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Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

January 5, 1993

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

SALEM GENERATING STATION
LICENSE NO. DPR-70
DOCKET NO. 50-272
UNIT NO. 1

LICENSEE EVENT REPORT 92-025-00

This Licensee Event Report is being submitted pursuant to the requirements of the Code of Federal Regulations 10CFR 50.73(a)(2)(i)(B). This report is required to be issued within thirty (30) days of event discovery.

Sincerely yours,

C. A. Vondra
General Manager -
Salem Operations

MJP:pc

Distribution

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Salem Generating Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 7 2	PAGE (3) 1 OF 0 4
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TITLE (4)
Tech Spec 3.0.3 entry; more than one analog rod position indicator per bank inoperable.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	2	06	9	2	00	0	1	05			0 5 0 0 0
9	2	92	9	2	00	0	1	05			0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																				
POWER LEVEL (10) 1 0 0	20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)	20.405(a)(1)(iv)	20.405(a)(1)(v)	20.405(c)	50.38(c)(1)	50.38(c)(2)	X 50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(x)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)

LICENSEE CONTACT FOR THIS LER (12)

NAME M.J. Pollack - LER Coordinator	TELEPHONE NUMBER 6 0 9 3 3 9 - 2 0 2 2
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	A/A	I/M/O/DH	015	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 12/6/92, at 1743 hours, during full power operation, the Control Room Operator identified that Analog Rod Position Indications (ARPIS) for Control Rods 1D4 and 2D1 were indicating greater than a + 12 step deviation from their group demand counter. Technical Specification 3.1.3.2.1 addresses the operability requirement of the "Reactivity Control System's" position indicating systems. Actions required when more than one ARPI per bank is inoperable exceed the Tech. Spec. 3.1.3.2.1 Limiting Condition For Operation; therefore, actions associated with Tech. Spec. 3.0.3 apply (and was entered on 12/6/92). On 12/6/92, at 1816 hours, the 2D1 control rod position indication was corrected by adjustment of the signal conditioning module. Tech. Spec. 3.0.3 was subsequently exited. Technical Specification Action 3.1.3.2.1a.a remained in affect until the 1D4 control rod position indication was corrected, at 1827 hours. The cause of the ARPIS having greater than + 12 step deviation from their group demand counter, for the two (2) control rods, is equipment failure. Signal conditioning module drift was responsible for the two (2) control rods to be indicating greater than + 12 steps from their group demand counter. The ARPI system electronics settings, associated with the Westinghouse Signal Condition Module, will have some drift. However, it is expected that this drift will not exceed the acceptable limit of + 12 steps from group demand within the 18 month calibration period. The signal conditioning module for the 2D1 and 1D4 control rod indicators are being replaced.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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PLANT AND SYSTEM IDENTIFICATION:

Westinghouse - Pressurized Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as {xx}

IDENTIFICATION OF OCCURRENCE:

Technical Specification 3.0.3 entry; more than one analog rod position indicator per bank inoperable

Event Dates: 12/06/92

Report Date: 1/05/93

This report was initiated by Incident Report No. 92-803.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 Reactor Power 100% - Unit Load 1150 MWe

On 12/6/92 at 0315 hours, a power increase from 81% to 100% was initiated. As of 0715 hours, reactor power was 100% with the control rods in manual.

DESCRIPTION OF OCCURRENCE:

On December 6, 1992, at 1743 hours, during full power operation, the Control Room Operator identified that Analog Rod Position Indications (ARPIs) for Control Rods 1D4 and 2D1 were indicating greater than a \pm 12 step deviation from their group demand counter.

Technical Specification 3.1.3.2.1 addresses the operability requirement of the "Reactivity Control System's" {AA} position indicating systems. The indicators are determined operable by verifying that the rod position indication system agrees within twelve (12) steps of the group demand counters. Actions required when more than one ARPI per bank is inoperable exceed the Technical Specification 3.1.3.2.1 Limiting Condition For Operation; therefore, actions associated with Technical Specification 3.0.3 apply (and was entered on December 6, 1992).

Technical Specification 3.0.3 states:

"When a Limiting Condition for Operation is not met except as provided in the associated ACTION requirements, within one hour action shall be initiated to place the unit in a MODE in which the specification does not apply by placing it, as applicable, in:

1. At least HOT STANDBY within the next 6 hours,
2. At least HOT SHUTDOWN within the following 6 hours, and

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DESCRIPTION OF OCCURRENCE: (cont'd)

3. At least COLD SHUTDOWN within the subsequent 24 hours. Where corrective measures are completed that permit operation under the ACTION requirements, the ACTION may be taken in accordance with the specified time limits as measured from the time of failure to meet the Limiting Condition of Operation. Exceptions to these requirements are stated in the individual specifications."

On December 6, 1992, at 1816 hours, the 2D1 control rod position indication was corrected by adjustment of the signal conditioning module. Technical Specification 3.0.3 was subsequently exited. Technical Specification Action 3.1.3.2.1a.a remained in affect until the 1D4 control rod position indication was corrected, at 1827 hours.

APPARENT CAUSE OF OCCURRENCE:

The cause of the ARPIS having greater than ± 12 step deviation from their group demand counter, for the two (2) control rods, is equipment failure. Maintenance-I&C investigation identified that signal conditioning module drift was responsible for the two (2) control rods to be indicating greater than ± 12 steps from their group demand counter.

The ARPI system electronics settings, associated with the Westinghouse Signal Condition Module, will have some drift. However, it is expected that this drift will not exceed the acceptable limit of ± 12 steps from group demand within the 18 month calibration period.

ANALYSIS OF OCCURRENCE:

Operability of the ARPIS is required to determine control rod position. This ensures compliance with control rod alignment and insertion limits assumed in the accident analyses. Investigation of this event showed that actual rod positions (per group demand) were correct, with only the indication being wrong. However, had a control rod actually been misaligned, the appropriate Technical Specification Action Statement requirements would have been met.

The false rod positioning indication did not affect the health or safety of the public; however, since Technical Specification 3.0.3 was entered, this event is reportable to the Nuclear Regulatory Commission in accordance with Code of Federal Regulations 10CFR 50.73(a)(2)(i)(B).

CORRECTIVE ACTION:

As stated in the Description of Occurrence section, the 2D1 and 1D4 control rod position indications were corrected by adjustment of their signal conditioning modules.

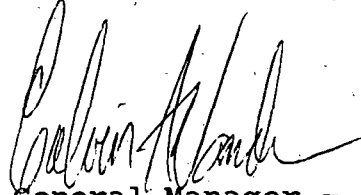
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CORRECTIVE ACTION: (cont'd)

A work order has been initiated to replace the 2D1 and 1D4 signal conditioning modules.

System Engineering is continuing its review of this event and similar prior events. Additional corrective action will be implemented based upon this review.



General Manager -
Salem Operations

MJP:pc

SORC Mtg. 93-001