

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

TO:  
Mr. R. D. Silver

FROM:  
Florida Power & Light Company  
Miami, Florida  
Mr. Robert E. Uhrig

DATE OF DOCUMENT  
7/30/76

DATE RECEIVED  
7/30/76

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DESCRIPTION

Ltr. notorized 7/30/76 w/attached.....re meeting on 7/29/76....regarding amdt to o1/ change to tech spec...concerning an unfavorable power distribution within the Unit 1 reactor core which has led to a delay in power ascension testing.

(4-P)

PLANT NAME:  
St. Lucie #1

ENCLOSURE

ACKNOWLEDGED  
DO NOT REMOVE

SAFETY

FOR ACTION/INFORMATION

ENVIRO 8/2/76

RJL

ASSIGNED AD:		ASSIGNED AD:
<input checked="" type="checkbox"/> BRANCH CHIEF:	Ziemann	BRANCH CHIEF:
<input type="checkbox"/> PROJECT MANAGER:		PROJECT MANAGER:
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			SITE TECH.
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TIC:	REG. VIE	ULRIKSON(ORNL)
NSIC:	LA PDR	
ASLB:	CONSULTANTS	
<input checked="" type="checkbox"/> ACRS / CYS HOLDING/SENT		

CONTROL NUMBER

7249

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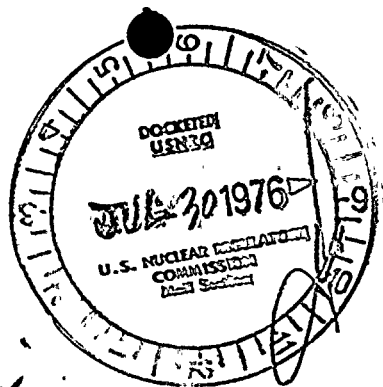
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U.S. NUCLEAR REGULATORY  
COMMISSION

1976 JUL 30 PM 4 18

TEL-COM-BR-DAO

Regulatory Docket File

DATE 7/30/76

MESSAGE FROM Larry L. Lobyan

Florida Power & Light Company  
PO Box 013100  
Miami, Florida 33101

TO: R.D. Silver - NRC. (DOR)

NO. OF PAGES 3 PLUS INSTRUCTION SHEET

TELECOPY NUMBER: (305) 552-4640 XEROX 410 - Unattended automatic machine

TELECOPY CONFIRMATION NUMBER (305) 552-3631

SPECIAL INSTRUCTIONS:

7249



7-11  
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July 30, 1976  
L-76-277

Director of Nuclear Reactor Regulation  
Attention: Mr. Victor Stello, Director  
Division of Operating Reactors  
U. S. Nuclear Regulatory Commission  
Washington DC 20555



Dear Mr. Stello:

Re: St. Lucie Unit 1  
Docket No. 50-335  
Proposed Amendment to  
Facility Operating License DPR-67

In accordance with 10 CFR 50.30, Florida Power & Light Company submits herewith three (3) signed originals and forty (40) copies of a request to amend Facility Operating License DPR-67.

Our request is the result of an unfavorable power distribution within the St. Lucie Unit 1 reactor core which has led to a delay in power ascension testing. Our plan for resolving this problem was discussed in a meeting with your staff on July 29, 1976. The plan includes the removal of several fuel elements followed by TV inspections of the reactor vessel internals. This will necessitate stopping the shutdown cooling water flow whenever the TV camera is in the core. However, Technical Specification 3.9.8 requires that at least one shutdown cooling loop shall be in operation. Therefore, we request that an interim license amendment be granted to waive this requirement during the TV inspections and to permit operation for up to 4 hours without shutdown cooling flow. A safety evaluation supporting our proposal is attached.

The proposed amendment has been reviewed and the conclusion reached that it does not involve a significant hazards consideration, therefore, prenoticing pursuant to 10 CFR 2.105 should not be required.

