

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO: Mr. Victor Stello		FROM: Florida Pwr. & Light Company Miami, Florida Robert E. Uhrig		DATE OF DOCUMENT 7/7/77
<input checked="" type="checkbox"/> LETTER <input checked="" type="checkbox"/> ORIGINAL <input type="checkbox"/> COPY		<input checked="" type="checkbox"/> NOTORIZED <input checked="" type="checkbox"/> UNCLASSIFIED		DATE RECEIVED 7/11/77
		PROP	INPUT FORM	NUMBER OF COPIES RECEIVED 1 SIGNED

DESCRIPTION

**DO NOT REMOVE
ACKNOWLEDGE**

(2-P)

PLANT NAME: St. Lucie Unit No. 1

RJL 7/12/77

ENCLOSURE

Amdt. to OL/change to Appendix A tech specs...notorized 7/7/77...concerning the relationship between linear heat rate and thermal power being revised & the rod bow penalty factor being deleted.....

(2-P)

SAFETY		FOR ACTION/INFORMATION		ENVIRONMENTAL	
ASSIGNED AD:		ASSIGNED AD:	V. MOORE (LTR)		
BRANCH CHIEF:	<i>Davis (5)</i>	BRANCH CHIEF:			
PROJECT MANAGER:	<i>Reviews</i>	PROJECT MANAGER:			
LICENSING ASSISTANT:	<i>DiSSS</i>	LICENSING ASSISTANT:			
		B. HARLESS			

INTERNAL DISTRIBUTION			
REG FILES	SYSTEMS SAFETY	PLANT SYSTEMS	SITE SAFETY & ENVIRON ANALYSIS
NRC PDR	HEINEMAN	TEDESCO	DENTON & MULLER
T & E (2)	SCHROEDER	BENAROYA	GRUTCHEFIELD
OELD		LAINAS	
GOSSICK & STAFF	ENGINEERING	IPPOLITO	
HANAUER	KNIGHT	F. ROSA	ENVIRO TECH.
MTPC	BOSNAK		ERNST
CASE	SIHWELL	OPERATING REACTORS	BALLARD
BOYD	PAWLICKI	STELLO	YOUNGBLOOD
		EISENHUT	
PROJECT MANAGEMENT	REACTOR SAFETY	SHAO	SITE TECH.
SKOVHOLT	ROSS	BAER	
P. COLLINS	NOVAK	BUTLER	GAMMILL (2)
HOUSTON	ROSZTOCZY	GRIMES	
MELTZ	CHECK		SITE ANALYSIS
HELTEMES			VOLLMER
SK	AT&I		BUNCH
	SALTZMAN		J. COLLINS
	RUTBERG		KREGER

EXTERNAL DISTRIBUTION		CONTROL NUMBER
LPDR: <i>FE Pierce Fla</i>		771930285 <i>Ap. 2</i> <i>GD</i>
TIC	NSIC	
NAT LAB		
REG IV (J. HANCHETT)		
16 CYS ACRS SENT CATEGORY <i>B</i>		

SECRET



July 7, 1977
L-77-214

Regulatory

File CV

Director of Nuclear Reactor Regulation
Attention: Mr. Victor Stello, Director
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Stello:

Re: St. Lucie Unit 1
Docket No. 50-335
Proposed Amendment to
Facility Operating License DPR-67



In accordance with 10 CFR 50.30, Florida Power & Light Company submits herewith three (3) signed originals and forty (40) copies of a request to amend Appendix A of Facility Operating License DPR-67.

