



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038
Salem Generating Station

August 12, 1991

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original copy of the monthly operating reports for the month of July 1991 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Safety Related Maintenance
10CFR50.59 Evaluations
Operating Summary
Refueling Information

Sincerely yours,

General Manager
Salem Operations

RH:pc

cc: Mr. Thomas T. Martin
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19046

Enclosures

8-1-7.R4
210023

The Energy People

9108210058 910731
PDR ADDCK 05000311
R PDR

IE 24
111

AVERAGE DAILY UNIT POWER LEVEL

Completed by: Mark Shedlock

Docket No.: 50-311
 Unit Name: Salem #2
 Date: 8/10/91
 Telephone: 339-2122

Month July 1991

Day Average Daily Power Level
 (MWe-NET)

Day Average Daily Power Level
 (MWe-NET)

1	<u>1088</u>
2	<u>1066</u>
3	<u>1080</u>
4	<u>1074</u>
5	<u>1071</u>
6	<u>1061</u>
7	<u>1061</u>
8	<u>1098</u>
9	<u>1099</u>
10	<u>1058</u>
11	<u>1083</u>
12	<u>1068</u>
13	<u>1062</u>
14	<u>1086</u>
15	<u>1093</u>
16	<u>1060</u>

17	<u>1035</u>
18	<u>1064</u>
19	<u>1076</u>
20	<u>1076</u>
21	<u>1065</u>
22	<u>1058</u>
23	<u>1078</u>
24	<u>1052</u>
25	<u>1075</u>
26	<u>1069</u>
27	<u>995</u>
28	<u>1066</u>
29	<u>1072</u>
30	<u>1076</u>
31	<u>1078</u>

OPERATING DATA REPORT

Completed by: Mark Shedlock

Docket No: 50-311
 Date: 8/10/91
 Telephone: 339-2122

Operating Status

1. Unit Name	<u>Salem No. 2</u>	<u>Notes</u>
2. Reporting Period	<u>July 1991</u>	
3. Licensed Thermal Power (MWt)	<u>3411</u>	
4. Nameplate Rating (Gross MWe)	<u>1170</u>	
5. Design Electrical Rating (Net MWe)	<u>1115</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1149</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1106</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason <u>N/A</u>		

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744</u>	<u>5087</u>	<u>85896</u>
12. No. of Hrs. Rx. was Critical	<u>744</u>	<u>4847.6</u>	<u>56203.8</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>744</u>	<u>4823.3</u>	<u>54533.3</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>2540352.0</u>	<u>16208186.4</u>	<u>122201655.4</u>
17. Gross Elec. Energy Generated (MWH)	<u>841020</u>	<u>5396544</u>	<u>57128471</u>
18. Net Elec. Energy Gen. (MWH)	<u>807776</u>	<u>5175380</u>	<u>54383191</u>
19. Unit Service Factor	<u>100</u>	<u>94.8</u>	<u>63.5</u>
20. Unit Availability Factor	<u>100</u>	<u>94.8</u>	<u>63.5</u>
21. Unit Capacity Factor (using MDC Net)	<u>98.2</u>	<u>92.0</u>	<u>57.2</u>
22. Unit Capacity Factor (using DER Net)	<u>97.4</u>	<u>91.2</u>	<u>56.8</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>0</u>	<u>22.7</u>

24. Shutdowns scheduled over next 6 months (type, date and duration of each)
Refueling outage scheduled to start 1-4-92 and last 75 days.

25. If shutdown at end of Report Period, Estimated Date of Startup:
N/A

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH JULY 1991

DOCKET NO.: 50-311
UNIT NAME: Salem #2
DATE: 8/10/91
COMPLETED BY: Mark Shedlock
TELEPHONE: 339-2122

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE

¹
F: Forced
S: Scheduled

²
Reason
A-Equipment Failure (explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File
(NUREG-0161)

⁵
Exhibit 1 - Same
Source

SAFETY RELATED MAINTENANCE
MONTH: - JULY 1991

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1991
COMPLETED BY: J. FEST
TELEPHONE: (609) 339-2904

