PSE&G

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Salem Generating Station

October 13, 1992

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of September 1992 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Safety Related Maintenance
10CFR50.59 Evaluations
Operating Summary
PORV or Safety Valve Challenges
Refueling Information

Sincerely yours,

General Manager -
Salem Operations

RH:pc

cc: Mr. Thomas T. Martin
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19046

Enclosures

8-1-7.R4

OPERATING DATA REPORT

Docket No: 50-311
Date: 10/10/92
Completed by: Mark Shedlock
Telephone: 339-2122

Operating Status

1.	Unit Name	Salem No. 2	Notes
2.	Reporting Period	September 1992	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity(Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	720	6575	96144
12. No. of Hrs. Rx. was Critical	640.8	2940.5	61556.6
13. Reactor Reserve Shutdown Hrs.	0	0	0
14. Hours Generator On-Line	627.3	2511.8	59410.6
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2032075.2	7789552.8	137901274.6
17. Gross Elec. Energy Generated (MWH)	674930	2535970	62263018
18. Net Elec. Energy Gen. (MWH)	643931	2370243	59238528
19. Unit Service Factor	87.1	38.2	61.8
20. Unit Availability Factor	87.1	38.2	61.8
21. Unit Capacity Factor (using MDC Net)	80.9	32.6	55.7
22. Unit Capacity Factor (using DER Net)	80.2	32.3	55.3
23. Unit Forced Outage Rate	12.9	26.0	23.4

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

Refueling outage scheduled to start 03/27/93 and last 55 days.

25. If shutdown at end of Report Period, Estimated Date of Startup:

N/A

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-311
Unit Name: Salem #2
Date: 10/10/92
Telephone: 339-2122

Completed by: Mark Shedlock

Month September 1992

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	1107	17	1099
2	1075	18	1103
3	413	19	1114
4	0	20	1113
5	0	21	1106
6	0	22	1090
7	359	23	1098
8	1086	24	1098
9	1073	25	1113
10	1088	26	340
11	1097	27	1065
12	1107	28	1028
13	1100	29	1002
14	1084	30	1072
15	858	31	
16	1051		

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH SEPTEMBER 1992

DOCKET NO.: 50-311
UNIT NAME: Salem #2
DATE: 10/10/92
COMPLETED BY: Mark Shedlock
TELEPHONE: 339-2122

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
0022	09/03/92	F	92.7	A	3	------	IA	CKTBRK	REACTOR TRIP BREAKER TRIP
0021	09/15/92	F	15.3	A	5	------	HJ	VALVEX	REHEAT STOP VALVE
0023	09/26/92	F	16.2	A	5	------	HF	MOTORX	CIRCULATING WATER PUMPS
0024	09/26/92	F	3.2	B	5	------	RB	ZZZZZZ	REACTOR PERFORMANCE DATA

1
F: Forced
S: Scheduled

2
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Requlatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

SAFETY RELATED MAINTENANCE
MONTH: - SEPTEMBER 1992

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: OCTOBER 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

WO NO	UNIT	EQUIPMENT IDENTIFICATION
920811102	2	VALVE 26SW25
		FAILURE DESCRIPTION: VALVE NOT FULLY CLOSING, INSPECT & REPAIR
920916077	2	2C DIESEL GENERATOR
		FAILURE DESCRIPTION: CLEAN OR REPLACE THE STARTER
920921116	2	PZR LEVEL
		FAILURE DESCRIPTION: PZR LEVEL CHANNEL II 3% DIFFERENT FROM CHANNELS I AND III - INVESTIGATE

10CFR50.59 EVALUATIONS
MONTH: - SEPTEMBER 1992

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: OCTOBER 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY

(None for the period.)

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 2
SEPTEMBER 1992

SALEM UNIT NO. 2

The Unit began the period operating at full power. On September 3, 1992, a Reactor Scram occurred due to the "A" Reactor Trip Breaker opening without a demand signal. An Unusual Event (UE) was declared following the trip due to steam generator safety valves lifting and not reseating as expected. The UE was terminated following closure of the safety valves. The Unit was restored to full power on September 7, 1992, and continued to operate at full power until September 15, 1992, when power was reduced to 40% to repair "22W" reheat stop valve. The repairs were completed and the Unit was restored to full power later, the same day. The Unit continued to operate at full power until September 26, 1992, when power was reduced to 28% due to tripping of the circulators which are powered from the Hope Creek Switchyard. The Unit was returned to full power on September 27, 1992, and continued to operate at essentially full power throughout the remainder of the period.

CHALLENGES TO PORVs OR SAFETY VALVES

Salem Unit 2

In accordance with the requirements of Salem Generating Station Unit 2 Technical Specifications Section 6.9.1.6, the following recent challenges to PORVs or Safety Valves are being reported:

* 09/03/92 - On September 3, 1992, at 0938 hours, following a trip of Salem Unit 2, Main Steam Safety Valves 21MS15 and 22MS15 lifted. The highest observed Steam Generator pressures were 1020 psig on 21 and 22 Steam Generator (S/G) loops, while 1035 psig was observed on 23 S/G. This pressure was higher than the others due to the 23MS10 being tagged for maintenance. An Unusual Event (UE) was declared at 1005 hours due to 21 and 22MS15s lifting and failing to reseat at a pressure (980 psig) below the safety valve setpoint. A cooldown was started at about 1010 hours to reseat the safety valves. 22MS15 briefly reseated from 1018 to 1020 hours with 22 S/G pressure at 990 psig. 21MS15 reseated at 1029 hours with Tave at 547^{o}F and 21 S/G pressure at 980 psig. 22MS15 reseated fully at 1041 hours with Tave at 540^{o}F and S/G pressure at 930 psig. A slow heatup was commenced at 1118 hours to verify proper reseating of the safety valves. The heatup was terminated at 1235 hours due to leakage observed from the safety valves at 544.6^{o}F on RCS Tave and at 986 and 970 psig pressures in 21 and 22 S/Gs respectively. Three safety valve lift setpoints (21-23MS15) have been determined to have lifted within the 3% tolerance band of the rated lift setpoint. Adjustments were made to all three valves to restore them to the 1% acceptance criterion.

REFUELING INFORMATION
MONTH: - SEPTEMBER 1992

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: OCTOBER 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

MONTH SEPTEMBER 1992

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: MARCH 27, 1993

3. Scheduled date for restart following refueling: MAY 21, 1993

4. a) Will Technical Specification changes or other license amendments be required?:
   YES _____ NO _____

   NOT DETERMINED TO DATE x

   b) Has the reload fuel design been reviewed by the Station Operating Review Committee?:
   YES _____ NO x

   If no, when is it scheduled?: FEBRUARY 1993

5. Scheduled date(s) for submitting proposed licensing action:
   N/A

6. Important licensing considerations associated with refueling:

7. Number of Fuel Assemblies:

   a. Incore 193
   b. In Spent Fuel Storage 408

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March 2003