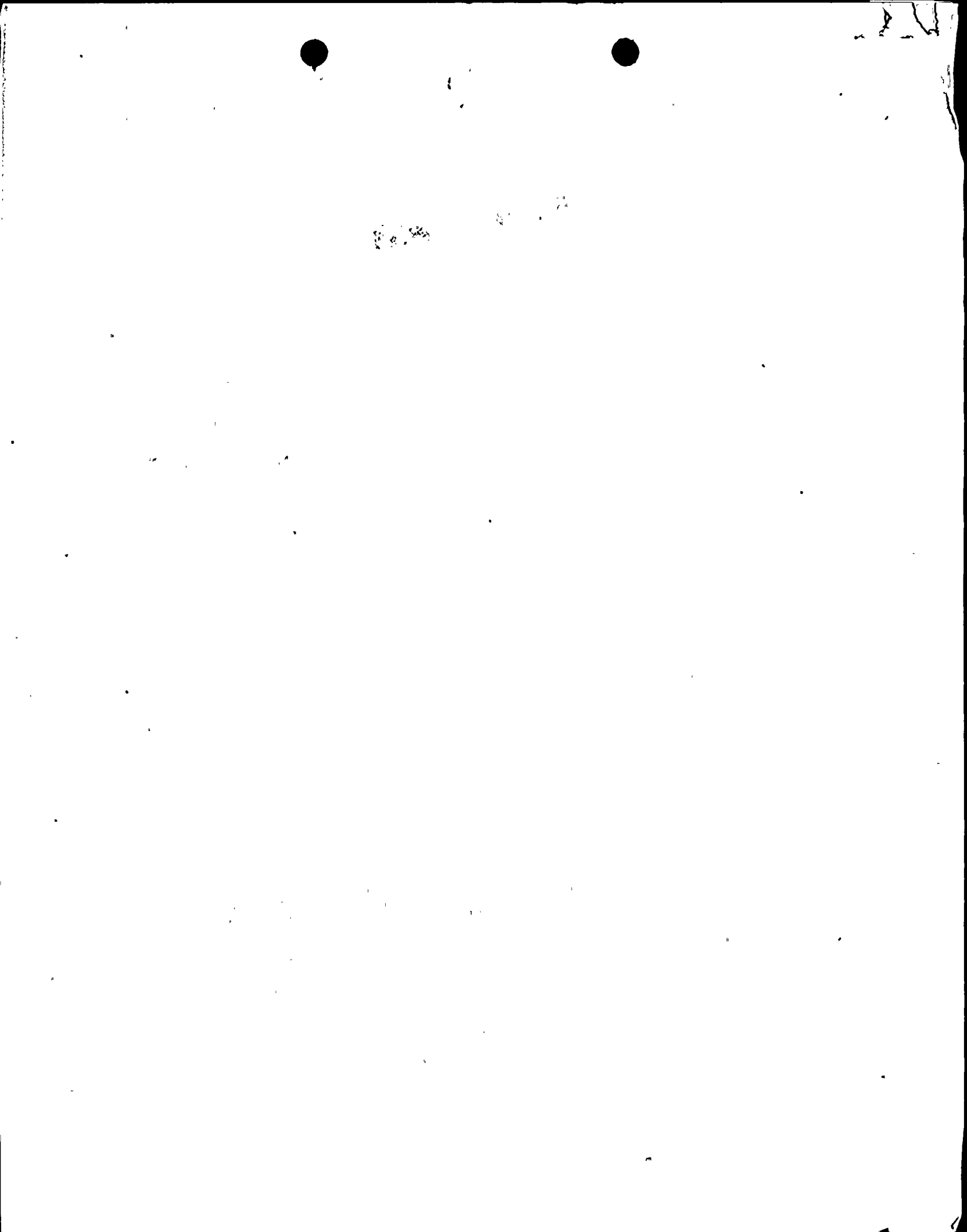
The proposed changes are described below and shown on the accompanying Technical Specification page with the date of this letter in the lower right hand corner.

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The relationship between linear heat rate and thermal power in Specification 4.2.1.3.c is revised, and the rod bow penalty factor in Specification 4.2.1.4.b is deleted. This is based on the position developed by our NSSS vendor that rod bow penalties on linear heat rate are covered by existing uncertainty factors and that rod bow penalties on DNB are not required. This position is further supported by the fact that rod bow penalties have been deleted from the Calvert Cliffs technical specifications.

The proposed amendment has been reviewed by the St. Lucie Facility Review Group and the Florida Power & Light Company Nuclear Review Board. They have concluded that it does not involve an unreviewed safety question. Please refer to proprietary topical report

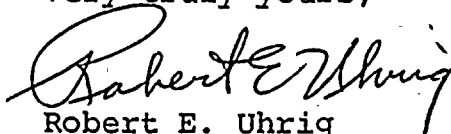
771930285



Director of Nuclear Reactor Regulation
Page Two

CENPD-225-P, "Fuel and Poison Rod Bowing", October 1976, for a detailed evaluation of rod bowing. The report concludes that no significant rod bow related effects on plant operation are expected for C-E reactors.