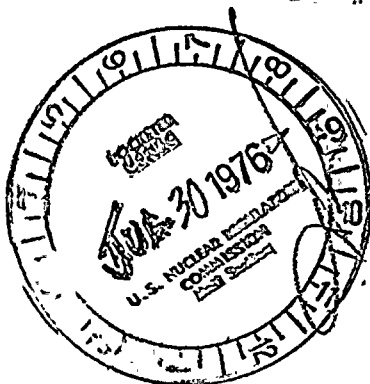
Very truly yours,

Regulatory Docket File

Robert E. Uhrig  
Vice President

REU:MAS:tg  
Attachment

cc: Normal C. Moseley  
Jack R. Newman, Esquire



7249

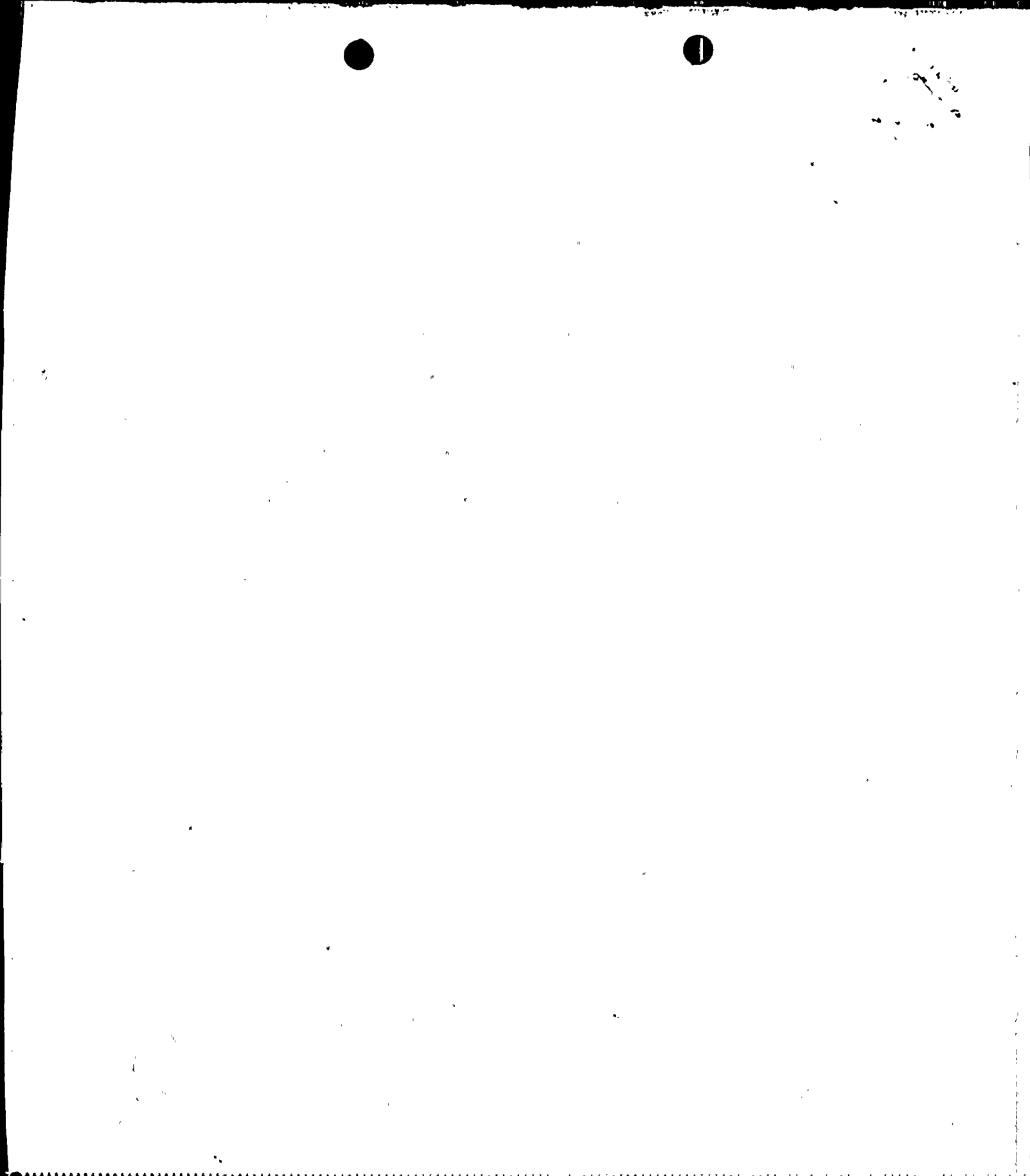


SAFETY EVALUATION FOR L-76-277

An exception is requested to Technical Specification 3.9.8 for the purpose of reactor vessel inspection in the refueling mode (MODE 6). This specification requires 3000 gpm shutdown cooling flow to the reactor vessel at all times in the refueling mode. The upcoming core inspection program requires securing of shutdown cooling flow to the reactor vessel for a four-hour period to permit inspection of the lower reactor vessel by remote means. This poses no hazard to the health and safety of the public for the following reasons:

1. The shutdown cooling system remains operable for immediate initiation on demand.
2. The 140°F RCS temperature limit in the refueling mode remains in effect. The present decay heat load due to a burnup of 800 MWD/MTU is much less than the decay heat loading due to a full core cycle burnup of 12,800 MWD/MTU.
3. The 3000 gpm flow requirement is based upon the boron dilution accident as analyzed in Chapter 15 of the FSAR. This accident contains the following assumptions as a basis:
  - a. All CEA's are removed from the core.
  - b. Boron dilution is effected via full flow from three charging pumps at maximum flow injecting primary dilution water into the reactor vessel for a period of 1.5 hours.
  - c. No shutdown cooling flow is present in the core

The boron dilution accident assumes conditions more severe than the conditions which will be present during the upcoming controlled evolution for core inspection. All CEAS's will be present in the fuel bundles and not removed from the core, thus giving at least 5% shutdown margin assuming no credit for boron concentration in the coolant. The CVCS flow path between primary dilution water and the charging pumps will be maintained locked closed under administrative control during this core inspection, thus eliminating the dilution mechanism assumed in the accident analysis.





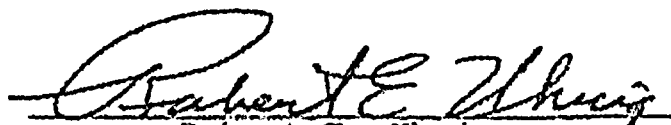
STATE OF FLORIDA )  
                          )  
COUNTY OF DADE )

ss.

Robert E. Uhrig, being first duly sworn, deposes and says:


That he is a Vice President of Florida Power & Light Company,  
the Licensee herein;

That he has executed the foregoing document; that the state-  
ments made in this said document are true and correct to the  
best of his knowledge, information, and belief, and that he  
is authorized to execute the document on behalf of said  
Licensee.

  
Robert E. Uhrig

Subscribed and sworn to before me this

30 day of July, 1976

  
NOTARY PUBLIC, in and for the County of Dade,  
State of Florida

My commission expires: NOTED THIS GENERAL INSURANCE UNDERWRITERS

NOTARY PUBLIC STATE OF FLORIDA AT LARGE  
MY COMMISSION EXPIRES JAN. 29, 1979

