



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

July 10, 1989

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

SALEM GENERATING STATION  
LICENSE NO. DPR-70  
DOCKET NO. 50-272  
UNIT NO. 1  
LICENSEE EVENT REPORT 89-025-00

This Licensee Event Report is being submitted pursuant to the requirements of the Code of Federal Regulations 10CFR 50.73(a)(2)(i)(B). This report is required within thirty (30) days of discovery.

Sincerely yours,

A handwritten signature in cursive script that reads "L. K. Miller/pw".

L. K. Miller  
General Manager -  
Salem Operations

MJP:pc

Distribution

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1/1

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The Energy People

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>Salem Generating Station - Unit 1</b>	DOCKET NUMBER (2) <b>0   5   0   0   0   2   7   2</b>	PAGE (3) <b>1   OF   0   4</b>
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TITLE (4)  
**1R46 Radiation Monitoring System Channels Inoperable Due To Inad. Admin. Controls**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
06	09	89	89	025	00	07	10	89			05000

OPERATING MODE (9) <b>N/A</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)						
POWER LEVEL (10) <b>N/A</b>	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)			
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)			
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 368A)			
	<input checked="" type="checkbox"/> 20.406(a)(1)(iv)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(vii)(A)				
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(vii)(B)				
	<input type="checkbox"/> 20.406(a)(1)(vi)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)				

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME <b>M. J. Pollack - LER Coordinator</b>		AREA CODE <b>609</b>	<b>339-4022</b>

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 6/9/89, it was discovered that the Radiation Monitoring System (RMS) main steamline monitor (R46) channels drain line was isolated due to closure of valve 1MS217, "Steam Trap Isolation Valve". Therefore contrary to the requirements of Technical Specification 3.3.3.1b the channels were inoperable. Investigation revealed that the valve has probably been closed since installation of a design modification in July 1988. This design change connected the R46 channel steam discharge to the Condenser. The root cause of the 1R46 A-E channels inoperability has been attributed to inadequate administrative control. Upon completion of a design change, the responsible System Engineer is required to initiate updates to the computerized Tagging Request Information System (TRIS) data base. A sign-off is provided in the design change package to ensure TRIS updates are addressed. System Engineers have not necessarily verified that TRIS update requests have been incorporated in TRIS before signing the design change package. The design change program, in 1988, did not specifically require verification. The channels were returned to operable status on June 9, 1989 upon opening the 1MS217 valve. The computerized TRIS has been updated to identify the requirement to maintain the 1MS217 valve "normally open". The design change program requirements will be revised to require verification of TRIS update implementation by System Engineering. This event will be reviewed with Technical Department System Engineering personnel.

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PLANT AND SYSTEM IDENTIFICATION:

Westinghouse - Pressurized Water Reactor

Energy Industry Identification System (EIIS) codes are identified in the text as {xx}

IDENTIFICATION OF OCCURRENCE:

Radiation Monitoring System Channels 1R46 A-E Inoperable Due To Inadequate Administrative Control

Discovery Date: 6/9/89

Report Date: 7/10/89

This report was initiated by Incident Report No. 89-330.

CONDITIONS PRIOR TO OCCURRENCE:

N/A

DESCRIPTION OF OCCURRENCE:

On June 9, 1989, it was discovered by Operations personnel that the Radiation Monitoring System (RMS) {IL} main steamline monitor (R46) channels drain line was isolated due to closure of valve 1MS217, "Steam Trap Isolation Valve". Therefore contrary to the requirements of Technical Specification 3.3.3.1b the channels were inoperable.

Investigation revealed that the valve has probably been closed since installation completion of a design modification (package 1EC-2136) in July 1988. This design change connected the R46 channel steam discharge to the Condenser.

APPARENT CAUSE OF OCCURRENCE:

The root cause of the 1R46 A-E channels inoperability has been attributed to inadequate administrative control.

Upon completion of a design change, the responsible System Engineer is required to initiate updates to the computerized Tagging Request Information System (TRIS) data base. A sign-off is provided in the design change package to ensure TRIS updates are addressed. System Engineers have not necessarily verified that TRIS update requests have been incorporated in TRIS before signing the design change package. The design change program, in 1988, did not specifically require verification.

To facilitate TRIS modifications, an Operations Department form is completed. A copy of this form is sent back to the originator upon implementation of the TRIS modification. A review of the DCR involved in this event has shown that a sign-off was made for TRIS updates; however, the TRIS update form could not be found.

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ANALYSIS OF OCCURRENCE:

The 1R46 A-E RMS channels are used to monitor main steam line radioactivity. This is done to identify significant primary to secondary leakage (e.g., tube rupture). The 1R19 Steam Generator Blowdown RMS channels and the 1R15 RMS air ejector channel corroborate the 1R46 channels' indications. These channels have been operable for most of the time period when the R46 channels were inoperable.

The 1R46 A-E channels provide alarm indication. They do not provide any interlocking functions. Prior to installation of the design change, condensed Main Steam would be blown to the ambient atmosphere.

Technical Specification 3.3.3.1b Action Statement 23 addresses the required actions when the R46 channels are inoperable. It states:

"With the number of OPERABLE Channels less than required by the Minimum Channels OPERABLE requirements, initiate the preplanned alternate method of monitoring the appropriate parameter(s), within 72 hours, and:

- 1) either restore the inoperable Channel(s) to OPERABLE status within 7 days of the event, or
- 2) prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status."

The preplanned alternate monitoring method when more than one R46 channel is inoperable is to take direct dose rate measurements off of the main steamlines. This data is then used to estimate the off-site release rate.

No major primary to secondary releases occurred during the period in which the R46 channels were inoperable based on 1R19 and 1R15 radiation monitor data. Additionally, since the channels only provide indication with no protective function, this event did not affect the health or safety of the public. However, since the Technical Specifications were not fully complied with, this event is reportable in accordance with Code of Federal Regulations 10CFR 50.73(a)(2)(i)(B).

CORRECTIVE ACTION:

The channels were returned to operable status on June 9, 1989 upon opening the 1MS217 valve.

The computerized TRIS has been updated to identify the requirement to maintain the 1MS217 valve "normally open".

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CORRECTIVE ACTION: (cont'd)

The design change program requirements will be revised to require verification of TRIS update implementation by System Engineering.

This event will be reviewed with Technical Department System Engineering personnel.

*LK Miller/ps*

General Manager -  
Salem Operations

MJP:pc

SORC Mtg. 89-072