PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Salem Generating Station

July 14, 1992

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of June 1992 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Safety Related Maintenance
10CFR50.59 Evaluations
Operating Summary
Refueling Information

Sincerely yours,

General Manager -
Salem Operations

RH:pc

cc: Mr. Thomas T. Martin
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19046

Enclosures

8-1-7.R4

9207200338 920630
PDR ADOCK 05000272
R PDR

095

95-2189 (10M) 12-89

AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-272
Unit Name: Salem #1
Date: 07/10/92
Telephone: 339-2122

Completed by: Mark Shedlock

Month June 1992

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	
16	0		

OPERATING DATA REPORT

Docket No: 50-272
Date: 07/10/92
Completed by: Mark Shedlock
Telephone: 339-2122

Operating Status

			Notes
1.	Unit Name	Salem No. 1	
2.	Reporting Period	June 1992	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity(Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	720	4367	131520
12. No. of Hrs. Rx. was Critical	0	2256.2	85856.4
13. Reactor Reserve Shutdown Hrs.	0	0	0
14. Hours Generator On-Line	0	2117.8	83165.7
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	7203120.0	262700265.2
17. Gross Elec. Energy Generated (MWH)	0	2416240	87220870
18. Net Elec. Energy Gen. (MWH)	-3331	2305872	83082445
19. Unit Service Factor	0	48.5	63.2
20. Unit Availability Factor	0	48.5	63.2
21. Unit Capacity Factor (using MDC Net)	0	47.7	57.1
22. Unit Capacity Factor (using DER Net)	0	47.4	56.7
23. Unit Forced Outage Rate	100	8.1	21.0

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

We are presently in a forced maintenance outage.

25. If shutdown at end of Report Period, Estimated Date of Startup:

July 28,1992

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH JUNE 1992

DOCKET NO.: 50-272
UNIT NAME: Salem #1
DATE: 07/10/92
COMPLETED BY: Mark Shedlock
TELEPHONE: 339-2122

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
0016	06/01/92	S	672.0	C	4	----------	RC	FUELXX	NUCLEAR NORMAL REFUELING
0017	06/29/92	F	48.0	B	4		CH	PIPEXX	STEAM GEN. PIPING REPAIRS

1
F: Forced
S: Scheduled

2
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Requlatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

SAFETY RELATED MAINTENANCE
MONTH: - JUNE 1992

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: JULY 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

WO NO	UNIT	EQUIPMENT IDENTIFICATION
890429014	1	1B REACTOR TRIP BYPASS BREAKER FAILURE DESCRIPTION: REPLACE THE UNDERVOLTAGE TRIP DEVICE
890429016	1	1A REACTOR TRIP BYPASS BREAKER FAILURE DESCRIPTION: REPLACE THE UNDERVOLTAGE TRIP DEVICE
910320151	1	FLUX THIMBLES FAILURE DESCRIPTION: REPLACE 6 UNIT 1 FLUX THIMBLES
910718114	1	VALVE 11SW40 FAILURE DESCRIPTION: VALVE HAS NO CLOSED INDICATION - TROUBLESHOOT
910724130	1	VALVE 16SW15 FAILURE DESCRIPTION: VALVE IS LEAKING THOUGH, INSPECT AND REPAIR
911118119	1	SOLENOID VALVE 1SV263 FAILURE DESCRIPTION: VALVE 1SV263 HAS LEAKING TUBING - REWORK
911226077	1	VALVE 1SW175 FAILURE DESCRIPTION: VALVE 1SW175 HAS NO CLOSED INDICATION - INVESTIGATE
920123168	1	SPOOL RD-1-SW-112 FAILURE DESCRIPTION: SPOOL RD-1-SW-112 UT EXAMINATION UNSAT - FABRICATE AND INSTALL REPLACEMENT SPOOL
920201007	1	VALVE 1PR15 FAILURE DESCRIPTION: 1PR15 - REPLACE DIAPHRAGM
920218126	1	VALVE 1PR2 FAILURE DESCRIPTION: 1PR2/PZR PORV - REPLACE COVER PLATE AND DIAPHRAGM

