REMOVE	INSERT
3.3.9-5	3.3.9-5
3.3.19-3	3.3.19-3
3.6.6-1	3.6.6-1
3.7.9-1	3.7.9-1
3.7.9-2	3.7.9-2
3.7.9-3	3.7.9-3

Appendix C to Facility Combined License No. NPF-92

REMOVE	INSERT
C-237	C-237
C-238	C-238

(7) Reporting Requirements

(a) Within 30 days of a change to the initial test program described in UFSAR Section 14, Initial Test Program, made in accordance with 10 CFR 50.59 or in accordance with 10 CFR Part 52, Appendix D, Section VIII, "Processes for Changes and Departures," SNC shall report the change to the Director of NRO, or the Director's designee, in accordance with 10 CFR 50.59(d).

(b) SNC shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) Incorporation

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively of this license, as revised through Amendment No. 125, are hereby incorporated into this license.

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SNC shall implement the programs or portions of programs identified below, on or before the date SNC achieves the following milestones:

(a) Environmental Qualification Program implemented before initial fuel load;

(b) Reactor Vessel Material Surveillance Program implemented before initial criticality;

(c) Preservice Testing Program implemented before initial fuel load;

(d) Containment Leakage Rate Testing Program implemented before initial fuel load;

(e) Fire Protection Program

1. The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt

Table 3.3.9-1 (page 1 of 2)
Engineered Safeguards Actuation System Instrumentation

	FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS
1.	Safeguards Actuation - Manual Initiation	1,2,3,4	2 switches	E
		5	2 switches	J
2.	Core Makeup Tank (CMT) Actuation - Manual Initiation	1,2,3,4[(a)]	2 switches	D
		4[(b)], 5[(d)]	2 switches	G
3.	Containment Isolation - Manual Initiation	1,2,3,4	2 switches	E
4.	Steam Line Isolation - Manual Initiation	1,2,3,4	2 switches	F
5.	Feedwater Isolation - Manual Initiation	1,2,3,4	2 switches	F
6.	ADS Stages 1, 2 & 3 Actuation - Manual Initiation	1,2,3,4	2 switch sets	E
		5[(d)]	2 switch sets	H
7.	ADS Stage 4 Actuation - Manual Initiation	1,2,3,4	2 switch sets	E
		5	2 switch sets	H
		6[(e)]	2 switch sets	I
8.	Passive Containment Cooling Actuation - Manual Initiation	1,2,3,4	2 switches	E
		5[(f)]	2 switches	J
		6[(f)]	2 switches	K
9.	Passive Residual Heat Removal Heat Exchanger Actuation - Manual Initiation	1,2,3,4	2 Switches	E
		5[(c)]	2 switches	G
10.	Chemical and Volume Control System Makeup Isolation - Manual Initiation	1,2,3,4[(a)]	2 switches	F
11.	Normal Residual Heat Removal System Isolation - Manual Initiation	1,2,3	2 switch sets	F

(a) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).

(b) With the RCS being cooled by the RNS.

(c) With the RCS pressure boundary intact.

(d) With RCS not VENTED.

(e) With upper internals in place.

(f) With decay heat > 7.0 MWt.

Table 3.3.19-1 (page 1 of 1)
DAS Manual Controls

	FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CONTROLS
1.	Reactor trip manual controls	1,2	2 switches
2.	Passive Residual Heat Removal Heat Exchanger (PRHR HX) control and In-Containment Refueling Water Storage Tank (IRWST) gutter control valves	1,2,3,4,5[(a)]	2 switches
3.	Core Makeup Tank (CMT) isolation valves	1,2,3,4,5[(a)]	2 switches
4.	Automatic Depressurization System (ADS) stage 1 valves	1,2,3,4,5[(a)]	2 switches
5.	ADS stage 2 valves	1,2,3,4,5[(a)]	2 switches
6.	ADS stage 3 valves	1,2,3,4,5[(a)]	2 switches
7.	ADS stage 4 valves	1,2,3,4,5,6[(c)]	2 switches
8.	IRWST injection squib valves	1,2,3,4,5,6	2 switches
9.	Containment recirculation valves	1,2,3,4,5,6	2 switches
10.	Passive containment cooling drain valves	1,2,3,4,5[(b)],6[(b)]	2 switches
11.	Selected containment isolation valves	1,2,3,4,5,6	2 switches

(a) With Reactor Coolant System (RCS) pressure boundary intact.

