



**PSEG**

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Salem Generating Station

November 13, 1992

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

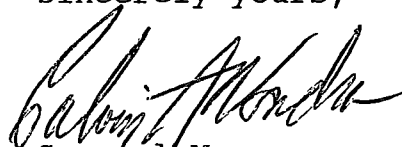
Dear Sir:

MONTHLY OPERATING REPORT  
SALEM NO. 1  
DOCKET NO. 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of October 1992 are being sent to you.

Average Daily Unit Power Level  
Operating Data Report  
Unit Shutdowns and Power Reductions  
Safety Related Maintenance  
10CFR50.59 Evaluations  
Operating Summary  
Refueling Information

Sincerely yours,

  
General Manager -  
Salem Operations

RH:pc

cc: Mr. Thomas T. Martin  
Regional Administrator USNRC  
Region I  
631 Park Avenue  
King of Prussia, PA 19046

Enclosures

8-1-7.R4

The Energy People

9211240055 921031  
PDR ADDCK 05000272  
R PDR

*JE24*

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272  
Unit Name: Salem #1  
Date: 11/10/92  
Telephone: 339-2122

Completed by: Mark Shedlock

Month OCTOBER 1992

Day Average Daily Power Level  
(MWe-NET)

Day Average Daily Power Level  
(MWe-NET)

1	<u>1052</u>
2	<u>1124</u>
3	<u>1122</u>
4	<u>1078</u>
5	<u>1124</u>
6	<u>1105</u>
7	<u>1113</u>
8	<u>1103</u>
9	<u>1099</u>
10	<u>1106</u>
11	<u>1124</u>
12	<u>1116</u>
13	<u>1113</u>
14	<u>1110</u>
15	<u>1110</u>
16	<u>1113</u>

17	<u>1097</u>
18	<u>1115</u>
19	<u>1104</u>
20	<u>1122</u>
21	<u>1106</u>
22	<u>1115</u>
23	<u>1112</u>
24	<u>957</u>
25	<u>38</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

OPERATING DATA REPORT

Completed by: Mark Shedlock

Docket No: 50-272  
 Date: 11/10/92  
 Telephone: 339-2122

Operating Status

1. Unit Name	<u>Salem No. 1</u>	<u>Notes</u>
2. Reporting Period	<u>October 1992</u>	
3. Licensed Thermal Power (Mwt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason	<u>N/A</u>	

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>745</u>	<u>7320</u>	<u>134473</u>
12. No. of Hrs. Rx. was Critical	<u>672.1</u>	<u>4144.7</u>	<u>87745.0</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>582.2</u>	<u>3766.2</u>	<u>84814.2</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1964308.8</u>	<u>12378115.2</u>	<u>267875260.4</u>
17. Gross Elec. Energy Generated (MWH)	<u>662440</u>	<u>4115240</u>	<u>88919870</u>
18. Net Elec. Energy Gen. (MWH)	<u>630856</u>	<u>3905527</u>	<u>84682100</u>
19. Unit Service Factor	<u>78.1</u>	<u>51.5</u>	<u>63.1</u>
20. Unit Availability Factor	<u>78.1</u>	<u>51.5</u>	<u>63.1</u>
21. Unit Capacity Factor (using MDC Net)	<u>76.6</u>	<u>48.2</u>	<u>56.9</u>
22. Unit Capacity Factor (using DER Net)	<u>75.9</u>	<u>47.9</u>	<u>56.5</u>
23. Unit Forced Outage Rate	<u>21.9</u>	<u>28.4</u>	<u>21.7</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

None

25. If shutdown at end of Report Period, Estimated Date of Startup:

N/A



SAFETY RELATED MAINTENANCE  
MONTH: - OCTOBER 1992

DOCKET NO: 50-272  
UNIT NAME: SALEM 1  
DATE: NOVEMBER 10, 1992  
COMPLETED BY: J. FEST  
TELEPHONE: (609)339-2904

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WO NO	UNIT	EQUIPMENT IDENTIFICATION
900810072	1	11 BAT PUMP
		FAILURE DESCRIPTION: UPGRADE PUMP - REPLACE POWER FRAME
921013206	1	VALVE 14AF920
		FAILURE DESCRIPTION: VALVE FAILED RT INSPECTION - DISSASSEMBLE AND INSPECT INTERNALS

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REFUELING INFORMATION  
MONTH: - OCTOBER 1992

