



PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Salem Generating Station

August 11, 1989

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specification, the original copy of the monthly operating reports for the month of July 1989 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Safety Related Maintenance
Major Plant Modification
Operating Summary
Refueling Information

Sincerely yours,

L. K. Miller
General Manager -
Salem Operations

RH:sl

cc: Mr. William T. Russell
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Enclosures
8-1-7.R4

The Energy People

8908230291 890731
PDR ADOCK 05000311
R PDC

IE24
11

95-2189 (5M) 12-88

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-311
 Unit Name Salem # 2
 Date 8-10-89
 Telephone 609-935-6000
 Extension 4722

Completed by Art Orticelle

Month JULY 1989

Day Average Daily Power Level
(MWe-NET)

Day Average Daily Power Level
(MWe-NET)

1 1085
 2 1107
 3 1040
 4 1072
 5 1083
 6 1053
 7 1058
 8 1063
 9 1066
 10 1070
 11 1067
 12 1086
 13 1056
 14 1078
 15 945
 16 695

17 787
 18 922
 19 1045
 20 951
 21 1068
 22 1039
 23 1075
 24 1058
 25 1055
 26 1089
 27 1089
 28 1016
 29 1066
 30 1091
 31 1068

OPERATING DATA REPORT

Docket No. 50-311
 Date 8-10-89
 Telephone 935-6000
 Extension 4722

Completed by Art Orticelle

Operating Status

- | | | |
|--|--------------------|--------------|
| 1. Unit Name | <u>Salem No. 2</u> | <u>Notes</u> |
| 2. Reporting Period | <u>July 1989</u> | |
| 3. Licensed Thermal Power (Mwt) | <u>3411</u> | |
| 4. Nameplate Rating (Gross MWe) | <u>1170</u> | |
| 5. Design Electrical Rating (Net MWe) | <u>1115</u> | |
| 6. Maximum Dependable Capacity (Gross MWe) | <u>1149</u> | |
| 7. Maximum Dependable Capacity (Net MWe) | <u>1106</u> | |
| 8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason | <u>N/A</u> | |
| 9. Power Level to Which Restricted, if any (Net MWe) | <u>N/A</u> | |
| 10. Reasons for Restrictions, if any | <u>N/A</u> | |

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744</u>	<u>5087</u>	<u>68376</u>
12. No. of Hrs. Reactor was Critical	<u>744</u>	<u>4466</u>	<u>42821.8</u>
13. Reactor Reserve Shutdown Hrs.	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>744</u>	<u>4289.3</u>	<u>41411.9</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>2475830</u>	<u>14067854</u>	<u>78732951</u>
17. Gross Elec. Energy Generated (MWH)	<u>836079</u>	<u>4691864</u>	<u>42576164</u>
18. Net Elec. Energy Generated (MWH)	<u>789000</u>	<u>4504565</u>	<u>40479870</u>
19. Unit Service Factor	<u>100</u>	<u>84.3</u>	<u>62.6</u>
20. Unit Availability Factor	<u>100</u>	<u>84.3</u>	<u>62.6</u>
21. Unit Capacity Factor (using MDC Net)	<u>95.9</u>	<u>80.0</u>	<u>53.5</u>
22. Unit Capacity Factor (using DER Net)	<u>95.1</u>	<u>79.4</u>	<u>53.1</u>
23. Unit Forced Outage Rate	<u>0</u>	<u>15.7</u>	<u>27.4</u>
24. Shutdowns scheduled over next 6 months (type, date and duration of each)	<u>NONE</u>		

25. If shutdown at end of Report Period, Estimated Date of Startup:

N/A

UNIT SHUTDOWN AND POWER REDUCTIONS
 REPORT MONTH JULY 1989

Docket No. 50-311
 Unit Name Salem No.2
 Date 8-10-89
 Telephone 609-935-6000
 Extension 4722

Completed by Art Orticelle

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
0165	07-15-89	F	0	A	5	---	EG	GENERA	MAIN BUS COOLING PROBLEM
0166	07-17-89	F	0	A	5	---	EG	GENERA	MAIN BUS COOLING PROBLEM
0167	07-17-89	F	0	A	5	---	EG	GENERA	MAIN BUS COOLING PROBLEM
0169	07-19-89	F	0	A	5	---	HF	XXXXXX	22A CIRCULATOR

1
 F: Forced
 S: Scheduled

2 Reason
 A-Equipment Failure-explain
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Exam
 F-Administrative
 G-Operational Error-explain
 H-Other-explain

3 Method
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation of
 Previous Outage
 5-Load Reduction
 9-Other

4 Exhibit G
 Instructions
 for Prepara-
 tion of Data
 Entry Sheets
 for Licensee
 Event Report
 (LER) File
 (NUREG 0161)

5 Exhibit
 Salem as
 Source

PS&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 2

WO NO UNIT EQUIPMENT IDENTIFICATION

890609120

2 2R42A

FAILURE DESCRIPTION: 21 GAS DECAY TANK AREA RAD MON/FAILED/REPAIR.