SAFETY RELATED MAINTENANCE
MONTH: - JUNE 1992

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UNIT NAME: SALEM 1
DATE: JULY 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

(cont'd)

WO NO	UNIT	EQUIPMENT IDENTIFICATION
920405078	1	SPOOL 1-SW-P-1108 FAILURE DESCRIPTION: SPOOL 1-SW-P-1108 HAS A THROUGH WALL LEAK - REPAIR
920415158	1	VALVE 1SS49 FAILURE DESCRIPTION: VALVE 12249 FAILED LEAK RATE TEST - OPEN, INSPECT AND REPAIR
920420109	1	CRDM CABLES FAILURE DESCRIPTION: UNIT 1 CRDM CABLE PROBLEMS FOUND DURING REACTOR DISASSEMBLY - REPAIR
920421149	1	VALVE 1CV4 FAILURE DESCRIPTION: VALVE 1CV4 FAILED LRT/STROKE - TROUBLESHOOT
920424091	1	13 SWITCHGEAR RM SUPPLY FAN FAILURE DESCRIPTION: 13 SWITCHGEAR ROOM SUPPLY FAN STARTS - WON'T STOP - INVESTIGATE
920504115	1	VALVE 11SW21 FAILURE DESCRIPTION: VALVE 11SW21 HAS NO CLOSED INDICATION - TROUBLESHOOT
920511088	1	VALVE 1SJ68 FAILURE DESCRIPTION: 1SJ68 GREASE UNSAT - REFURBISH OPERATOR
920511098	1	VALVE 1SJ40 FAILURE DESCRIPTION: 11SJ40 GREASE UNSAT - REFURBISH OPERATOR
920511099	1	VALVE 11RH29 FAILURE DESCRIPTION: 11RH9 GREASE UNSAT - REFURBISH OPERATOR

SAFETY RELATED MAINTENANCE
MONTH: - JUNE 1992

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: JULY 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

(cont'd)

WO NO	UNIT	EQUIPMENT IDENTIFICATION	
920522157	1	1 AUX ANNUNCIATOR	
		FAILURE DESCRIPTION:	1 AUX ANNUNCIATOR DID NOT ALARM - TROUBLESHOOT & REWORK
920525016	1	1A REACTOR TRIP BREAKER	
		FAILURE DESCRIPTION:	REPLACE THE UNDERVOLTAGE TRIP DEVICE
920530125	1	1A D/G OUTPUT BREAKER	
		FAILURE DESCRIPTION:	1A D/G OUTPUT BREAKER SPRINGS NOT CHARGING - TROUBLESHOOT
920607088	1	VALVE 1CV277	
		FAILURE DESCRIPTION:	1CV277 BLOWN DIAPHRAGM - REPAIR
920610098	1	13 CHARGING PUMP	
		FAILURE DESCRIPTION:	13 CHARGING SUCTION STABILIZER BLADDER LEAKS - REPLACE
920611212	1	VALVE 1CV41	
		FAILURE DESCRIPTION:	SIMULTANEOUS OPEN & CLOSED LIGHTS WITH VALVE IN CLOSED POSITION - INVESTIGATE
920613059	1	VALVE 1PR6	
		FAILURE DESCRIPTION:	INSPECT AND TROUBLESHOOT THE 1PR6 OPERATOR
920615207	1	VALVE 1CC131	
		FAILURE DESCRIPTION:	VALVE 1CC131 HAS YOKE MOVEMENT - REWORK AS REQUIRED
920617158	1	VALVE 13SW72	
		FAILURE DESCRIPTION:	13SW72 JAMMED - OPEN, INSPECT & REPAIR

SAFETY RELATED MAINTENANCE
MONTH: - JUNE 1992

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: JULY 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

(cont'd)