(b) With the reactor decay heat > 7.0 MWt.

(c) With reactor internals in place.

3.6 CONTAINMENT SYSTEMS

3.6.6 Passive Containment Cooling System (PCS)

LCO 3.6.6 The passive containment cooling system shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4,
MODES 5 and 6 with the reactor decay heat > 7.0 MWt.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One passive containment cooling water flow path inoperable.	A.1 Restore flow path to OPERABLE status.	7 days
B. Two passive containment cooling water flow paths inoperable.	B.1 Restore one flow path to OPERABLE status.	72 hours
C. One or more water storage tank parameters not within limits.	C.1 Restore water storage tank to OPERABLE status.	8 hours
D. Required Action and associated Completion Time of Condition A, B, or C not met in MODE 1, 2, 3, or 4. OR LCO not met for reasons other than Condition A, B, or C in MODE 1, 2, 3, or 4.	D.1 Be in MODE 3. AND D.2 Be in MODE 5.	6 hours 84 hours

3.7 PLANT SYSTEMS

3.7.9 Spent Fuel Pool Makeup Water Sources

LCO 3.7.9 Spent fuel pool makeup water sources shall be OPERABLE.

- NOTES -

1. OPERABILITY of the cask washdown pit is required when the spent fuel pool decay heat > 4.0 MWt and ≤ 7.0 MWt.

2. OPERABILITY of the cask loading pit is required when the spent fuel pool decay heat > 5.0 MWt and ≤ 7.0 MWt.

3. OPERABILITY of the Passive Containment Cooling Water Storage Tank (PCCWST) is required as a spent fuel pool makeup water source when the spent fuel pool decay heat > 7.0 MWt. If the reactor decay heat is > 7.0 MWt, the PCCWST must be exclusively available for containment cooling in accordance with LCO 3.6.6.

4. OPERABILITY of the Fuel Transfer Canal is required.

APPLICABILITY: When irradiated fuel assemblies are stored in the spent fuel pool.

ACTIONS

- NOTE -

LCO 3.0.3 is not applicable.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more required spent fuel pool makeup water sources inoperable.	A.1 Initiate action to restore the required makeup water source(s) to OPERABLE status.	Immediately

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.7.9.1	---------- **- NOTE -** Only required to be performed when spent fuel pool decay heat is > 7.0 MWt. ---------- Verify one passive containment cooling system, motor-operated valve in each flow path is closed and locked, sealed, or otherwise secured in position.	7 days
SR 3.7.9.2	---------- **- NOTE -** Only required to be performed when spent fuel pool decay heat is > 7.0 MWt. ---------- Verify the PCCWST volume is ≥ 756,700 gallons.	7 days
SR 3.7.9.3	---------- **- NOTE -** Only required to be performed when spent fuel pool decay heat is ≤ 7.0 MWt. ---------- Verify the water level in the cask washdown pit is ≥ 13.75 ft.	31 days
SR 3.7.9.4	---------- **- NOTE -** Only required to be performed when spent fuel pool decay heat is > 5.0 MWt and ≤ 7.0 MWt. ---------- Verify the water level in the cask loading pit is ≥ 43.9 ft. and in communication with the spent fuel pool.	31 days
SR 3.7.9.5	Verify the fuel transfer canal is in communication with the spent fuel pool.	31 days

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.7.9.6	Verify the spent fuel pool makeup isolation valves PCS-PL-V009, PCS-PL-V045, PCS-PL-V051, SFS-PL-V042, SFS-PL-V045, SFS-PL-V049, SFS-PL-V066, and SFS-PL-V068 are OPERABLE in accordance with the Inservice Testing Program.	In accordance with the Inservice Testing Program