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TELEPHONE: (609)339-2904

MONTH OCTOBER 1992

1. Refueling information has changed from last month:  
YES \_\_\_\_\_ NO X
2. Scheduled date for next refueling: OCTOBER 2, 1993
3. Scheduled date for restart following refueling: DECEMBER 13, 1993
4. a) Will Technical Specification changes or other license amendments be required?:  
YES \_\_\_\_\_ NO \_\_\_\_\_  
NOT DETERMINED TO DATE X
- b) Has the reload fuel design been reviewed by the Station Operating Review Committee?:  
YES \_\_\_\_\_ NO X  
If no, when is it scheduled?: \_\_\_\_\_
5. Scheduled date(s) for submitting proposed licensing action:  
N/A
6. Important licensing considerations associated with refueling:  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_
7. Number of Fuel Assemblies:
  - a. Incore 193
  - b. In Spent Fuel Storage 656
8. Present licensed spent fuel storage capacity: 1170  
Future spent fuel storage capacity: 1170
9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September 2001

10CFR50.59 EVALUATIONS  
MONTH: - OCTOBER 1992

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The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.  
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ITEM

SUMMARY

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A. Design Change Packages (DCPs)

1EE-0006 Pkg. 1

"Circulating Bearing Lube Blowdown and Drain Piping" - The purpose of this change is to install 1-1/2" circulator pump bearing lube blowdown piping and supports running from the pump stuffing box connection to a nearby floor drain for each of the six Unit 1 Circulating Water pumps. It also replaces corroded 1-1/2" carbon steel pump drain piping with stainless steel piping for the Unit 1 pumps, and adds 1-1/2" stainless steel drain piping for the Unit 2 pumps. The addition of drains and blowdown piping has no affect on the margins of safety that are the basis for the Technical Specifications. (SORC 92-108)

1EC-3198 Pkg. 1

"Demolition of Cathodic Protection Equipment and Chlorination Piping" - This modification removes: 1) The Cathodic Protection Equipment, raceway and associated cables for the Circulating Water (CW) pumps and Traveling Screens from both Units of Salem Generating Station (SGS). This modification also adds a grounding system for the Traveling Screens. 2) Abandoned portions of the chlorination system that had been used to inject hypochlorite into the intake bays to inhibit marine growth. This modification installs blind flanges on the supply lines from screen wash system. It removes the previously abandoned piping, supports, valves, and instrument tubing and trays from the Circulating Water Intake Structure, including injection piping submerged in the Circulating Water pump intake bays. This modification does not have any adverse effect on the margins of safety that are the basis for the Technical Specifications. The cathodic protection and chlorination systems are non-safety related and are part of the non-safety related Circulating Water (CW) system. Removal of these systems will not affect operation of the CW system. (SORC 92-108)

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(cont'd)

ITEM	SUMMARY
1EC-3195 Pkg. 1	<p>"Lube Oil Storage Facility Revitalization Project FC-0001 Units 1 &amp; 2" - This change involves the construction of a new common Lube Oil Storage Facility in the Unit 2 Turbine Building on floor elevation 100 ft for the storage of lubrication (lube) oils, grease and related accessories presently stored on floor elevation 88 ft and 120 ft of the Unit 1 Turbine Building. The proposed modification is non safety related and is not important to safety. This facility will service both Salem Units. The addition of the sprinkler system for the Lube Oil Storage Facility will not impact the capacity of the fire protection pumps. The water supply required for the sprinkler located in this facility is approximately 560 gpm which is well within the pump capacity of 2500 gpm as stated in the Technical Specifications. Therefore, this modification does not reduce the margin of safety as defined in the basis for any Technical Specification. (SORC 92-109)</p>
1SC-2267 Pkg. 4	<p>"SEC Circuit - Service Water Pump Start Test Switch Installation" - The purpose of this change is to provide a Key Lock Test Switch in service water pump breaker cubicle 1B8D for 14 primary service water pump, located in the 4KV Vital Bus Switchgear Room. This switch will be placed in the test position during Full Flow Test Surveillance of the service water pumps in Bay 3 to ensure that two service water pumps aligned to the plant will start upon receipt of an SEC initiation signal during all SEC Mode Operation conditions, thus avoiding a situation in which only a single pump aligned to the plant is started. The switch is not required when full flow testing is completed on the pumps located in Bay 1. This proposal does not reduce the margin of safety as defined in the basis for any Technical Specification. The only Technical Specification for the Service Water system states that "At least two independent service water loops shall be operable". Since the installation of a key lock test switch to provide an added degree of personnel safety during performance of existing test procedures does not affect the subject</p>



10CFR50.59 EVALUATIONS  
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ITEM	SUMMARY
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Technical Specification requirement, the margin of safety as defined in the basis for any Technical Specification is not reduced. (SORC 92-110)

1EE-0008 Pkg. 1

"Removal of STOCK EQUIPMENT Cement Station" - The STOCK EQUIPMENT cement station is to be demolished and removed from the Auxiliary Building radwaste facility truck Bay area (Elevation 100' - 0") to make room for the box compactor. The "station" has been emptied and cleaned out. There are existing facility service ties (electrical, pneumatic, ventilation) that must be modified/deleted before the cement station can be removed. The cement station may require dismantling and/or partial demolition prior to removal due to its size and weight. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 92-110)