WO NO	UNIT	EQUIPMENT IDENTIFICATION	
920618108	1	VALVE 1SS64	
		FAILURE DESCRIPTION:	1SS64 HAS NO OPEN INDICATION - INVESTIGATE & REPAIR
920620085	1	VALVE 12CV104	
		FAILURE DESCRIPTION:	12CV104 WILL NOT OPEN - INVESTIGATE
920625090	1	12 CHILLER COMPRESSOR	
		FAILURE DESCRIPTION:	12 CHILLER COMPRESSOR WILL NOT START - TROUBLESHOOT & REPAIR
920627058	1	VALVE 1CC18	
		FAILURE DESCRIPTION:	VALVE 1CC18 FAILED STROKE TIME - INVESTIGATE
920006026	1	VALVE 1PR7 CABLES	
		FAILURE DESCRIPTION:	REPLACE 1PR7 CONTROL AND POWER CABLES

10CFR50.59 EVALUATIONS
MONTH: - JUNE 1992

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: JULY 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed and concurs with these evaluations.

ITEM	SUMMARY
A. Design Change Packages (DCP)	
DCP# 1EC-3138 Pkg. 1	"Radwaste Panel Upgrade" - This DCP will provide for revitalization upgrades to Rad Waste Panel 104 and related equipment on elevations 84' and 64' of the Auxiliary Building. The upgrades will consist of the following: a) replacement of the obsolete instrumentation, b) physical repair of Panel 104, c) provision of new labels to enhance operator interface with this panel, d) removal of unused components, including associated components external to the panel, e) upgrade of panel wire, cable restraints and labels, and f) functional verification of all panel instruments and components. Replacement of certain components used to monitor the administratively controlled transfer and release of radiological liquids and gasses will not affect the limiting conditions or safety margins as described in the Technical Specifications. (SORC 92-065)
DCP # 1EA-1030 Pkg. 1	"Replacement of 1CC924 Mark #D-2 with Mark #D-65" - The purpose of this design change is to revise the appropriate documentation to change valve 1CC94 from a mark no. D-2 to a mark no. D-65. This is a Component Cooling drain valve located in the no. 12 CC Heat Exchanger Room, which had been replaced under WO 881010102. The replacement of a non-nuclear valve with a nuclear class 3 valve ensures the integrity of the system. Installing the valves properly with a Code Job Package and performing the NDE ensures the integrity of the pressure boundary. This does not change the function of the system and does not impact the requirement of having two independent trains. Therefore, this change will not reduce the margin of safety as defined in the basis for the Technical Specifications. (SORC 92-067)

ITEM	SUMMARY
DCP# 1EC-3155 Pkg. 1	"Circulating Water Screen Wash Strainer Breaker" - The purpose of this change is to change the type and rating of the four circuit breakers that control the branch circuits used to feed No. 11, 12, 13, and 14 screen wash strainers. These circuit breakers are located at the 6A20, 6A22, 6A9 and 6A10 positions in the No. 11 Circulating Water Intake 230V Control Center Sect, No. 1 (located in CWI Control House #1). The change is to GE type TED 15 amp circuit breakers. This DCP does not reduce the margin of safety as defined in the basis for any Technical Specification because the change in screen wash pump strainer circuit breakers will not affect any of the operating limits necessary to protect the fission barriers inherent in the plant's design. (SORC 92-069)
B. Procedures and Revisions	
SC.SA-AP.ZZ-0012(Q)	"Technical Specification Surveillance Program Matrix" Rev. 16 - This proposal involves a revision to Station Administrative Procedure, SC.SA-AP.ZZ-0012(Q), Technical Specification Surveillance Program. The responsibility for performing Appendix "B" surveillance number 4.2.1 (Unit 1 & 2) has been changed from Licensing and Regulation to Chemistry/ Licensing and Regulation. This revision to the procedure does not change any previously analyzed testing requirement nor does it change any testing method, therefore the proposal cannot reduce the margin of safety as defined in the basis for any Technical Specification. (SORC 92-064)
TS1.OP-SO.AF-0001(Q)	"Auxiliary Feedwater System Operation" - The No. 13 Auxiliary Feedwater Pump (AFWP) recirculation line was cut by mistake in order to allow the Maintenance Department to rebuild the 13 AFWP turbine. This line is open ended back to the tank so there is no way to isolate the line to repair it by welding the pipe back together. This temporary procedure will allow the Operations Department to drain the contents of the Unit 1 Auxiliary Feed Water Storage Tank by filling the steam generators using the motor driven AFWP. The