Very truly yours,

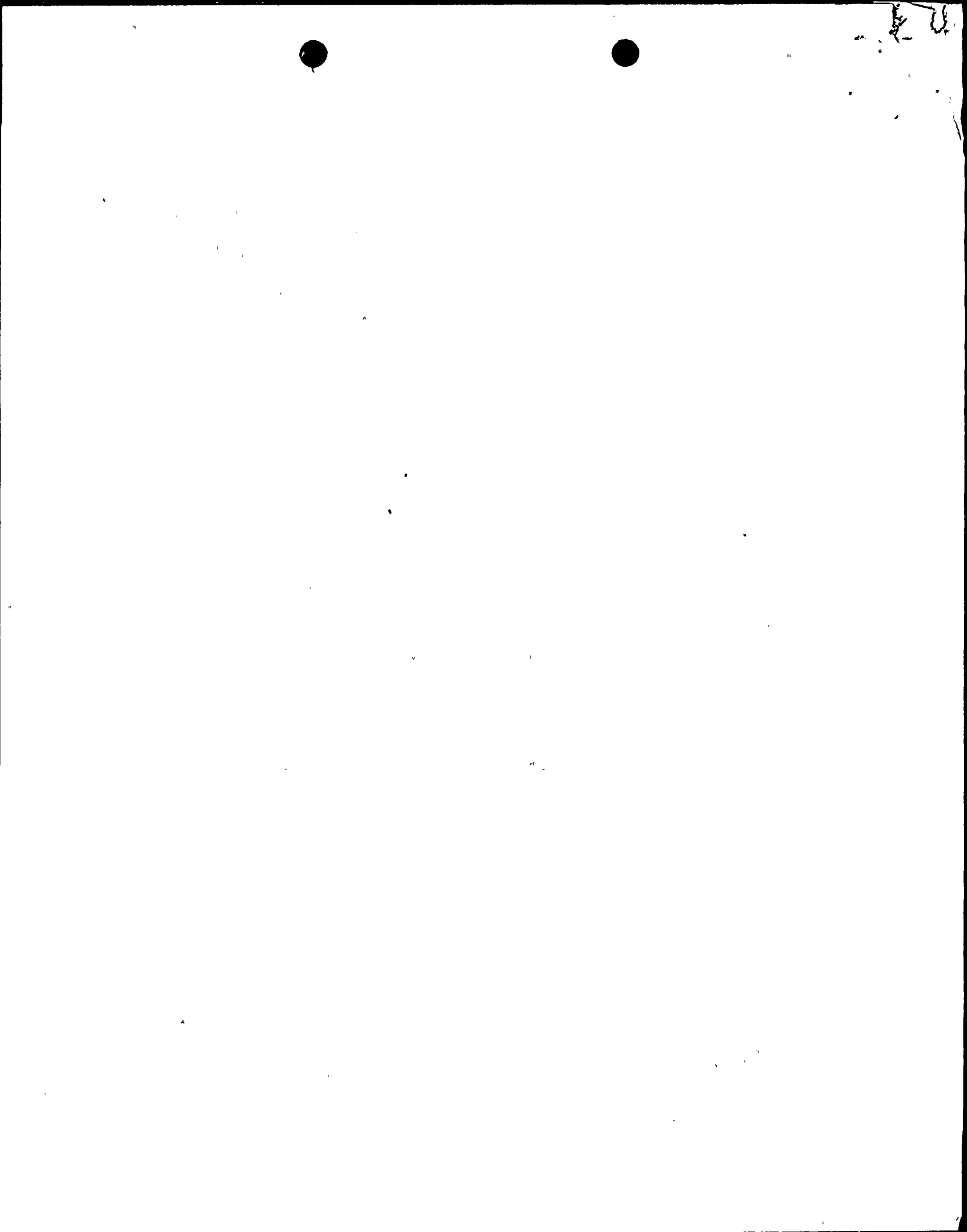


Robert E. Uhrig
Vice President

REU/MAS/cpc

Attachment

cc: Mr. Norman C. Moseley, Region II
Robert Lowenstein, Esquire



POWER DISTRIBUTION LIMITS

SURVEILLANCE REQUIREMENTS (Continued)

- c. Verifying at least once per 31 days that the THERMAL POWER does not exceed the value determined by the following relationship:

$$\frac{L}{17.0} \times M$$

where:

1. L is the maximum allowable linear heat rate as determined from Figure 3.2-1 and is based on the core average burnup at the time of the latest incore flux map.
2. M is the maximum allowable THERMAL POWER level for the existing Reactor Coolant Pump combination.

4.2.1.4 Incore Detector Monitoring System - The incore detector monitoring system may be used for monitoring the core power distribution by verifying that the incore detector Local Power Density alarms:

- a. Are adjusted to satisfy the requirements of the core power distribution map which shall be updated at least once per 31 days.
- b. Have their alarm setpoint adjusted to less than or equal to the limits shown on Figure 3.2-1 when the following factors are appropriately included in the setting of these alarms:
 1. Flux peaking augmentation factors as shown in Figure 4.2-1,
 2. A measurement-calculational uncertainty factor of 1.10,
 3. An engineering uncertainty factor of 1.03,
 4. A linear heat rate uncertainty factor of 1.01 due to axial fuel densification and thermal expansion,
 5. A THERMAL POWER measurement uncertainty factor of 1.02.

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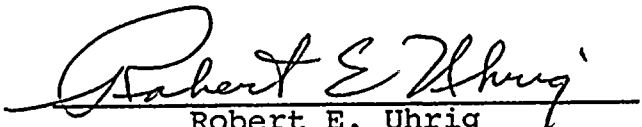


STATE OF FLORIDA)
)
COUNTY OF DADE) SS.

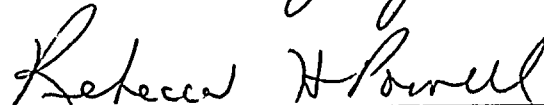
Robert E. Uhrig, being first duly sworn, deposes and says:

That he is a Vice President of Florida Power & Light Company,
the Licensee herein;

That he has executed the foregoing document; that the state-
ments made in this said document are true and correct to the
best of his knowledge, information, and belief, and that he
is authorized to execute the document on behalf of said
Licensee.


Robert E. Uhrig

Subscribed and sworn to before me this
7th day of July, 1977


NOTARY PUBLIC, in and for the County of Dade,
State of Florida

My commission expires: _____
NOTARY PUBLIC STATE OF FLORIDA at LARGE
MY COMMISSION EXPIRES APRIL 2, 1980
BONDED THRU MAYNARD BONDING AGENCY

100 11 11 11 11

1977 JUL 11 PM 2 01

RECEIVED DOCUMENT
PROCESSING UNIT