890626092

2 2FT523

FAILURE DESCRIPTION: 2FT523 22 SG STM FLW/LOW IND/TROUBLESHOOT.

890630102

2 2LI935B

FAILURE DESCRIPTION: 22 ACCUM LVL/IND INCORRECT/TROUBLESHOOT.

890630103

2 2LI934B

FAILURE DESCRIPTION: 22 ACCUM LVL/IND INCORRECT/TROUBLESHOOT.

890706112

2 2R1B

FAILURE DESCRIPTION: CONTROL RM. DUCT RAD.MON./NO
COMMUNICATION TO CPU/REPAIR.

MAJOR PLANT MODIFICATIONS
REPORT MONTH JULY 1989

DOCKET NO. 50-311
UNIT NAME: Salem 2
DATE: August 10, 1989
COMPLETED BY: P. White
TELEPHONE: 609/339-4455

<u>*DCR NO.</u>	<u>PRINCIPAL SYSTEM</u>	<u>DESCRIPTION</u>
2EC-01381A	Condensate	This design change replaced the existing condensate pumps with higher capacity pumps and motors.
2EC-02245	Reactor Protection	This design change deleted the 1/4 Reactor Coolant Pump Breaker Open Anticipatory Trip Signal.
2SM-00637	Safety Injection	This design change drilled weep holes in the disks of the 21 and 22 SJ40 valves on the upstream side of the valve disks.

* Design Change Request

MAJOR PLANT MODIFICATIONS
REPORT MONTH JULY 1989

DOCKET NO: 50-272
UNIT NAME: Salem 2
DATE: August 10, 1989
COMPLETED BY: P. White
TELEPHONE: (609)339-4455

*DCR SAFETY EVALUATION 10 CFR 50.59

- 2EC-01381A This design change replaced the existing condensate pumps with higher capacity pumps. Installation of the new pumps and motors provides higher net positive suction head to the Steam Generator Feed Pumps (SGFP) and reduces the likelihood of a reactor trip as a result of low suction pressure to the SGFPs. There was no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.
- 2EC-02245 This design change deleted the 1/4 reactor Coolant Pump Breaker Open Anticipatory Trip Signal. This modification was made in accordance with the recommendations of the Westinghouse Owner's Group Trip Reduction Program to enhance the reliability of Unit operation. There was no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.
- 2SM-00637 This design change drilled weep holes in the disks of the 21 and 22 SJ40 valves on the upstream side of the valve disks. Drilling of the weep holes provides a thermal expansion path for fluid trapped between the two disks and helps to prevent valve binding. There was no change to any plant process or discharge or to the environmental impact of the plant. No unreviewed safety or environmental questions are involved.

* DCR - Design Change Request

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY -- UNIT NO. 2
JULY 1989

SALEM UNIT NO. 2

The Unit began the period operating at 100% power and continued to operate at essentially full power until July 15, 1989. During the period between July 15 and July 20, 1989, several load reductions were performed to accomplish necessary maintenance activities. On July 20, 1989, the Unit was restored to 100% power and continued to operate at essentially full power for the remainder of the period.

REFUELING INFORMATION

COMPLETED BY: P. White

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: August 10, 1989
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month JULY 1989

- 1. Refueling information has changed from last month:
YES _____ NO X
- 2. Scheduled date for next refueling: March 30, 1990
- 3. Scheduled date for restart following refueling: May 15, 1990
- 4. A) Will Technical Specification changes or other license amendments be required?
YES _____ NO _____
NOT DETERMINED TO DATE X
- B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
YES _____ NO X

If no, when is it scheduled? February 1990

- 5. Scheduled date(s) for submitting proposed licensing action:
January 1990
- 6. Important licensing considerations associated with refueling:
NONE

7. Number of Fuel Assemblies:

A) Incore	<u>193</u>
B) In Spent Fuel Storage	<u>224</u>

8. Present licensed spent fuel storage capacity: 1170

Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: March 2003