ITEM	SUMMARY
	pumps are required to be operable in Modes 1, 2 and 3, however, they are not required to be operable in Modes 4, 5 and 6. The basis for these operability requirements is to be able to cool the plant down to less than 350°F from normal operating conditions even in the event of a loss of offsite power. When the plant is in Modes 5 or 6, the Reactor Coolant System temperature is less than 350°F and is being cooled by the Residual Heat Removal (RHR) System, so the basis for the Auxiliary Feed Technical Specifications does not apply. This temporary procedure does not reduce the margin of safety as defined in the basis of Technical Specification 3.7.1.2 because the plant can be maintained in a safe shutdown condition by the RHR system and auxiliary feed is not required. (SORC 92-064)
SCP1 Rev. 3	"Loss or Degradation of Physical Security System" - This procedure has been revised to incorporate the changed reference to security contingency event reporting pursuant to revised NRC guidance. The procedure also contains editorial changes which do not alter the procedure's intent or implementation. There is no security related accident analysis in the SAR. There is no connection between this procedure and any plant system. Therefore, there is no reduction to the margin of safety as defined in the basis for any Technical Specification. (SORC 92-067)
SCP2 Rev. 3	"Loss of Security Computer Power" - This procedure has been revised to incorporate the changed reference to security contingency event reporting pursuant to revised NRC guidance and to include Operations under the responsibilities section (for restoration of power). The procedure also contains editorial changes which do not alter the procedure's intent or implementation. There is no security related accident analysis in the SAR. There is no connection between this procedure and any plant system. Therefore, there is no reduction to the margin of safety as defined in the basis for any Technical Specification. (SORC 92-067)

ITEM	SUMMARY
SCP5 Rev. 3	"Security Threat" - The procedure has been revised to add extortion to the threat profile. A sentence is added on security force job knowledge, self control, and professionalism, particularly under stress, as contributing to the perception and reality of a secure work place. The systematic method of searching for destructive devices is changed to be compatible with current training methods. The procedure also contains editorial changes which do not alter the procedure's intent or implementation. There is no security related accident analysis in the SAR. There is no connection between this procedure and any plant system. Therefore, there is no reduction to the margin of safety as defined in the basis for any Technical Specification. (SORC 92-067)
SCP6 Rev. 3	"Discovery of Intruders or Attack" - The procedure for initial rapid deployment and follow up actions in response to a confirmed intrusion has been revised. The procedure also contains editorial changes which do not alter the procedure's intent or implementation. There is no security related accident analysis in the SAR. There is no connection between this procedure and any plant system. Therefore, there is no reduction to the margin of safety as defined in the basis for any Technical Specification. (SORC 92-067)
SCP10 Rev. 3	"Discovery of Destructive Devices or Evidence of Malicious Acts" - The revision changes assignments for making required notifications of the onset of the event, to reduce the burden on alarm station personnel during initial response. New emphasis is added on searching for additional devices and malicious acts, regardless of what caused the event to be declared. The procedure also contains editorial changes which do not alter the procedure's intent or implementation. There is no security related accident analysis in the SAR. There is no connection between this procedure and any plant system. Therefore, there is no reduction to the margin of safety as defined in the basis for any Technical Specification. (SORC 92-067)

ITEM	SUMMARY
SCP12 Rev. 3	"Security Alert" - The procedure has been revised to expand the discussion of possible basis for declaring a Security Alert and to add direction concerning use of defensive positions. The procedure also contains editorial changes which do not alter the procedure's intent or implementation. There is no security related accident analysis in the SAR. There is no connection between this procedure and any plant system. Therefore, there is no reduction to the margin of safety as defined in the basis for any Technical Specification. (SORC 92-067)