Table 2.3.7-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
			iii) Inspection will be performed for the existence of a report verifying that the as-built equipment including anchorage is seismically bounded by the tested or analyzed conditions.	iii) A report exists and concludes that the as-built equipment including anchorage is seismically bounded by the tested or analyzed conditions.
397	2.3.07.05.ii	Not used per Amendment No. 84		
398	2.3.07.05.iii	Not used per Amendment No. 84		
399	2.3.07.06a	6.a) The Class 1E components identified in Table 2.3.7-1 are powered from their respective Class 1E division.	Testing will be performed on the SFS by providing a simulated test signal in each Class 1E division.	A simulated test signal exists at the Class 1E components identified in Table 2.3.7-1 when the assigned Class 1E division is provided the test signal.
400	2.3.07.06b	Not used per Amendment No. 84		
401	2.3.07.07a	Not used per Amendment No. 84		
402	2.3.07.07b.i	7.b) The SFS provides spent fuel cooling for 7 days by boiling the spent fuel pool water and makeup water from on-site storage tanks.	i) Inspection will be performed to verify that the spent fuel pool includes a sufficient volume of water.	i) The volume of the spent fuel pool, fuel transfer canal, and both gate areas above the fuel assemblies and below the spent fuel pool cooling suction piping is greater than or equal to 130,350 gallons.
403	2.3.07.07b.ii	7.b) The SFS provides spent fuel cooling for 7 days by boiling the spent fuel pool water and makeup water from on-site storage tanks.	ii) Inspection will be performed to verify the cask washdown pit includes sufficient volume of water.	ii) The water volume of the cask washdown pit from the cask washdown pit floor to 13.75 feet above the cask washdown pit floor is greater than or equal to 34,100 gallons.
404	2.3.07.07b.iii	Not used per Amendment No. 84		
405	2.3.07.07b.iv	Not used per Amendment No. 84		

Table 2.3.7-4
Inspections, Tests, Analyses, and Acceptance Criteria

No.	ITAAC No.	Design Commitment	Inspections, Tests, Analyses	Acceptance Criteria
406	2.3.07.07b.v	Not used per Amendment No. 84		
407	2.3.07.07b.vi	Not used per Amendment No. 84		
881	2.3.07.07b.vii	7.b) The SFS provides spent fuel cooling for 7 days by boiling the spent fuel pool water and makeup water from on-site storage tanks.	vii) Inspection will be performed to verify the cask loading pit includes sufficient volume of water.	vii) The water volume of the cask loading pit above the bottom of the gate and below the spent fuel pool cooling suction piping is greater than or equal to 46,050 gallons.
408	2.3.07.07c	7c) The SFS provides check valves in the drain line from the refueling cavity to prevent flooding of the refueling cavity during containment flooding.	Exercise testing of the check valves with active safety-functions identified in Table 2.3.7-1 will be performed under pre-operational test pressure, temperature and flow conditions.	Each check valve changes position as indicated on Table 2.3.7-1.
		8. The SFS provides the nonsafety-related function of removing spent fuel decay heat using pumped flow through a heat exchanger.	ii) Testing will be performed to confirm that each SFS pump provides flow through its heat exchanger when taking suction from the SFP and returning flow to the SFP.	ii) Each SFS pump produces at least 900 gpm through its heat exchanger.
		9. Safety-related displays identified in Table 2.3.7-1 can be retrieved in the MCR.	Inspection will be performed for retrievability of the safety-related displays in the MCR.	Safety-related displays identified in Table 2.3.7-1 can be retrieved in the MCR.
		10. Controls exist in the MCR to cause the pumps identified in Table 2.3.7-3 to perform their listed functions.	Testing will be performed to actuate the pumps identified in Table 2.3.7-3 using controls in the MCR.	Controls in the MCR cause pumps identified in Table 2.3.7-3 to perform the listed functions.
		11. Displays of the SFS parameters identified in Table 2.3.7-3 can be retrieved in the MCR.	Inspection will be performed for retrievability in the MCR of the displays identified in Table 2.3.7-3.	Displays of the SFS parameters identified in Table 2.3.7-3 are retrieved in the MCR.
409	2.3.07.08.i	8. The SFS provides the nonsafety-related function of removing spent fuel decay heat using pumped flow through a heat exchanger.	i) Inspection will be performed for the existence of a report that determines the heat removal capability of the SFS heat exchangers.	i) A report exists and concludes that the heat transfer characteristic, UA, of each SFS heat exchanger is greater than or equal to 2.2 million Btu/hr-°F.
410	2.3.07.08.ii	Not used per Amendment No. 112		
411	2.3.07.09	Not used per Amendment No. 112		
412	2.3.07.10	Not used per Amendment No. 112		
413	2.3.07.11	Not used per Amendment No. 112		