1EC-3108 Pkg. 1

"Phase 2 of the Chiller Condenser Modifications" - The purpose of this change involves the replacement of 4" cement lined carbon steel service water piping with 6% molybdenum stainless steel. This DCP is the second of two phases implemented for the 4" chiller condenser piping. Phase 1 involved the replacement of all the 4" chiller condenser piping on elv. 100' in the mainsteam penetration area and chiller rooms. This DCP is scheduled for non-outage implementation (November 1992). The scope of work entails the replacement of No. 11 & 12 4" chiller condenser S.W. headers from the 24" S.W. nuclear headers located on el. 78' in the S.W. water valve rooms to a set of break flanges on el. 100' in the mainsteam penetration area. The piping configuration will remain the same with the following exceptions: 1. The four (4) manual isolation valves (11 & 12SW469 & 470), which are currently located in the overhead valve rooms. New chain operators will be installed in the valves due to the valves being located in the overhead (el 96'- 8"). 2. Orifice flanges and associated flow orifice plates will be installed on the supply lines to provide flow measurement

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(cont'd)

ITEM	SUMMARY
1EC-3138 Pkg. 1	capability for thermal performance testing of the chiller condensers. The margin of safety as defined in the Technical Specification bases is unaffected. Sufficient cooling is available to ensure control room air temperature does not exceed the allowable temperature for continuous duty rating for the equipment and habitability. (SORC 92-112)
	This DCP provides for revitalization upgrades to Rad Waste Panel 104 and related equipment on elevations 64' and 84' of the auxiliary building. Upgrades will consist of the following: a) replacement of obsolete instrumentation, b) physical repair of Panel 104, c) provision of new labels to enhance the operator interface with this panel, d) removal of unused components, including associated components external to the panel, e) upgrade of panel wire and cable restraints and labels, and f) functional verification of all panel instruments and components. Replacement of certain components used to monitor the administratively controlled transfer and release of radiological liquids and gasses will not affect any of the limiting conditions or safety margins described in Section 3/4.11. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 92-113)
B. Procedures and Revisions	
NC.NA-AP.ZZ-0025(Q)	"Nuclear Department Operational Fire Protection Program", Rev. 1 - This proposal involves a full revision to the Nuclear Department Operational Fire Protection Program procedure. This revision incorporates the requests by Station Management to modify the section on combustible material control with regards to the storage and use of combustible materials both in and out of safety related areas. Administrative control of transient combustibles has changed as a different method for completing the transient combustible evaluation by job supervisors and planners is now required. This revision does not alter the operability and

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(cont'd)

ITEM	SUMMARY
Security Plan	surveillance requirements of the Fire Protection Plant systems and Fire Barriers at Salem and does not alter the administrative controls specifically identified in either the Salem or Hope Creek Technical Specifications. This revision does not reduce the margin of safety as defined in the basis for any Technical Specification. (SORC 92-107)
Security Plan	"Revision 3 to Artificial Island Security Contingency Plan - Package 1" - The revision changes: 1) Assignments for making required modification of a threat event to reduce the burden on alarm station operators during initial response. 2) New emphasis is added on searching for additional devices and malicious acts, regardless of what caused the event to be declared. 3) Adds extortion to the threat profile. 4) The systematic method of searching for destructive devices is changed to be compatible with current training methods. 5) Incorporates changed reference to contingency event reporting pursuant to revised NRC guidance. 6) Expands the discussion of possible bases for declaring a Security Alert and to add direction concerning the strategic use of defensive positions. 7) Editorial changes which do not alter the plan's intent or implementation. There is no mention of this plan in the Technical Specifications. Therefore, the margin of safety as defined in the Technical Specifications is not reduced. (SORC 92-109)
Security Plan	"Revision 3 to the Artificial Island Security Plan" - This plan change described the physical changes to the protected area (PA) and incorporates provisions for screening material as a result of inclusion of the Centralized Warehouse in the PA. There is a change to the description of the duties of the security force in controlling vehicle access to vital areas. There is no mention of this plan in the Technical Specifications. Therefore, the margin of safety as defined in the Technical Specifications is not reduced. (SORC 92-109)

SALEM GENERATING STATION  
MONTHLY OPERATING SUMMARY - UNIT 1  
OCTOBER 1992

SALEM UNIT NO. 1

The Unit began the period increasing power from 70%. Power had been reduced to 70% to support Hope Creek outage 13KV switchyard activities. The Unit returned to full power on October 1, 1992, and continued to operate at essentially full power until October 24, 1992. On October 24, 1992, a power reduction commenced due to leakage on main feedwater regulating valve 12BF19. The Unit was removed from the grid on October 25, 1992, and remained shutdown throughout the rest of the period for repairs to 12BF19 and various other leaks on the Service Water, Heater Drain and Main Steam systems.