C. Temporary Modifications

TMR# 92-037	"Monitoring Temperatures at Different Locations Inside the Pressurizer Enclosure" - The pressurizer insulation was replaced with NUKON fiberglass type insulation, to reduce the temperature inside the pressurizer enclosure. This temporary modification proposes to monitor the temperature inside the pressurizer enclosure. The results will be used to evaluate the efficiency of the fiberglass insulation, determine the qualified life of cables inside the enclosure, and provide reliable means of evaluating heat stress conditions inside the pressurizer enclosure. The temperature monitoring device does not perform any safety related function and does not affect any safety related system required for the safe shutdown of the Salem Unit. There is no reduction to the margin of safety as defined in the basis for any Technical Specification. (SORC 92-065)
TMR# 92-039 Rev. 1	"Transfer Unit 2 BAST to Unit 1 BAST" - The purpose of this temporary modification is to cross connect Unit 1 and Unit 2 Boric Acid storage Tanks (BAST) and be able to pump Unit 2 BAST to Unit 1 BAST to meet Tech Spec requirements on Unit 1. The Technical Specifications dealing with required boration flowpath were reviewed. The flowpath is from the Boric Acid Storage Tank (BAST) to the Boric Acid Transfer Pump (BATP) to the Reactor Coolant System (RCS), or Refueling Water Storage

ITEM	SUMMARY
	Tank (RWST) to a Charging Pump to the RCS. In Mode 5, only 1 flowpath is required. Unit 1 and Unit 2 are both in Mode 5 and are using the flowpath from the RWST as the required flowpath. The BAST and BATP are not required in Mode 5 if the RWST is lined up for injection. This TMOD will not impair this flowpath from the RWST. There is no reduction in the margin of safety as defined in the basis for any Technical Specification. (SORC 92-068)

D. Deficiency Reports (DR) - "Use-As-Is"

DR SMD 92-643	"Repair of 1PS-28" - The purpose of this DR is to evaluate taking a 15 mils skim cut on the stem of 1PS28, some indications still remain in back seat area. Since the correct stem is not available to meet the outage schedule, this DR and safety evaluation accepts use-as-is disposition for 1 cycle. It will be replaced with a new correct stem during 1R11. Use of P2R spray (1P53) or its isolation by 1PS28 is evaluated here. This activity, by itself, does not introduce a failure mode of the stem or reduce the margin of safety since the stem is usually designed with a factor of safety greater than 2, the 20 mil diameter reduction is acceptable. Additionally, the threaded section of the stem is minimum diameter reduced by depth of thread which is approximately 160 mils. Therefore, a skim cut of 20 mils in the area of packing is acceptable with no reduction in margin of safety as provided by original component design. (SORC 92-066)

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 1
JUNE 1992

SALEM UNIT NO. 1

The Unit was shutdown for the entire period for the 10th Refueling Outage.

REFUELING INFORMATION
MONTH: - JUNE 1992

DOCKET NO: 50-272
UNIT NAME: SALEM 1
DATE: JULY 10, 1992
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

MONTH JUNE 1992

1. Refueling information has changed from last month:
   YES X NO _____

2. Scheduled date for next refueling: OCTOBER 2, 1993

3. Scheduled date for restart following refueling: DECEMBER 13, 1993

4. a) Will Technical Specification changes or other license amendments be required?:
   YES _____ NO _____

   NOT DETERMINED TO DATE x

   b) Has the reload fuel design been reviewed by the Station Operating Review Committee?:
   YES x NO _____

   If no, when is it scheduled?: _____

5. Scheduled date(s) for submitting proposed licensing action:
   N/A

6. Important licensing considerations associated with refueling:

7. Number of Fuel Assemblies:

   a. Incore 193
   b. In Spent Fuel Storage 656

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4