WO NO	UNIT	EQUIPMENT IDENTIFICATION
910514227	2	VALVE 21SW150 FAILURE DESCRIPTION: VALVE HAS HIGH STROKE TIME - INVESTIGATE
910521094	2	MAIN STEAM RADIATION MONITOR 2R46C FAILURE DESCRIPTION: RMS MONITOR 2R46C READS HIGH - INVESTIGATE
910711080	2	21 SERVICE WATER PUMP FAILURE DESCRIPTION: BEARING LUBE OIL SUPPLY VALVE 21SW8 CLOSES - INVESTIGATE
910718181	2	AIR PARTICULATE DETECTOR FAILURE DESCRIPTION: NO INDICATED FLOW WITH THE PUMP RUNNING - TROUBLESHOOT
910723208	2	CONTROL ROOM INTAKE RADIATION MONITOR 2R1B FAILURE DESCRIPTION: MONITOR 2R1B SPIKED - INVESTIGATE
910723244	2	RADIATION MONITOR 2R45C FAILURE DESCRIPTION: MONITOR 2R45C SPIKED INTO ALARM - INVESTIGATE
910724072	2	PLANT VENT STACK FLOW RECORDER FAILURE DESCRIPTION: RECORDER 2FA8605/2 FAILED - INVESTIGATE
910729116	2	2B DIESEL GENERATOR FAILURE DESCRIPTION: 2B DG LUBE OIL HEATER WILL NOT STAY ENERGIZED - INVESTIGATE

10CFR50.59 EVALUATIONS
MONTH: - JULY 1991

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1991
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

The following items were evaluated in accordance with the provisions of the Code of Federal Regulations 10CFR50.59. The Station Operations Review Committee has reviewed these evaluations and agrees with their conclusions that these items do not present Unreviewed Safety Questions (USQs).

ITEM

SUMMARY

A. Design Change Packages

2EC-3053 Pkg. 1

"2R19 Detector and Well Replacement" - This modification is to replace detectors and detector wells and recalculating alarm and warning setpoints for the 2R19A,B,C and D radiation monitors. The margin of safety is not reduced by ensuring that the setpoints are calculated using the correct detector sensitivities and are with the limits specified in the Offsite Dose Calculation Manual and the Technical Specifications. (SORC 91-052)

2EC-3054 Pkg. 1

"2R37 Detector and Well Replacement" - This modification is to replace the detector and detector well and recalculating alarm and warning setpoints for the 2R37 radiation monitor. The margin of safety is not reduced by ensuring that the setpoints are calculated using the correct detector sensitivities and are with the limits specified in the Offsite Dose Calculation Manual and the Technical Specifications. (SORC 91-052)

2EC-3049 Pkg. 1

"2R18 Monitor Setpoint Calculation" - Package 1 proposes adjustment of the setpoint so that it falls within the envelope of the instantaneous release limits listed in the Offsite Dose Calculation Manual. The DCP merely recalculates the alarm and warning setpoints based on variables that have been investigated for validity (specifically "sensitivity", "dilution rate" and "liquid effluent release rate") and have been conservatively derated ("dilution rate" and "liquid effluent release rate") where applicable. This ensures that setpoints are well within the guidelines set by the Offsite Dose Calculation Manual. (SORC 91-55)

10CFR50.59 EVALUATIONS
MONTH: - JULY 1991

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1991
COMPLETED BY: J. FEST
TELEPHONE: (609) 339-2904

(Cont'd)

ITEM	SUMMARY
------	---------

B. Temporary Modifications

TMR 91-041

"24GB-3 Temporary Leak Repair" - This modification proposes a temporary leak repair be applied to stop a body to bonnet leak on valve 24GB-3. The valve will remain fully functional since there is no breach of the pressure boundary. (SORC 91-057)

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT 2
JULY 1991

SALEM UNIT NO. 2

The Unit began the period operating at full power and continued to operate at essentially full power throughout the remainder of the period.

REFUELING INFORMATION
MONTH: - JULY 1991

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: AUGUST 10, 1991
COMPLETED BY: J. FEST
TELEPHONE: (609)339-2904

MONTH JULY 1991

1. Refueling information has changed from last month:
YES _____ NO X
2. Scheduled date for next refueling: JANUARY 4, 1992
3. Scheduled date for restart following refueling: MARCH 15, 1992
4. a) Will Technical Specification changes or other license amendments be required?:
YES _____ NO _____
NOT DETERMINED TO DATE x
- b) Has the reload fuel design been reviewed by the Station Operating Review Committee?:
YES _____ NO x
If no, when is it scheduled?: December 1991
5. Scheduled date(s) for submitting proposed licensing action:
N/A
6. Important licensing considerations associated with refueling:

7. Number of Fuel Assemblies:
a. Incore 193
b. In Spent Fuel Storage 316
8. Present licensed spent fuel storage capacity: 1170
Future spent fuel storage capacity: 1170
9. Date of last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: March 2003